

200-BP-5 Operable Unit Treatability Test Report





United States Department of Energy Richland, Washington

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ACRONYMS

AmReC American Resources Corporation

B-5 Reverse Well
bgs
below ground surface
BHI
BiPO₄
BV

216-B-5 Reverse Well
below ground surface
Bechtel Hanford, Inc.
bismuth phosphate
bed volume

BV bed volume
BY-Cribs 216-BY Cribs
Concentration

COPC contaminants of potential concern DOE U.S. Department of Energy

DOE-RL U.S. Department of Energy, Richland Operations Office

DOW description of work
DQO data quality objectives
DWS drinking water standard

Ecology Washington State Department of Ecology
EIP Environmental Investigation Procedure
EPA U.S. Environmental Protection Agency

ERA expedited response action

ERC Environmental Restoration Contractor

FY fiscal year

GAC granular activated carbon

GFAA graphite furnace atomic adsorption

HEIS Hanford Environmental Information System HSRAM Hanford Site Risk Assessment Methodology

ILCR incremental lifetime cancer risk IRM interim remedial measure

ITS in-tank solidification K_{d} distribution coefficient LFI limited field investigation **MCL** maximum contaminant level **MTCA** Model Toxics Control Act **NPL** National Priorities List **PFP** Plutonium Finishing Plant PIF Plutonium Isolation Facility

PNNL Pacific Northwest National Laboratory

PRF Plutonium Reclamation Facility
PUREX Plutonium/Uranium Extraction
QAPjP quality assurance project plan

QC quality control

QRA Qualitative Risk Assessment
RBDA Risk-Based Decision Analysis
RCA Radiation Control Area

RCRA Resource Conservation and Recovery Act of 1976

REDOX reduction-oxidation
SpA specific activity
SrCl₂ strontium chloride

ACRONYMS (Continued)

TBP tributyl phosphate

TCLP toxic characteristic leach procedure
TDL Technology Development Laboratory

Tri-Party Agreement Hanford Federal Facility Agreement and Consent Order

UCL upper confidence limit

WESF Waste Encapsulation and Storage Facility

WHC Westinghouse Hanford Company

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1.0 INTRODUCTION

The 200-BP-5 Operable Unit was established in response to recommendations presented in the 200 East Groundwater Aggregate Area Management Study Report (AAMSR) (DOE-RL 1993a). Recognizing different approaches to remediation, the groundwater AAMSR recommended separating groundwater from source and vadose zone operable units and subdividing 200 East Area groundwater into two operable units. The division between the 200-BP-5 and 200-PO-1 Operable Units was based principally on source operable unit boundaries and distribution of groundwater plumes derived from either B Plant or Plutonium/Uranium Extraction (PUREX) Plant liquid waste disposal sites (Figures 1-1 and 1-2).

Following a rationale outlined in the *Hanford Past-Practice Strategy* (DOE-RL 1991), the AAMSR (DOE-RL 1993a) also identified two locations in the 200-BP-5 Operable Unit where high-priority candidate plumes were found and recommended accelerated action along either an expedited response action (ERA) or an interim remedial measure (IRM) pathway. At one location at the 216-B-5 Reverse Well (B-5 Reverse Well) site, ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr concentrations were identified as the plume contaminants; at the 216-BY Cribs (BY-Cribs) plume site, ⁶⁰Co, ⁹⁹Tc, cyanide, and nitrate were identified as the plume contaminants. Plutonium-239/240 and technetium-99 were considered to be the most important radionuclides because of their long half-lives. By virtue of its higher concentrations, the ⁹⁰Sr plume was recommended as a candidate for the ERA pathway but was grouped with the IRM-level contaminants, ¹³⁷Cs and ^{239/240}Pu, for ease of treatment (DOE-RL 1993a).

An agreement between the U.S. Department of Energy (DOE), the U.S. Environmental Protection Agency (EPA), and the Washington State Department of Ecology (Ecology), as documented in the *Hanford Federal Facility Agreement and Consent Order* (Tri-Party Agreement) (Change Control Form M-13-93-03) (Ecology et al. 1994), enacted these recommendations. It required the preparation of a treatability test plan by January 1994 and startup of pilot-scale pump-and-treat treatability test systems at each plume site by August 1994. The treatability tests, if successful, were to be the first step toward developing a full-scale IRM treatment system for each plume. The change also specified that ⁶⁰Co, ¹³⁷Cs, ^{239/240}Pu, ⁹⁰Sr, and ⁹⁹Tc would be the primary target contaminants. Cyanide and nitrate were considered to be secondary contaminants of concern.

The treatability test plan (DOE-RL 1995c) identified ion exchange as the treatment technology to be evaluated. It also indicated the need for improved understanding of both groundwater contaminant distribution and aquifer property evaluation at each plume. Treatability testing began on August 29, 1994; regular operations started on January 18; 1995, and were halted on May 29, 1995. This report summarizes the results of the treatability tests at the two sites and the aquifer characterization efforts. Additionally, the report provides information related to historic plume migrations that was deemed useful in planning future actions at the 200-BP-5 Operable Unit.

1.1 PURPOSE AND SCOPE

The pilot-scale treatability tests were performed to assess the following: (1) the ability of an aboveground pump-and-treat system to extract sufficient volumes of groundwater from the B-5 Reverse Well and BY-Cribs plumes to affect the groundwater contaminant concentrations, and (2) the effectiveness of aboveground ion-exchange treatment systems in removing cesium, cobalt, plutonium, strontium, and technetium contaminants from the extracted groundwater. During the

course of testing, additional elements were added to the scope. These activities provided data to support decisions on future remedial actions at the plumes.

Treatability tests were also intended to better determine the distribution of the respective contaminants at each plume and improve understanding of aquifer properties. Modifications to Rev. 0 of the treatability test plan specified drilling activities to determine the BY-Cribs plume location. A 200 National Priorities List (NPL) Agreement/Change Control Form (BHI-00203, dated November 9, 1994) detailed a number of specific activities to characterize the aquifer at both sites. Activities included additional well sampling, tracer tests, and in situ flow velocity tests. These activities are described in greater detail in Section 2.6.1.

A qualitative risk analysis (QRA) was proposed for fiscal year (FY) 1995 to assess contaminant risk to human health. An evaluation of the QRA process found it to be overly conservative and relatively costly to prepare. An alternate Risk-Based Decision Analysis (RBDA) approach to the QRA process was proposed by the Environmental Restoration Contractor (ERC) (BHI 1995b) and approved by the U.S. Department of Energy, Richland Operations Office (DOE-RL). The QRA and RBDA differed according to the conventions by which contaminant concentrations were assigned in the plume and how the plume was then rated. The QRA typically assigned the highest concentration of the target contaminant to the entire plume and then calculated the risk. From there the plume was then qualitatively rated as a low, medium, or high risk and applied to the entire area. The RBDA preserved spatial distribution from available well data and then calculated risk values for individual concentration values. The risks were then represented as risk contours on a map. In addition, the RBDA modeled the migration of contaminants to downgradient locations or specified points in time to assess the effects of natural attenuation on risk to potential future users of the ground water.

Per an agreement between the DOE, EPA, and ERC, additional text was developed for the treatability test report to address historical site operational conditions and knowledge and to relate it to the current 200-BP-5 Operable Unit data. Several groundwater reports over the years have indicated plume migrations and behavior that have not been previously analyzed in a coherent fashion.

1.2 SITE DESCRIPTION

The two plumes resulted from low-level radioactive liquid waste disposal activities associated with operations at the B Plant complex, located in the northwest corner of the 200 East Area. Waste was discharged to the two sites periodically during the first 30 years of Hanford Site operations. Both source sites are situated on the Separations Area Plateau at elevations of 189 to 207 m (620 to 680 ft) above sea level. The ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr plumes remain close to the B-5 Reverse Well, but the ⁶⁰Co and ⁹⁹Tc plume from the BY-Cribs has migrated north toward Gable Gap. Consequently, while the B-5 treatment system was set up at the original waste facility, the BY treatment system was set up at the center of the current plume, well 699-50-53A.

1.2.1 216-B-5 Reverse Well

The B-5 Reverse Well plume is centered around the B-5 Reverse Well, which is located about 305 m (1,000 ft) northeast of the 221-B Canyon Building at the B Plant facility (Figures 1-3 and 1-4). The 241-B-361 Settling Tank is located about 15.2 m (50 ft) southwest of the B-5 Reverse Well and was used to settle-out precipitated solids in the aqueous wastes before they entered the well. The reverse

well is flanked by a series of four wells that lie along a northwest-southeast line within 20 m (65 ft) of the B-5 Reverse Well. These wells were drilled or deepened in 1979 to 1980. The wells encounter the water table 85 m (280 ft) below the ground surface and extend to the bottom of the 15-m (48-ft) thick aquifer. The next nearest wells are 155 to 168 m (510 to 550 ft) northwest and 152 m (500 ft) east of the B-5 Reverse Well.

The pilot-scale treatability test system was set up in close proximity to the B-5 Reverse Well because the plumes were believed to be confined around the reverse well. As described in Section 4.1, based on well fitness-for-use testing, well 299-E28-23, located 1.2 m (4 ft) southeast of the B-5 Reverse Well, was chosen to be the extraction well. Well 299-E28-7 was chosen to be the return well after an ERC regulatory analysis determined that the originally selected return well, 299-E28-1, was outside the plume boundary. The treatment system was not designed to remove ⁹⁰Sr to below drinking water standards (DWS) (8 pCi/L or 5.67 x 10⁻⁸ ppb), and therefore reinjection at the well 299-E28-1 could spread contamination to an otherwise uncontaminated part of the 200 East Area aquifer [for comparison, the 1.2 pCi/L maximum contaminant level (MCL) for ^{239/240}Pu is 1.96 x 10⁻⁶ ppb and the 120 pCi/L MCL for ¹³⁷Cs is 1.38 x 10⁻⁶ ppb]. A site map and well as-builts are included in Appendix F.

1.2.2 216-BY Cribs

The BY-Cribs plume is centered at well 699-50-53A, located 914 m (3,000 ft) north of the source BY-Cribs and 241-BY Tank Farm (Figure 1-5). From well fitness-for-use testing (see Section 4.1), it was determined that well 699-50-53A was the only candidate for the extraction well. This site was selected based on available groundwater chemistry data from local wells that indicated that the area of greatest contaminant concentration was centered at that location. The pilot-scale system was set up around well 699-50-53A. Well 699-49-55A, approximately 610 m (2,000 ft) to the west-southwest, was chosen for reinjection when fitness-for-use testing proved its suitability as a return well. Other nearby wells were 1,067 to 1,219 m (3,500 to 4,000 ft) away. More distant wells defining the plume are scattered irregularly on 457- to 610-m (1,500- to 2,000-ft) centers. The distance between wells was not considered optimal, but efforts to site and drill a well closer to the extraction well (see Section 2.6.2) were unsuccessful. A map location and well as-builts are included in Appendix F.

1.3 TREATMENT TECHNOLOGY DESCRIPTION

Ion exchange was selected as the treatment technology for both 200-BP-5 Operable Unit pilot-scale treatability tests. Ion exchange as a unit process is commonly applied in chemical processing, hydrometallurgy, and water and waste treatment as a technique for sorbing soluble metal species from solution. Cesium, cobalt, plutonium, strontium, and technetium are transition metals that are amenable for removal by ion exchange. This treatment technology is a proven process that does not require major technological development and is easily implemented. It has been evaluated at radioactively contaminated Superfund sites (EPA 1990) and the Oak Ridge National Laboratory (Robinson 1990). Ion exchange has been used in previous treatment studies and systems at the Hanford Site (Ames 1959, Mercer 1960, Delegard et al. 1986, Barney et al. 1992) and numerous municipal and industrial water treatment applications (Baker et al. 1988; Sorg 1989, 1991; Jelinek and Sorg 1988; Del Cul et al. 1992). These applications demonstrate that ion exchange is an effective treatment process for removing radionuclides from both water and groundwater.

Ion-exchange technology removes ions from solution by adsorption onto a solid medium (e.g., ion-exchange resin). Typically, with groundwater treatment, extracted groundwater is prefiltered and then pumped through a series of two or more beds (ion-exchange columns) of adsorbent where the ionic species in the groundwater exchange with or are adsorbed onto the surface of the adsorbent, thereby removing them from solution.

Generally, ion-exchange treatment systems consist of three columns that are plumbed to provide flow through any two columns at one time in a lead-polish mode, while a third column is offline for resin regeneration or resin replacement. The lead column provides primary removal of the contaminant(s) while the polish column "polishes" the discharge from the lead column to achieve a high-quality effluent. The columns are operated in a rotational basis. This arrangement provides for continuous operation in that when the lead column reaches breakthrough, it is valved out and the polish (or second) column is switched to lead position and the standby column is switched into polish position.

If the objective of ion-exchange treatment is to provide a high-quality effluent (achieve a very low target contaminant level) the lead column is generally run to 30% to 50% breakthrough to ensure that the polish column can achieve the overall desired removal. An option is to use a third column in series, then the lead column can be loaded to saturation. However, this increases system pressures and costs. The cost savings for increased loading of the lead column may not be offset by the installed cost of the third column (the overall system would require four columns for continuous operation).

The 200-BP-5 pilot-scale treatment systems used two to three ion-exchange columns in a series-flow arrangement. At the B-5 Reverse Well plume site, two lead columns of different media were required to remove the three contaminants followed by a mixed bed polishing column. At the BY-Cribs plume site, a typical two-column arrangement was used.

Resins can be regenerated, but this requires a large volume of water and regenerant solutions (acids, salt, or caustic solutions) and creates large volumes of spent solutions that require further treatment before disposal. If there were a market or demand for the recovered material, regenerating the resin might be beneficial. The 200-BP-5 Treatability Test Plan (DOE-RL 1995c) specified that the spent resins would be dewatered, removed, and drummed for disposal (see Sections 2.2.1, 3.1). Resins were sampled and analyzed for waste characterization (Section 3.1.2.3).

To properly select an adsorbent for use in an ion-exchange treatment system, a qualitative prediction of its sorptive behavior must be made. This involves determining the type and ionic species of the contaminants that might be encountered so that an adsorbent with the proper functional group can be identified. A review of the historical processes and waste treatment operations of the facilities contributing the wastes and hydrochemical characteristics of the groundwater is helpful in determining the ionic species. The groundwater in the 200 Areas contains dissolved solids resulting from both natural and man-made processes. It also contains a significant quantity of free, or dissolved, oxygen, thus creating an oxidizing chemical environment.

1.3.1 216-B-5 Reverse Well Site Test

The B-5 Reverse Well was used for the disposal of medium-level radioactive wastes from B Plant starting in 1945 and received approximately 303 x 10⁷ L (8 x 10⁶ gal) until its abandonment in 1947. Waste liquids were derived from fuel rod dissolution and waste neutralization processes at the 221-B

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Building and from waste neutralization processes at the 224-B Concentrator Building. A precipitate in the waste stream was allowed to settle in the 241-B-361 Settling Tank, located just upstream of the well. The residual waste stream containing reduced levels of cesium, plutonium, and strontium was then allowed to overflow to the ground via the B-5 Reverse Well. Contaminant activities and contaminant plume data are included in Table 1-1.

Plutonium has four oxidation states: (III), (IV), (V), and (VI). Pu(V) is unstable, disproportionating to other valence states. Pu(III) is readily converted to Pu(IV) in the presence of oxidants. This processing generally ensured that the plutonium in plant waste discharges was in the Pu(IV) state. Moreover, when plutonium is in contact with soil sediments, the higher oxidation states are slowly reduced to the Pu(IV) state (Barney 1988). The plutonyl ion is unstable and, at the pH ranges encountered in the aquifer (pH 7 to 8.5), generally combines with carbonates, hydroxides, and sulfates. Therefore, plutonium exists mainly as anions in hydroxyl and carbonate complexes in the form PuO₂(CO₃)₃⁵ or Pu(OH)₅ (Barney et al. 1992), or in the nitrate form Pu(NO₃)₆² (Ryan 1960). Cations and uncharged species may predominate at lower pH values, but generally because of the cation-exchange capacity of the soil, cations are sorbed/exchanged from groundwater. Based on previous laboratory work (Barney et al. 1992) it was determined that the distribution coefficient (K_d) for plutonium on bone char (registered trademark of TIGG Corporation/Tate & Lyle Process Technology Specialty Chemicals, Greenock, Scotland) was about 30,000 mL/g of resin (i.e., plutonium was strongly attracted to and held by the bone char). Column tests concluded that bone char could process 83,278 L (22,000-gal) of Plutonium Finishing Plant (PFP) water (with a plutonium concentration of 100 pCi/L) per gallon of bone char to break through. Based on these results, bone char was selected for plutonium removal.

Strontium, an alkaline earth element, is an active metal, and is readily soluble in nitric acid, forming bivalent cations. Strontium is not readily extractable from nitrate solutions in solvent extraction processing, and therefore ends up in the raffinate (aqueous waste stream from solvent extraction columns containing the impurities) discharged as waste. Being bivalent, strontium is rather strongly adsorbed by cation-type adsorbents. Clinoptilolite [registered trademark of American Resource Corporation (AmReC)/Ash Meadow Clino] was selected as the adsorbent for 90Sr removal.

Cesium, also an alkaline metal element, forms few water-insoluble compounds and is readily soluble in nitric acid, forming a monovalent cation. Cesium is not appreciably extracted into organic solvents under the conditions of solvent extraction. However, several insoluble metal-ferrocyanide and ferricyanides act to precipitate and/or scavenge cesium from solution. The affinity of cesium for cation-exchange adsorbents is generally low as a result of its low ionic charge. However, one sulfonated cation resin (IR-100) (registered trademark of Rohm and Haas, Philadelphia, Pennsylvania) did indicate an unusually high affinity for cesium (Reactor Handbook 1961). Cesium has been shown to adsorb readily from alkaline solutions by certain complex silicate minerals such as the clay, montmorillonite, and the zeolite, clinoptilolite (Reactor Handbook 1961). Laboratory tests indicated satisfactory cesium removal by the natural zeolite clinoptilolite. Therefore, clinoptilolite was selected as the adsorbent for cesium removal.

The ion-exchange process system for the B-5 Reverse Well pilot-scale treatment test consisted of three ion-exchange columns in a series-flow arrangement: a bone char column followed by a clinoptilolite column followed by a mixed bed column with 50% bone char, 50% clinoptilolite.

1.3.2 216-BY Cribs Site Test

During the first years of Hanford Site operations, separation waste containing significant quantities of uranium was stored in the 241-Tank Farms. To reduce tank construction requirements and to recover uranium, the Uranium Recovery Program (URP) was initiated at the 221/244-U Buildings. This process captured most of the uranium but, in the process, concentrated significant quantities of fission products in the waste being returned to the tank farms. In the process, the URP generated waste at a 2:1 ratio for every gallon of tank waste brought in to the URP (Waite 1991). Evaporators in the Uranium Recovery Process and at the T and B tank farms were not able to keep up with the processing rate, and there was net loss in tank space (Anderson 1990). Plans were made to operate the tanks in a cascade and discharge the supernatant to the ground to prevent exhausting tank capacity. However, this was recognized to result in unacceptably high concentrations of radionuclides going to the ground and was not implemented.

An in-plant scavenging process that added potassium ferrocyanide to the waste stream before going to the tank farms was included in post-September 1954 waste treatment operations. Liquid effluents sent to the 241-BY Single-Shell Tank Farms prior to in-plant fission product removal were transferred to the 241-CR vault for ferrocyanide scavenging. Upon return, the waste was cascaded through a three-tank system with most of the precipitated cesium dropping out in the first tank. Overflow liquids were then discharged to the soil column at the BY-Cribs through the 216-B-43 to 216-B-49 Cribs between November 1954 and December 1957.

Cobalt-60 was not recognized in the groundwater until January 1956 when it was detected in groundwater samples near the BY-Cribs (Thomas et al. 1956). The presence of ⁶⁰Co in the groundwater was surprising because it was generally known to readily sorb onto soils and to be significantly retarded in moving through the soil column. Cobalt was assumed to have complexed with a large molecule or to exist in a nonionized form. Cyanide is highly mobile in groundwater and soil and was present in the waste solutions discharged to the BY-Cribs (Thomas et al. 1956). By the time ⁶⁰Co had been detected, the BY-Cribs had already been closed to further waste discharges because beta activity had been detected in the groundwater under the cribs (Kasza 1993). Regular groundwater sampling and analysis for ⁹⁹Tc was initiated in 1985 due to improved analytical methods. Contaminant plume data are presented in Table 1-1.

Two in-tank solidification (ITS) processing systems pumped more than 144,000,000 L (38,000,000 gal) of tank liquor condensate to the 216-B-50 and 216-B-57 Cribs, located about 61 m (200 ft) north of the 241-BY Tank Farm. With these systems, tank liquids were evaporated, extracted from the tank, and then condensed. The process condensate was sent to the soil column. ITS-1 was used in Tanks 241-BY-101 and 241-BY-102, while ITS-2 was used in Tank 241-BY-112. Tank liquids were received from the Waste Fractionization Process at the 221-B Plant where ¹³⁷Cs and ⁹⁰Sr were recovered from existing tank wastes.

Del Cul et al. (1992) and Barney (1994) have reported that technetium is predominantly in the pertechnatate form (TcO₄) an anion. They assume that this is because of the oxic (oxidizing) conditions of groundwater. Dissolved oxygen concentrations of 5.7 to 10.5 mg/L are reported in the Hanford Environmental Information System (HEIS) database for groundwater in the 200 Areas. It may also be because of the other conditions that provide a reducing environment in the groundwater at depths beyond 61 m (200 ft). Cobalt-60 normally exists as a divalent cation in acidic to mildly alkaline conditions and readily sorbs onto soil via cation exchange, much as the contaminants at the B-5 Reverse Well. However, cobalt can form anion or neutral complexes. Since cyanides have a

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strong affinity for forming metal complexes, it is thought that cyanide has complexed with the cobalt. A cobalt-cyanide complex in the anionic form Co(CN)₆⁴ would explain the increased mobility of cobalt.

Because many ion-exchange resin manufacturers have developed adsorbents with functional groups that have an affinity for transition metals and because the cobalt and technetium were thought to be in anionic form, Dowex 21K (registered trademark of DOW Chemical Company, Midland, Michigan), a strong base anion ion-exchange resin, was selected for evaluation at the BY-Cribs site. The BY-Cribs ion-exchange treatment system consisted of two ion-exchange columns in a series-flow arrangement.

1.4 GENERAL GEOLOGY AND HYDROGEOLOGY

The geology of the Hanford Site and surrounding area has been reported in several documents (Myers et al. 1979, Delaney et al. 1991, Reidel et al. 1992). The 200 East Area geology and hydrology is presented in detail in the AAMSR (DOE-RL 1993a), in Connelly et al. (1992), and Lindsey et al. (1992), among others. For the purposes of this treatability test report, a brief discussion is given to provide a foundation for understanding the geology and hydrologic conditions at the two treatability test sites and to provide a basis for understanding recommendations for future activities. Local geologic features affecting the groundwater flow system are also emphasized.

1.4.1 Topography

The 200-BP-5 Operable Unit encompasses the northeastern portion of the 200 Area Plateau located on the Cold Creek Bar (also known as Separations or Central Plateau) and the slope north to Gable Mountain (Figure 1-6). The plateau's northern slope drops more than 61 m (200 ft) to a flood channel and coulee system at the base of Gable Mountain. The B-5 Reverse Well site is located near the center of the plateau; the BY-Cribs site lies on the northern slope between deep flood channel depressions south of Gable Mountain. The presence of the flood channels substantiates some of the bedrock features interpreted from well logs, as discussed in the following section.

1.4.2 Stratigraphy

The 200-BP-5 Operable Unit is underlain by a thick sequence of Columbia River basalts that are, in turn, overlain by the Ringold and Hanford Formation sediments. A thin layer of eolian sediments mantel the surface. Figure 1-7 depicts the stratigraphic relationships of the sedimentary units.

The Elephant Mountain Member of the Columbia River Basalt Group forms the base for the upper, unconfined aquifer system throughout most of the 200-BP-5 Operable Unit. The Columbia River Basalt Group, which erupted approximately 17 to 6 million years ago, consists of a number of flood basalt flows typically 30 to 91 m (100 to 300 ft) thick that are separated by relatively thin interbeds of sediments belonging to the Ellensburg formation. The interbeds and interflow zones are typically the sites of confined regional aquifers.

Lying unconformably on the basalt under the southern part of the 200-BP-5 Operable Unit is the Ringold Formation (Figure 1-8). This sedimentary sequence is a regionally extensive unit filling the

Pasco Basin. It consists of river channel-deposited sands and gravels combined with overbank sand and silt deposits. Between the channel deposits are fresh-water lake mud deposits that tend to form less permeable layers for groundwater movement. In general terms, this formation is not found in the area north of the 200 East Area fenceline and south of Gable Mountain. The Ringold Formation tends to thicken to the south and west toward the Cold Creek Syncline, and its disappearance north toward Gable Mountain is attributed to erosion by the Missoula Floods and the ancestral Columbia River. Smith (1980) estimates the Ringold Formation to be approximately 27 to 30 m (90 to 100 ft) thick at the B-5 Reverse Well. Connelly et al. (1992) reinterpreted the Ringold Formation to be approximately 9 to 11 m (30 to 35 ft) thick in the vicinity of the B-5 Reverse Well. The Ringold Formation is not present in the area of the BY-Cribs plume.

The overlying Hanford formation is the result of sediment deposition associated with the cataclysmic Missoula Floods, the last of which occurred approximately 13,000 years ago. The formation consists of coarse gravel layers overlain by upward-fining deposits of gravels, sands, and silts. The sedimentary unit is extensive and fills eroded channels in the underlying Elephant Mountain Basalt and the Ringold Formation with very coarse gravels. Away from the channels, deposits are finer grained. The Hanford formation comprises approximately the upper 76 m (250 ft) of sediments at the B-5 Reverse Well site. It is the only formation found overlying the BY-Cribs plume site, where its thickness is dependent on the surface topography.

1.4.3 Structure

The major structural feature found in the basalts beneath the 200-BP-5 Operable Unit is the subsurface extension of the Umtanum Ridge anticline. The east-west-trending axis of this structure lies to several thousand feet north of the 200 East Area fenceline and is approximated by Gable Mountain and Gable Butte. The basalt surface under most of the operable unit dips gently to the south toward the Cold Creek Syncline, while the north limb dips steeply to the north.

The basalt has been locally eroded by the Missoula Floods and the ancestral Columbia River, leaving erosional scours or channels in the Elephant Mountain basalt. At least one erosional window through the basalt is recognized in the vicinity of well cluster 699-53-55 and well 699-55-55, where sediments up to 76 m (250 ft) thick are encountered. In the area of Gable Gap, Elephant Mountain basalt has been eroded to expose the older Pomona, Esquatzel, Asotin, and Umatilla Members. East of wells 699-50-53A and 699-52-54, basalt is found above the water table as either an eastward extension of the Gable Butte fold or as a less-eroded portion of the top of the Elephant Mountain flow.

1.4.4 Hydrogeology

The unconfined aquifer beneath the 200-BP-5 Operable Unit is a composite aquifer system composed of the sediments resting on the top of the Elephant Mountain basalt and open fractures within basalt flow. The saturated sediments range from 0 m at the margins of the subsurface basalt high near well 699-50-53A to 3 to 5 m (10 to 15 ft) thick at wells around the BY-Cribs and to thicknesses of 13.7 m (45 ft) at the B-5 Reverse Well. The total thickness is undefined due to erosional effects on the basalt. In general, the aquifer thickens to the south in response to the dip of the basalt. Typically, the aquifer is present in gravels of the Hanford formation north of the 200 East Area fenceline and in coarser portions of the Ringold Formation, south of the boundary (Figure 1-9).

Pre-Hanford groundwater flow was generally west to east. The flow pattern was modified with the start of Separation Area operations by the discharge of significant volumes of cooling water to surface ponds and the soil column. Water-table mounding associated with the 216-B-3 Pond operations was noted as early as 1951 by Bierschenk (1957), who recognized pond operations as a means of retarding radionuclide migration. Water-table rises of 3 to 5 m (10 to 15 ft) were reported as resulting from B Plant cooling water discharges of approximately 5,700 L/min (1,500 gal/min) to the B Pond system from 1944 to 1951. These levels increased by another 1.5 to 3 m (5 to 10 ft) with the onset of the PUREX Plant operations, during which an average 14,000 L/min (3,600 gal/min) was discharged to the 216-B-3 and 216-A-25 pond systems. The influence of pond discharges was noted at the B-5 Reverse Well in 1947 and is attributed to be the driving force for the BY-Cribs plume's northward migration. Water-table elevations fluctuated over time in response to changes in plant operations but remained at elevated levels until the recent termination of the PUREX Plant operations. Due to the reduced quantities of water discharged to the ponds, groundwater levels have declined at rates of 0.15 to 0.30 m/yr (0.5 to 1.0 ft/yr), and several wells are being affected. Because of the interference of numerous local and regional groundwater influences and the low hydraulic gradient over much of the 200 East Area, actual groundwater movement over time in and through the 200-BP-5 Operable Unit is not well documented.

Before the treatability test, aquifer properties were not well known at either of the extraction wells, and large variations were recognized for the two plumes. The best data available, as reported in the 200-BP-5 Treatability Test Plan (DOE-RL 1995c), indicated that tests performed near the 241-BY Tank Farm yielded an aquifer transmissivity value of 4,459 m²/day (48,000 ft²/day) and hydraulic conductivity values of 1,524 to 3,048 m/day (5,000 to 10,000 ft/day). Groundwater level measurements indicate gradients of 0.00006. Based on an assumed effective porosity of 20%, an average linear flow velocity of 0.45 m/day (1.5 ft/day) was calculated. Newcomer et al. (1992) reported a transmissivity value of 186 m²/day (2,000 ft²/day) and an equivalent hydraulic conductivity of 122 m/day (400 ft/day) for well 699-52-54 near the BY-Cribs plume.

The generally poor understanding of groundwater conditions and uncertainties expressed in the test plan conceptual models for the two plumes prompted a greater emphasis on characterizing groundwater conditions at the 200-BP-5 Operable Unit. Several contemplated characterization activities were discussed in the 200-BP-5 Treatability Test Plan (DOE-RL 1995c), while others were approved in a 200 NPL Agreement/Change Control Form (BHI-00203). Results of these activities are presented in Chapters 4.0 and 5.0.

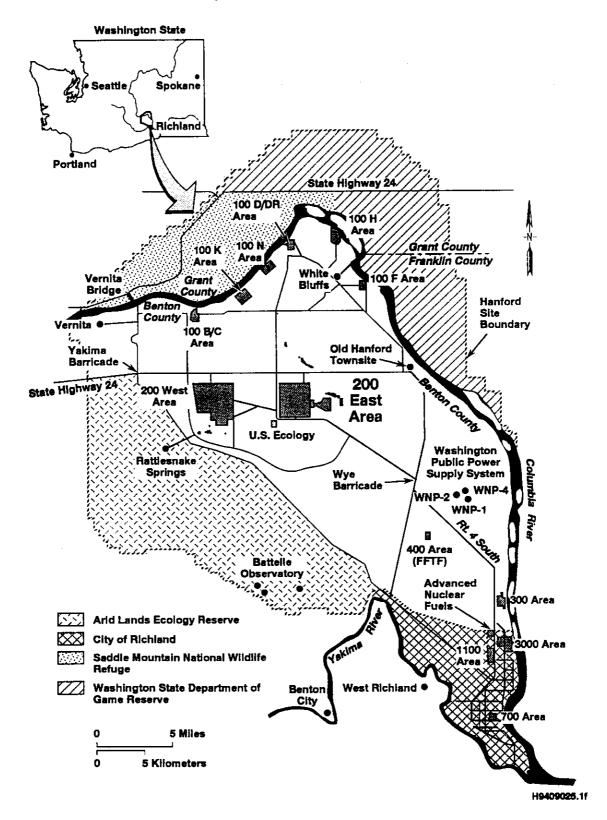
1.5 REPORT ORGANIZATION

This report follows EPA's general recommendations for content and organization of treatability test reports, but incorporates modifications developed in the 200-UP-1 Test Report (DOE-RL 1995c). The document also includes significant additions and changes in scope to address and support the data requested by the DOE, EPA, and Ecology. Chapter 2.0 discusses the test goals and data quality objectives (DQOs) in the 200-BP-5 Treatability Test Report that drove the design, operations, sampling requirements, and equipment requirements for the treatability test and test requirements for groundwater and contaminant distributions. It also includes those changes produced through agreements, unforeseen/unexpected conditions, and redirections from improved understandings of test facets.

The results of treatability testing and the associated laboratory and field test programs for both systems are reported in Chapter 3.0. Chapter 4.0 reports on the contaminant extraction and groundwater properties related to pumping during the treatment test. Chapter 5.0 summarizes historical data related to plume movement and presents revised conceptual models of the two plumes. It also estimates the risk associated with three 200-BP-5 Operable Unit plumes as reported in the Risk Based Decision Analysis for the 200-BP-5 Groundwater Operable Unit (BHI 1995c). Chapter 5.0 concludes with a discussion of the data gaps and data needs for potential future work. Chapter 7.0 provides conclusions and recommendations for potential future work for the 200-BP-5 Operable Unit. Appendices include tabulations of the raw data used in support of the discussions in Chapters 3.0 and 4.0.

NOTE: Throughout the text, the term MCLs is described as the concentration of a radionuclide in groundwater that is considered as the DWS. The concentrations used (60 Co = 100 pCi/L, 137 Cs = 120 pCi/L, $^{239/240}$ Pu = 1.2 pCi/L, 90 Sr = 8 pCi/L, 99 Tc = 900 pCi/L) were first delineated as a standard in Price (1986). Table C-2 of Appendix C in Price (1986) presents most of these values and references Table IV-2A of EPA (1976) as the source. The method of calculation has been changed, and the proposed new MCL values are not presented in this document. Thus, for project consistency, neither the terminology nor the cited MCL concentrations will be changed.

Figure 1-1. Hanford Site Map.



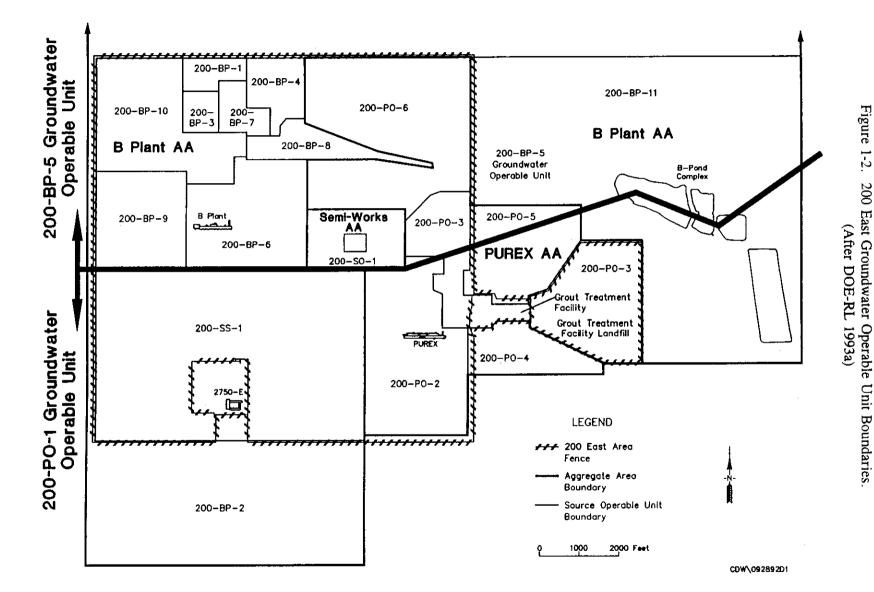


Figure 1-3. 200-BP-5 Plume Boundaries and Locations. (1993)

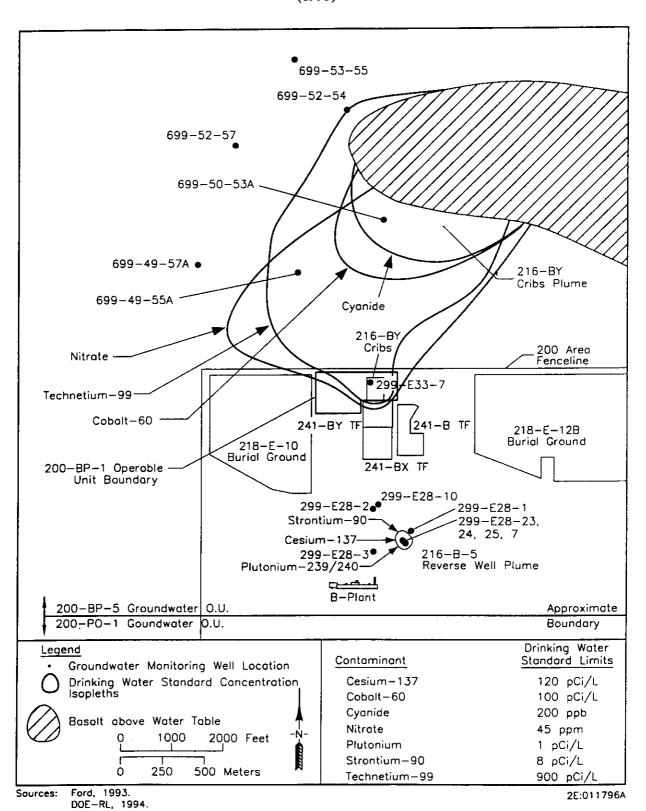
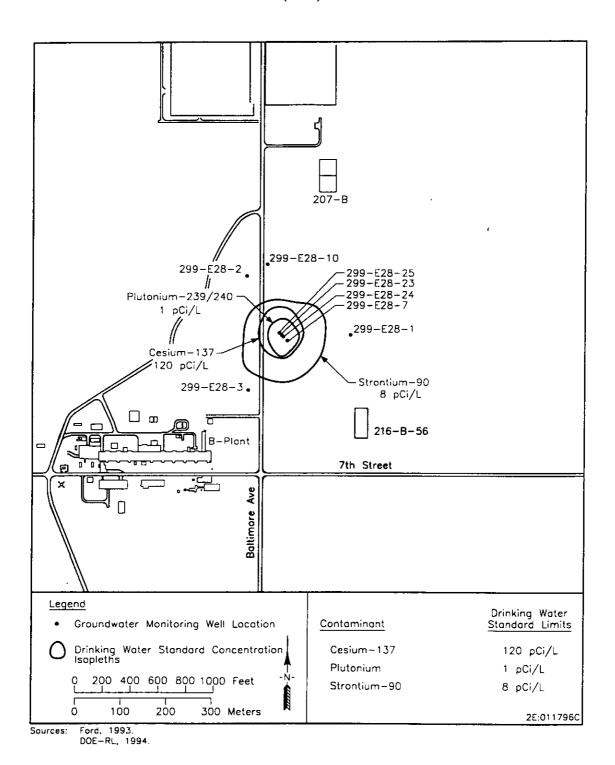


Figure 1-4. 216-B-5 Reverse Well Plumes. (1995)



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Figure 1-5. 216-BY Cribs Plume. (1995)

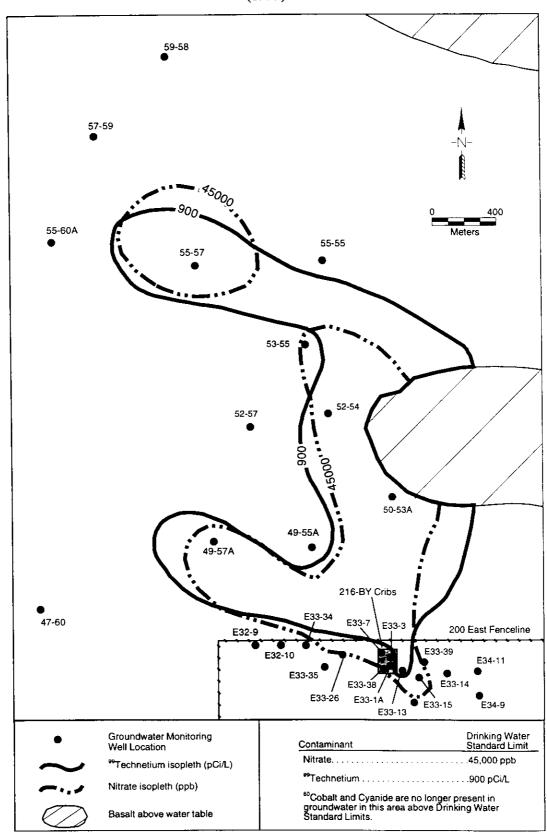


Figure 1-6. Topography and Geomorphology of the 200 Areas, Hanford Site. (Last et al. 1989)

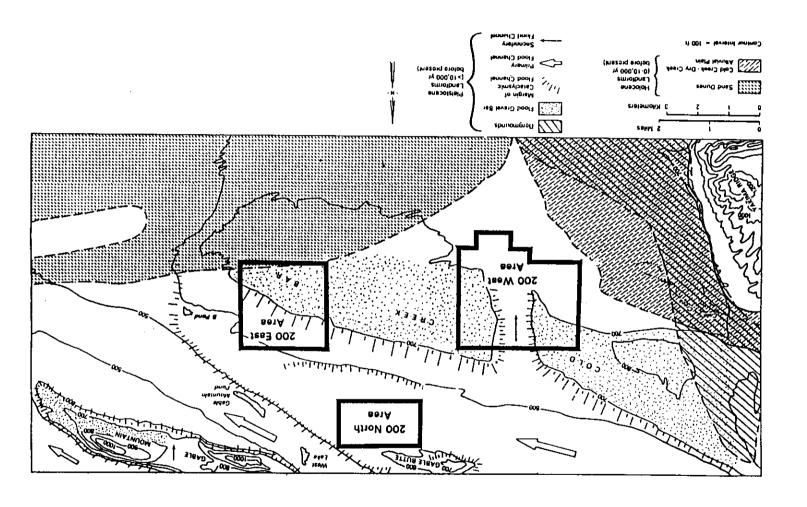


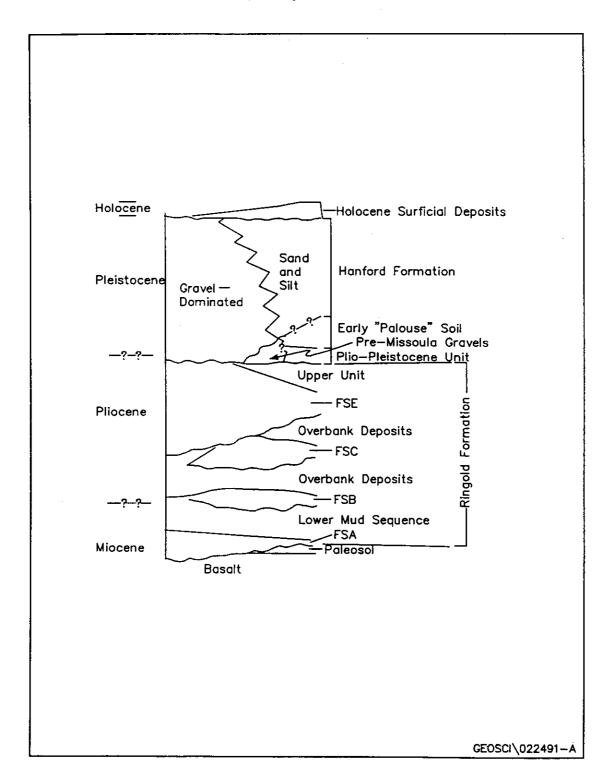
Figure 1-7. Generalized Stratigraphy of the Hanford Site. (DOE 1989)

Period	Epoch	Group	Formation	Isolopic Age	108 x 108	Member (Formal and Informal)	Sediment Stratigraphy or Basalt Flows						
QUATERNARY	Holocene				Sur	ficial Units	Loess Sand Dunes Altuvian and Altuvial Fans Talus Colluviam						
공	Pleisto- cene		Fan		Tou	chet beds Pasco gravels							
							Plio-Pleistocene unit						
	Plio- cene					Ringold	Formation						
				8.5	Ice	Harbor Member	basalt of Goose Island basalt of Martindale basalt of Basin City Levey interbed						
			Saddle Mountains Basalt	10.5	Ele	phant Mountain Member	basalt of Ward Gap basalt of Elephant Mountain Rattlesnake Ridge interbed						
			ıntai	12.0	Pomona Member		basalt of Pomona Selah interbed]					
			Σ		Esquatzel Member		basalt of Gable Mountain						
			율	13.5	Asotin Member		Cold Creek interbed basalt of Huntzinger						
			Sa	13.5	H		basalt of Humaniger	-					
					Wi	lbur Creek Member	basalt of Wahluke						
	1	ļ	1		Un	natilla Member	basalt of Sillusi basalt of Umatilla						
			-	14.5	-		Mabion interbed	-					
	ĺ	훙	뒿	물	퇑	탕	身			Priest Rapids Member		basait of Lolo basait of Rosalia	١,
		Columbia River Basalt Group	=				Quincy interbed	odiemos principal					
TERTIARY	906	asa	Nanapum Basah		Ro	oza Member	basalt of Roza Squaw Creek interbed	إل					
H	Miocene	i iii	<u> </u>		Г		basait of Lyons Ferry] [
щ	≥	\ `	를		1		basalt of Sentinel Gap	1					
]	ä	Ş		Fr	enchman Springs Member	basalt of Sand Hollow basalt of Silver Falls	}					
		팀	-				basait of Girkgo	┨ "					
		उ					basalt of Palouse Falls	1					
				15.6	Н		Vantage interbed	_					
	ŀ	ļ					basalt of Museum basalt of Rocky Coulee	4					
			sah.	1			basait of Hocky Coulee	4					
						Sentinel Bluffs Unit	basalt of Cohassett	1					
		1			N ₂		basalt of Birkett]					
	1	-	١٣	16.5	1		basalt of McCoy Canyon]					
			Į	10.5	1	Umtanum Unit	basalt of Umtanum	-					
	1	1	%			Stack Canyon Unit Ortley Unit	basalt of Benson Ranch	1					
			Grande Ronde Bas			Grouse Creek Unit		1					
			G.		R2	Wapshilla Ridge Unit		1					
	1	1	-	1	L	Mt. Horrible Unit							
		Ī			z	China Creek Unit							
				i	Ŗ.	Teepee Butte Unit Buckhom Springs Unit	-						
		1				I POWNIUM PRINGS VIII	1	1					
			Imnaha	17.5	-	ock Creek Unit		1					

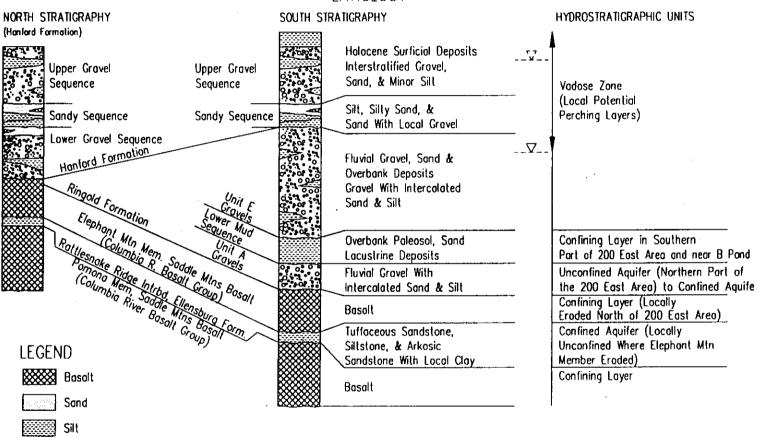
^{*}The Grande Ronde Basalt consists of at least 120 major basalt flows. Only a few flows have been named. N_2 , R_2 , N_1 and R_1 are magnetostratigraphic units.

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Figure 1-8. Generalized Stratigraphy of the Suprabasalt Sediments Beneath the Hanford Site. (Delaney et al. 1991)



LITHOLOGY



1-9. Conceptual Hydrogeologic Column for the 200 East Area (Delaney et al. 1991)

Potential Perching Layers
In Hanford Formation

__∇__ Groundwater Table

(Also Locally Intersects Hanford Formation Sandy Sequence & Ringold Formation Sediments;

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Table 1-1. Summary Data for the 200-BP-5 Operable Unit.

Plume	Contaminant	Quantity disposed*	Dissolved Plume Quantity ^b	Plume Area, 1992	Half life (yr) ^b	Drinking Water Standard	Mobility (k _d in mL/g) ^b
	∞Со	0.45 Ci	0.43 Ci	750,000 m ³	5.3	100 pCi/L	Low (2000)
216-BY Cribs	⁹⁹ Tc	[unknown]	21.9 Ci	1,500,000 m ²	213,000	4,000 pCi/L	High (0)
Plume	Cyanide	13,900 kg	985 kg	850,000 m ²		200 ppb	High (0.1)
	Nitrate	5,650,500 kg	740,000 kg	?		45 ppm	High (0)
	∞Sr	76 Ci	0.17 Ci	66,000 m²	28.5	8 pCi/L	Moderate (20)
216-B-5 Reverse Well	¹³⁷ Cs	81 Ci	0.014 Ci	22,000 m ²	30	120 pCi/L	Low (500)
Plume	^{239/240} Pu	2,000 g (ca. 300 Ci)	0.0006 Ci	19,000 m ³	24,400/ 6,560	1 pCi/L	Low (2000)

^aData on quantity disposal from Maxfield (1979), Smith (1980), and *Phase I Remedial Investigation Report for* 200-BP-1 Operable Unit (DOE-RL 1993d). Plutonium converted from mass to activity according to average specific activity for ²³⁹Pu and ²⁴⁰Pu; cyanide data converted from ferrocyanide data.

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^bData on plume quantity, half life, mobility, and primary decay mode from 200 East Groundwater Aggregate Area Management Study Report (DOE-RL 1993a).

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2.0 TREATABILITY TEST APPROACH

This chapter describes how the treatability test implemented the requirements of the 200-BP-5 Treatability Test Plan (DOE-RL 1995c). The treatability test objectives, presented in Chapter 3.0 of the test plan, are restated in Section 2.1. Section 2.2 discusses the experimental approach and methods used to meet these objectives. Section 2.3 provides a description of the equipment used in the tests. The sampling and analysis program is described in Section 2.4, and data management is discussed in Section 2.5. Significant deviations from the test plan are discussed in Section 2.6.

2.1 TEST OBJECTIVES

2.1.1 Test Performance Objectives

Test performance objectives and DQOs are used to clarify and guide the testing process and to outline the quality and quantity of data needed for this treatability test program. Test performance objectives identify information needs required to evaluate the pilot-scale pump-and-treat alternatives. The DQOs link the information requirements with the intended data uses to define the level of quality required for the measured variables. Test objectives were prepared for the pilot-scale treatment systems and groundwater extraction activities.

2.1.1.1 Treatment Test Performance Objectives. The primary test performance objective for the pilot-scale treatment systems was to determine the removal efficiency that could be achieved for the primary contaminants ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr in groundwater pumped from the B-5 Reverse Well plume, and ⁶⁰Co and ⁹⁹Tc in groundwater pumped from the BY-Cribs plume. Test performance objectives for the pilot-scale treatment system were grouped into three categories: (1) measurement of the effectiveness of ion-exchange treatment, (2) assessment of parameters or factors influencing the ion-exchange operation, and (3) determination of the resource needs associated with the test. Specific objectives under each of the three categories included the following.

Effectiveness:

- Determine the effectiveness of the ion-exchange system to consistently remove the following from the groundwater: ⁶⁰Co and ⁹⁹Tc at the BY-Cribs system and ¹³⁷Cs, ^{239/240}Pu at the B-5 Reverse Well system. Based on previous Hanford Site experience and general expectations for ion-exchange technology and consistent with the *Guide for Conducting Treatability Studies Under CERCLA* (EPA 1992), removal effectiveness should be at least 90% of the influent concentration.
- -- Identify optimum/preferred ion-exchange resin(s) for removing ⁶⁰Co, ¹³⁷Cs, ^{239/240}Pu, ⁹⁰Sr, and ⁹⁹Tc.
- -- Assess the impact and removal of secondary contaminants (e.g., cyanide, nitrate) in both plumes.

-- Assess the impact and removal of the radionuclides of interest that may be present as solids. Solids may contain radionuclides that will be removed by the prefilters and/or the resin beds. Ultrafine solids may pass through both the filter and the resin bed.

Although not an original objective of the test plan, it became important to assess the capability of the treatment system to remove ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr at the B-5 Reverse Well site to the MCL as defined by the Safe Drinking Water Act to enable reinjecting treated groundwater outside the contaminant plume(s).

Operating Parameters:

- -- Refine operational configurations, requirements, and procedures.
- -- Assess impacts of groundwater constituents on operational efficiency.
- -- Assess operating parameters (e.g., flow rates, residence times, pH) to optimize treatment efficiency.
- -- Demonstrate operational reliability and safety of an ion-exchange-based treatment system at a scale sufficient to allow scaleup to a full-size remedial system.

• Resource Requirements:

- Develop estimates of significant cost components, including equipment and material costs; resin contaminant exchange capacity, exhaustion rate, and cost; electrical power and utility costs; chemical costs and use rates; process residue and secondary waste management costs; maintenance requirements; and operator and personnel requirements.
- -- Refine health and safety requirements.
- 2.1.1.2 Contaminant Extraction Test Performance Objectives. At the time of test plan preparation, the ability to provide the B-5 treatment system with quantities of significantly contaminated groundwater was anticipated to be a problem. In general, the radionuclide concentrations from groundwater monitoring that prompted rating the B-5 Reverse Well plume as a significant hazard were not regarded as being sustainable for the life of the treatability test. This concern resulted from the expectation that radionuclides attached to the aquifer sediment would not reach an equilibrium condition with the groundwater at a rapid enough rate as less contaminated groundwater was drawn through the sediments to the extraction well. This would result in sending less contaminated water to the treatment system, making treatment a much less successful process. At the BY-Cribs plume, the highly mobile nature of the contaminants was not thought to present a similar problem.

Initially, contaminant extraction testing was intended to be performed only at the B-5 Reverse Well plume and not at the BY-Cribs plume. However, when April 1994 groundwater monitoring data showed significantly reduced concentrations of the target constituents, the requirement was applied to the BY-Cribs system. Specific contaminant extraction test procedures and results are described in Section 4.2.

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The primary test performance objective for the contaminant extraction portion of the treatability test in the B-5 and BY-Cribs plumes then was to assess the potential for recovering the primary contaminants, ¹³⁷Cs, ^{239/240}Pu, ⁹⁰Sr, ⁶⁰Co, and ⁹⁹Tc, that are associated with groundwater plume. If the contaminant recovery was satisfactory, an estimate of the long-term rate at which primary contaminants could be extracted from the aquifer was to be developed. This primary objective involved a number of ancillary objectives that were grouped into three categories: (1) measurement of the effectiveness of the contaminant extraction process, (2) assessment of aquifer parameters or operational procedures to allow optimization of the contaminant extraction process, and (3) determination of resource needs. Specific objectives under each of the three categories included the following.

• Effectiveness:

-- Determine the effectiveness of contaminant recovery for the primary contaminants from the aquifer matrix during operation of a pilot-scale pump-and-treat system (i.e., measure the amount of material extracted from aquifer).

Operating Parameters:

- -- Refine knowledge of aquifer hydraulic properties, primary contaminant distribution properties, and estimates of total contaminant quantities to allow prediction of long-term recovery effectiveness.
- -- If needed, optimize extraction pump cycling (pumping rates, aquifer reequilibration times, and the effect of changing extraction wells) to maximize recovery.
- -- If testing is successful and treatment continues, develop a computer model of local groundwater flow and primary contaminant phase transformation and transport to estimate aquifer and geochemical parameters and predict long-term system performance.

• Resource Requirements:

-- Develop estimates of significant cost contributors, including electrical costs of pumping, installation of additional wells, technical personnel time to interpret process effectiveness, and analytical costs.

As described in Section 2.6, not all test performance objectives were met.

2.1.2 Data Quality Objectives

To ensure that data collected were sufficient to meet test objectives, appropriate DQOs were developed.

2.1.2.1 Treatment Test Data Quality Objectives. DQOs and critical data requirements specific to laboratory-scale and pilot-scale treatability testing are provided in Tables 2-1, 2-2, and 2-3, respectively (tables and appendices referenced therein are based on DOE-RL 1995c). In general, data to assess treatment effectiveness and costs were considered most critical to meet test objectives and

2-3

required quantification through the use of quality control (QC) checks. Less critical was the assessment of operating parameters, which were intended primarily to support design optimization. As a result, only a qualitative engineering evaluation of operating parameters was performed.

Sampling and analytical requirements initially specified both lower level (Levels I and II) and higher level (Levels III and V) analyses. Field screening and field analysis techniques (Levels I and II) were originally planned for daily monitoring requirements and measurements. However, there are no on-line detection or field tests for radionuclides at the very low concentrations found at the two sites. Instead, Level V analyses at the 222-S Laboratory were relied on for process control. Confirmatory (Levels III and V) analyses were performed occasionally for critical parameters such as pre- and post-treatment contaminant concentrations to check the 222-S Laboratory results and to analyze for specific groundwater constituents.

2.1.2.2 Contaminant Extraction and Hydrogeologic Assessment Data Quality Objectives. Data collected during the contaminant extraction tests performed at both sites were initially intended to support developing a numerical model of aquifer and primary contaminant response to groundwater removal, if successful testing favored continued operations. With changes to the program, the DQOs for hydrogeologic assessment and groundwater monitoring in Tables 2-4, 2-5, and 2-6 were used. Data collected during contaminant extraction tests performed at the two sites were the same as those used to assess contaminant treatment effectiveness. Thus, they were considered critical to meet test objectives. These data were submitted for QC checks (confirmatory sampling) by Level III or V analyses. Specific DQOs for contaminant extraction testing at both sites are identified in Table 2-4.

The primary objectives of the hydrogeologic assessment of the two plumes were to further define the nature and lateral extent of contamination and further define the aquifer thickness. To achieve these primary objectives at the BY-Cribs plumes site, drilling activities proceeded in two phases. Phase I was a limited exploratory drilling and sampling program in the vicinity of well 699-50-53A. Upon a successful conclusion of Phase 1, Phase 2 was proposed to include installation of one extraction well, one return well, and up to eight additional observation wells, along with necessary aquifer testing and sampling. To avoid random well siting, a high-resolution seismic reflection survey was proposed to provide additional information regarding the placement of wells.

Phase 1 exploratory sampling consisted of drilling up to eight borehole/probes to the bottom of the unconfined aquifer (top of basalt) in a grid pattern around well 699-50-53A. Groundwater samples were collected at the bottom of the borehole/probe. The two main objectives of this exploratory program were to further define both the nature and extent of contaminants around well 699-50-53A and the aquifer thickness. The second objective of this exploratory program was to generate data that would guide the placement of additional observation, extraction, or return wells to support the treatability test.

2.2 EXPERIMENTAL DESIGN AND PROCEDURE

The test performance objectives outlined in Section 2.1.1 were met at each plume by conducting the following tests: a preliminary laboratory test suite to identify the most appropriate resins, a pilot-scale test of an ion-exchange-based treatment system, and field-based bench-scale testing of adsorbents. The laboratory and field bench-scale testing typically consisted of batch equilibrium and small-scale flow-through (mini-column) tests. The pilot-scale tests are large-scale flow-through column tests that are used to determine treatment system effectiveness, define parameters necessary

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for system optimization, and provide information about resource needs and costs for operating adsorbent-based (e.g., ion exchange) treatment systems on the Hanford Site. They are normally used to test a proposed design and only after all test variables and treatment alternatives have been worked out by laboratory tests. Table 2-3 summarizes the approach used to meet the test performance objectives for the pilot-scale treatment systems.

Pre-treatment laboratory studies, including batch equilibrium and mini-column tests, designated as resin selection tests, were performed at the IT Corporation's Knoxville, Tennessee, Technology Development Laboratory (TDL). Based on those tests, clinoptilolite, a natural zeolite, was selected for ¹³⁷Cs and ⁹⁰Sr removal at the B-5 Reverse Well pilot-scale test and bone char was selected for plutonium removal. Section 2.2.3.1 discusses those tests and procedures, and results are described in Chapter 3.0. At the BY-Cribs site, Dowex 21K was identified for ⁶⁰Co and ⁹⁹Tc removal and was evaluated in both mini-column tests and in the pilot-scale system.

Extensive field-based laboratory-scale tests were conducted at both sites. At the B-5 Reverse Well, batch equilibrium tests were used to help select a specific adsorbent(s) for removal of target contaminants. Batch equilibrium tests for the BY-Cribs site were not conducted because previous experience at the 200-UP-1 treatment system (DOE-RL 1995a) indicated that Dowex 21K would successfully remove the target contaminants (99Tc). Small-scale flow-through (mini-column) tests were conducted at both sites to provide additional data to supplement the data obtained from pilot-scale treatment system in order to more fully determine the treatment effectiveness of the treatment systems.

2.2.1 Pilot-Scale Tests

The pilot-scale treatment systems were operated to (1) collect data necessary to demonstrate treatment system effectiveness and define the operating parameters necessary for system optimization in the design of systems for a potential IRM Proposed Plan and (2) determine resource needs and costs for operating an adsorbent/ion-exchange groundwater treatment system at the Hanford Site.

Treatment system effectiveness is determined by comparing the influent versus the effluent concentrations of contaminants of concern, then determining the percent removal of the contaminants.

Percent Contaminant Removal = $\frac{Concentration\ In\ -\ Concentration\ Out}{Concentration\ In}$ x 100

By plotting the removal efficiencies for the various sampling points versus the volumes of water treated, a graphical representation of system performance, called a breakthrough curve, can be developed. Volume of water treated is commonly reported as bed volumes (BVs), which is equal to the volume of resin in a column, without regard to porosity.

The removal efficiency is commonly calculated for both the lead column and total system and graphed. When the removal efficiency drops below a specified level for a column (90% for the 200-BP-5 systems), the system is said to have reached breakthrough. At this point, the polishing column then is relied on to remove the target contaminant to agreed-to levels, such as MCLs. Typically, the system is run until the first column has reached 90% to 100% saturation, which provides input into the cost of adsorbent for future treatment. If the effluent from the polishing column has also begun to exceed established standards, either another column needs to be added to

the system or changeout of the adsorbent in the first column needs to occur before it reaches saturation.

Groundwater parameters were also monitored to determine whether they might impact removal efficiencies, in case the treatment system removal efficiencies were not as expected. In some cases, interfering ions may competitively load on the adsorbent(s), thereby shortening adsorbent life or reducing removal of the primary contaminants of concern and the ability to achieve a high-quality effluent. Parameters that were monitored included anions (Cl, F, NO₃, and SO₄), cations (calcium, iron, magnesium, potassium, sodium, and zinc), and alkalinity. In addition, the operating parameters that were monitored included flow rate and totalized flow, pH, temperature, pressure, dissolved oxygen, and turbidity. Residence/contact time is also an important factor that relates to flow rate.

Resource requirements for the pilot-scale tests were tracked and accounted for by the following categories. Some overlap between the categories exists and accounts for the differences between the sum of costs versus the budget. There is insufficient detail to break out the overlaps.

- Engineering/Design/Construction: Design and construction expenditures included labor and materials for design, well evaluation and remediation, procured equipment, fabrication, construction and construction supplies, and mobilization. Much of the construction work and site preparation was performed by other onsite contractors.
- Operations: Operations expenditures included labor and material costs for operation of the pilot-scale treatment systems. Items in this category included preparation of operating documents, site preparations, operations personnel, site health and safety monitoring, field supervision, and radiation monitoring. Operating materials included ion-exchange adsorbents, fuel, operating supplies, and safety equipment and support facilities (trailers, office equipment, etc.). In addition, field bench-scale testing is considered here.
- Analytical Support for Operations: Analytical support expenditures included analytical costs for both onsite and offsite laboratories. Analytical costs for process control, field bench-scale tests, waste classification, and groundwater are reported here.
- Waste Management: Waste management expenditures included costs for waste management and waste management plan preparation, disposal of regulated and nonregulated wastes, and costs for waste storage supplies and containers (drums, plastic liners, and materials).
- Maintenance and System Standby: Maintenance expenditures included equipment calibrations, equipment repair and replacement, well maintenance, and maintenance management. In addition, tank cleanout and winterization activities are also considered here.
- **Project Management**: Project management expenditures included labor and material costs associated with project oversight and support, preparation of regulatory and planning documents, safety assessments, and cost account management and support.
- **Project Support**: Technical labor and associated costs are tracked here in support of all phases of treatability testing. Exempt labor is the dominant expense item.

Other costs included are subcontractor support charges, general site services, indirects and overheads, and other miscellaneous costs not associated with the above categories.

- Groundwater Characterization: Groundwater characterization activities including drilling, groundwater sampling, and hydrologic testing are considered here. Subheadings include labor and material subcontracts and sampling analytical costs.
- 2.2.1.1 Treatment System Operation. The two pilot-scale treatment systems were designed according to nearly identical process flowsheets, except that the B-5 Reverse Well treatment system required two different adsorbents for the three contaminants. Process flow diagrams for the treatment systems are provided in Appendix A. The physical arrangement differed from the design in the treatability test plan because of the proximity of extraction and return wells to the treatment systems. Only one adsorbent for removal of the cobalt and technetium was required for the BY-Cribs treatment system.

Each system consisted of an extraction well, a 31,000-L (8,200-gal) influent storage tank, a skid-mounted influent pumping station (with filtration), a skid-mounted adsorbent/ion-exchange treatment system, a 31,000-L (8,200-gal) effluent storage tank, a skid-mounted effluent pumping station (with filtration), and a return well. The extraction systems were designed to operate 24 h/day to fill the influent storage tank and were provided with both automatic level-sensing shutoff switches and overflow return lines back to the extraction wells. The treatment systems' design processing rates of 95 to 189 L/min (25 to 50 gal/min) were chosen because it was a good size from which to scale-up a larger system and because it would provide appropriate cost data. The systems were typically operated only during day shift to process the water accumulated in the influent storage tank. Smaller systems were considered for both sites but lacked sufficient capacity for efficient use at other potential treatment sites. A granular activated carbon (GAC) adsorber was added to each system after the ion-exchange system to remove organic constituents suspected to have originated in the storage tanks (see Section 2.6.3.1). Each system used a programmable logic controller and annunciator panels for monitoring system status and safety interlocks.

B-5 Reverse Well Site: Groundwater was pumped from well 299-E28-23 to the 31,000-L (8,200-gal) influent storage tank. The well was capable of producing at least 132 L/min (35 gal/min), but the well pump was capable of delivering only 106 L/min (28 gal/min). Groundwater was pumped from the influent storage tank via the influent pumping station through filters to the ion-exchange treatment system. The treatment system consisted of four downflow columns: three were used as cation adsorber columns for removal of primary contaminants and the fourth for the GAC. Filtered groundwater was pumped to the first column containing bone char for plutonium removal; to the second column containing clinoptilolite for 137Cs and 90Sr removal; then to the third column containing 50% clinoptilolite and 50% bone char as a backup polishing column for all contaminants. The three columns were filled with 1.7 m² (18 ft³) of adsorbent, which resulted in a contact time of 5.4 min and a hydraulic surface loading rate of 1.9 L/min/m² (5.4 gal/min/ft²) in each column at a flow rate of 95 L/min (25 gal/min). (Bed volume is defined as the volume of adsorbent, 18 ft3; contact time is the bed volume divided by volumetric flow rate.) Treated groundwater was transferred to the 31,000-L (8,200-gal) effluent storage tank. The effluent pumping station consisted of a set of pumps and a set of filters to filter the treated water before disposal via the return well.

After processing approximately 4,000 BVs at the B-5 Reverse Well, the clinoptilolite column and the clinoptilolite-bone char column media were changed. The initial run was designated as "Run 1." Both columns were recharged with clinoptilolite, and the flow was reduced to

56.8 L/min (15 gal/min) to provide a contact time of 8 minutes for another run, designated as "Run 2."

• **BY-Cribs Site**: Groundwater was pumped from well 699-50-53A to the 31,000-L (8,200-gal) influent storage tank. Because the aquifer at the BY-Cribs site is so thin, well 699-50-53A was only capable of producing 3 to 4 gal/min. Groundwater was pumped from the influent storage tank via the influent pumping station through filters to the ion-exchange treatment system. The ion-exchange treatment system consisted of three downflow columns: two were used as ion-exchange columns and the third was used as a GAC column. Both ion-exchange columns used Dowex 21K for the removal of ⁶⁰Co and ⁹⁹Tc. The ion-exchange columns were filled with 16 ft³ of Dowex 21K, which resulted in a 5-minute contact at 95 L/min (25 gal/min).

Treated groundwater was transferred from the ion-exchange treatment system to the 31,000-L (8,200-gal) effluent storage tank. The effluent pumping station consisted of a set of pumps and a set of filters to remove particulates before disposal to the return well.

2.2.2 Laboratory-Scale Tests

Laboratory-scale tests were conducted to evaluate the performance of adsorbents that were selected for the B-5 Reverse Well and the BY-Cribs pilot-scale systems. Candidate adsorbents were selected based on experience and a review of the chemistry of groundwater constituents. Data were generated by both batch equilibrium and small-scale flow-through (mini-column) tests. In addition, because the pilot-scale treatment systems would not be run long enough to reach saturation of the media, the mini-column tests would provide supplemental data to satisfactorily determine treatment effectiveness and adsorbent loading data (see Section 2.6.4).

Three different groups of batch equilibrium tests were conducted at the B-5 Reverse Well site, primarily to evaluate adsorbents for strontium removal. The resin selection tests were performed by the IT Corporation on groundwater shipped to the IT Laboratory in Knoxville, Tennessee. These tests are described in Section 2.2.3.1, 2.2.3.2, and the results are presented in Chapter 3.0. The remaining two groups of tests were conducted at the Hanford Site for the B-5 Reverse Well site and were performed according to the Workplan and Procedures for Bench-Scale Testing of Potential Adsorbents for the 200-BP-5 Groundwater Operable Unit (BHI 1995e). The laboratory-scale tests conducted at the Hanford Site are designated as batch equilibrium tests.

Batch equilibrium tests can be of two types; both involve placing a known amount of adsorbent into a known volume of groundwater. One type determines the K_d , or adsorption, of an adsorbent. The K_d is a single-point expression of the equilibrium, or maximum capacity of an adsorbent for a particular contaminant, expressed in grams of contaminant removed or adsorbed/grams of adsorbent. The K_d is represented as follows:

 K_d (mL/g) = Contaminant Mass/g of Adsorbent Contaminant Mass/mL of Solution

The test uses a single dose of adsorbent that is allowed to equilibrate in the solution for 24 h. It may be repeated several times and at various conditions of temperature and/or pH of the solution. Statistical techniques are applied to the data to arrive at a significant K_d value.

An adsorption isotherm test is a version of the batch equilibrium test in which several different amounts of adsorbent are allowed to equilibrate at a constant temperature, hence, the term isotherm. In both cases, the residual solution is sampled (filtered if not clear) for the contaminants of interest, and then the amount of contaminant loaded/adsorbed on the adsorbent is calculated. The K_d yields a single value. The numerous values of an adsorption isotherm test provide several data points that may be plotted, as isotherms (amount of contaminant removed per gram of adsorbent versus the residual concentration from each dose batch). The slope of the isotherm plots and comparison of the adsorbent dose versus amount of contaminant removed provides an indication of the potential capacity and kinetics of the adsorbent and the ability to meet a specific effluent discharge requirement.

Lastly, small-scale flow-through mini-column tests were conducted to evaluate the adsorbents in dynamic flow conditions similar to the pilot-scale systems to obtain loading data similar to those obtained in the pilot-scale tests.

Flow-through mini-column tests provide dynamic testing of adsorbents that have been selected or screened by the batch equilibrium tests previously described. Mini-column tests provide additional data regarding the performance of a particular adsorbent in a flow-through column operation. By adjusting test conditions (flow, pH, column size) the loading data obtained provide a better definition of loading capacity, breakthrough characteristics, and kinetics of an adsorbent than that obtained from batch equilibrium tests. Ideally, the adsorbent in flow-through column tests is run to saturation, and samples of the adsorbent are removed to elute the contaminant and perform a mass balance to obtain the ultimate capacity of the adsorbent for the contaminants. Because of schedule constraints, this was not done in the mini-column tests, but adsorbent samples were analyzed from the pilot-scale system.

These tests collectively provide the following:

- An optimum adsorbent for the site-specific conditions and contaminants of interest at the 200-BP-5 treatability test sites
- The volume of groundwater that may be processed by a given amount of adsorbent
- The loading capacities of the adsorbent(s), in milligrams per gram (picocuries per gram), thus providing adsorbent volume requirements
- The ability of an adsorbent to achieve a certain quality of effluent.

In the course of treatability test at the B-5 Reverse Well site, it became apparent that 90 Sr removal by clinoptilolite was not effective at the residence times being used. The failure to remove 90 Sr was attributed to its trace concentration in the groundwater. The extremely low concentrations were insufficient to establish a kinetic driving force to remove the 90 Sr. The maximum 90 Sr influent concentration reported for the treatability tests, 6,410 pCi/L, is equivalent to 4.5 x 10^{-5} ppb. Likewise, the maximum concentrations for $^{239/240}$ Pu (45 pCi/L) and 137 Cs (1,820 pCi/L) are equivalent to 7.3 x 10^{-5} ppb and 2.1 x 10^{-5} ppb, respectively.

A technique known as isotopic dilution or spiking was investigated as a means of improving 90 Sr removal. The addition of nonradioactive strontium, in the form $SrCl_2$, was proposed as a means of increasing the driving force to the clinoptilolite. An addition of a quantity of $SrCl_2$ sufficient to increase total strontium to 2 ppm was tested in field batch-equilibrium and mini-column tests. Results are presented in Section 3.2.2.1.

2.2.2.1 Resin Selection Tests. Samples of groundwater from the prospective extraction wells in the BY-Cribs and B-5 Reverse Well plumes were tested. The test program consisted of the following:

- Homogeneity/characterization analyses
- Batch equilibrium test for screening of ion-exchange media and adsorbents
- Column tests for process confirmation.

The homogeneity analyses were performed to ensure that the groundwater in each sample container was similar in contaminant concentration.

The groundwater from the B-5 Reverse Well plume (well 299-E28-33) contained representative concentrations of the radionuclides ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr. Batch equilibrium and column studies were performed on this sample. IRC-718 (a chelating ion-exchange resin) (registered trademark of Rohm and Haas, Philadelphia, Pennsylvania), clinoptilolite (a natural zeolite mineral), and bone char were tested.

The groundwater from the BY-Cribs plume (well 699-50-53A) contained representative concentrations of the radionuclides ⁶⁰Co and ⁹⁹Tc as well as cyanide and nitrates. Batch equilibrium studies were performed on the groundwater sample to evaluate the removal of ⁹⁹Tc and to determine whether the cobalt was in a cyanide complex. Two anion-exchange resins, Dowex 21 and Reillex HPQ (registered trademark of Reilly Industries, Indianapolis, Indiana), were tested. Column tests were not run.

2.2.2.2 Batch Equilibrium Tests. Because of previous work at the 200-UP-1 Operable Unit using Dowex 21K for technetium removal, no batch equilibrium tests were performed for the BY-Cribs site. Two groups of batch equilibrium tests were conducted for the B-5 Reverse Well system. The purpose of the first group of B-5 batch tests was to reevaluate different adsorbents for their ability to reduce the levels of cesium, plutonium, and strontium. In the second group of tests, the purpose was to further evaluate clinoptilolite for enhanced strontium removal. Other resins were considered for the second batch equilibrium tests, but the generally good removal efficiency of clinoptilolite coupled with its great economy favored focusing attention on improving its performance.

Batch equilibrium tests were performed by weighing gram amounts of adsorbent that were added to known volumes of groundwater to give varying gram/liter doses. The different amounts of adsorbent were added to filtered groundwater samples taken from the pilot-scale system and stirred for 24 h to equilibrate. The solution was allowed to settle and then sampled and analyzed for ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr in the first group of batch tests. Strontium-90 and total strontium were the only analytes in the second group of tests. pH measurements were taken before the adsorbent was added to the groundwater and again at the conclusion of the test. The first group of Hanford Site batch equilibrium tests used 0.4-, 4.0-, and 20.0-g/L doses of different adsorbents and varying doses of nonradioactive strontium chloride (SrCl₂) (0.2, 2, and 5 ppm, as strontium) added to increase the total strontium concentrations. The second group of batch equilibrium tests used 0.2-, 2.0-, and 20.0-g/L adsorbent doses.

Adsorbents evaluated in the first group of onsite batch equilibrium tests are as follows. Data sheets for the adsorbents are given in Appendix B.

• Bone Char: An absorbent made from cattle bones that have been activated as is activated carbon (Tate & Lyle, Greenock, Scotland: Tigg Corp., Pittsburgh, Pennsylvania)

- Clinoptilolite: A natural zeolite (American Resource Corp., Greenbrae, California)
- Durasil 230: An inorganic oxide-based cation ion-exchange media (registered trademark of GTS Duratek, Inc., Columbia, Maryland)
- IRC-718: A chelating resin (Rohm & Haas, Philadelphia, Pennsylvania)
- UOP A-51: A synthetic zeolite/molecular sieve (registered trademark of UOP Molecular Sieve Adsorbents, Mt. Laurel, New Jersey)
- UOP E-95: A synthetic zeolite/molecular sieve (registered trademark of UOP Molecular Sieve Adsorbents, Mt. Laurel, New Jersey).

Clinoptilolite was the only adsorbent evaluated in the second group of batch equilibrium tests. In addition, the Phipps & Bird Jar Test (registered trademark of Phipps & Bird, Richmond, Virginia) apparatus was the only device used for stirring the solution.

The second group of batch equilibrium tests consisted of the following:

- Test 1 evaluated the effect of pH on adsorption. Two batch tests (in duplicate) were run, one at the normal pH of the groundwater (Test 1a) and one at a lower pH (Test 1b). The pH was lowered by two points (from 8.3 to 6.3) with hydrochloric acid to evaluate whether pH had an effect on adsorption. Clinoptilolite doses of 0.2, 2.0, and 20.0 g/L were added to 1,000 mL of groundwater; no SrCl₂ was added.
- Test 2 evaluated longer contact times and the addition of SrCl₂ after the initial contact. In Test 2a, four batch tests were run at clinoptilolite doses of 2.0 and 20.0 g/L. Duplicates of the 2.0-g/L (at normal pH) and 20.0-g/L (at lower pH) dose from the first test were also run in this test. In Test 2b, after the initial 24-h contact of Test 2a and after sampling, the residual solutions were combined, and fresh clinoptilolite doses of 2.0 and 20.0 g/L were added together with 165 ppb of SrCl₂ (165 ppb as strontium).
- 2.2.2.3 Flow-Through Column Tests (Mini-Columns). Flow-through column tests were conducted at different contact times, nominally 5 and 12 minutes, to evaluate the kinetics (rate of uptake), the effect of various flow rates (synonymous with contact times), and the shape of the breakthrough curve. Five mini-column tests in two groups were performed at the B-5 Reverse Well site, and two mini-column tests were performed at the BY-Cribs site to obtain the loading data described above. Dowex 21K was the adsorbent tested in the mini-columns at the BY-Cribs. The data sheet for Dowex 21K is presented in Appendix B. The adsorbents tested at the B-5 Reverse Well included bone char, clinoptilolite, and UOP A-51.

At the B-5 Reverse Well, two groups of mini-column tests were conducted, coincident with the two runs of the B-5 pilot-scale test. The first group consisted of two sets of columns using a bone char column followed by two clinoptilolite columns in a series flow arrangement (i.e., bone char-clinoptilolite-clinoptilolite) that simulated the pilot-scale system. Both sets of columns were run at the same contact time as the pilot-scale treatment system (5 to 6 minutes). The first set of columns (MCT-1) was run to duplicate Run 1 of the pilot-scale system. In the second set (MCT-2), SrCl₂ was added to each of the clinoptilolite columns to evaluate a strontium spike addition as a method for enhancing ⁹⁰Sr removal, but all other conditions duplicated the first set (and the pilot-scale system).

The second group of tests was conducted after the pilot-scale system's first run was terminated and the resin replaced for Run 2. Test MCT-1 was continued under the same designation and at the same 5- to 6-minute contact time, as the lead column had not reached saturation for %Sr or breakthrough of 137Cs. The second test, designated "2MCT-2," consisted of one bone char column followed by two clinoptilolite columns, run at a 12-minute contact time. The third test (2MCT-3) consisted of a bone char column followed by a clinoptilolite column followed by a UOP-51 column and was run at a 12-minute contact time. Table 2-7 defines the test parameters/conditions with regard to test number (i.e., 2MCT-3 consisted of bone char, clinoptilolite, and UOP A-51 resins with a 12-minute contact time.

At the BY-Cribs site, two mini-column tests were conducted, each consisting of two mini-columns filled with Dowex 21K and each run at a nominal 5-minute contact time to duplicate the pilot-scale system. The purpose of these tests was to obtain supplemental loading and breakthrough data because it was assumed that the BY-Cribs pilot-scale test would conclude before reaching breakthrough. For these tests, a storage tank was filled with prefiltered groundwater from the pilot-scale system that fed the mini-columns (Figure 2-1). The treated effluent was collected in an effluent storage tank and returned batch-wise to the pilot-scale treatment system.

For both sites, those mini-column tests were run until the effluent concentrations from the lead column at each site were 30% to 50% of the feed concentration (breakthrough). After breakthrough the lead columns were run to saturation monitoring 90Sr at the B-5 Reverse Well site and 90Tc at the BY-Cribs site at increased sampling frequencies to provide data for plotting the breakthrough curve. To determine each treatment system's removal effectiveness for the other contaminants, 60Co, 137Cs, and 239/240Pu concentrations were monitored periodically at the respective sites.

The mini-columns at the B-5 Reverse Well site were operated and sampled in accordance with the Workplan and Procedures for Bench-Scale Testing of Potential Adsorbents for the 200-BP-5 Groundwater Operable Unit (BHI 1995e). The mini-columns were operated and sampled at the BY-Cribs site according to the Workplan and Procedures for Bench-Scale Testing for the 200-BP-5, Unit #2 Groundwater Operable Unit (BHI 1995d).

2.2.3 Groundwater Monitoring/Hydrogeologic Assessment Activities

Groundwater monitoring activities, including water-level measurements and groundwater sampling, were conducted to assess hydrologic impacts and the contaminant response to pumping on the local aquifer system. Sample collection and water-level measurements were performed in accordance with BHI-EE-01, *Environmental Investigation Procedures* (BHI 1994). At the B-5 Reverse Well site, water-level pressure transducers connected to a remote datalogger recorded water-level fluctuations in wells 299-E28-7, 299-E28-23, 299-E28-24, and 299-E28-25. Water levels were typically recorded every 60 minutes during periods of normal operation. Water-level measurements were collected at shorter time intervals during the period of system startup and continuous operation. Water levels were also measured with steel tapes in selected wells in the vicinity of the test to support characterization of groundwater flow.

At the BY-Cribs plume site, water-level pressure transducers were installed in both the extraction well 699-50-53A and the adjacent well 699-50-53B, which monitors the upper confined aquifer. Because of the distance between the injection and the return wells, a separate water-level data logging system was used to monitor the return well (699-49-55A) and the upper confined monitoring well

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699-49-55B. Water levels were also measured with steel tapes in selected wells in the vicinity of the BY-Cribs site to support characterization of groundwater flow.

Groundwater samples were collected before the treatability test to baseline the concentration of contaminant in the groundwater. Baseline samples were then analyzed at an offsite laboratory. Groundwater samples were also collected at regular intervals during the operation to determine the concentrations of the contaminants of concern. Samples were collected at the wellhead to serve as time-specific representations of the groundwater quality. These samples were analyzed with the process samples at the 222-S Laboratory. Samples were collected for Level V confirmatory analyses before the start of the process operations and near the end of the operating cycle.

Additional hydrogeologic assessment activities were conducted at both sites. Details of these activities are presented in Section 4.4.

2.3 EQUIPMENT AND MATERIALS

Major components of testing equipment and materials are described in the following sections.

2.3.1 Pilot-Scale Tests

The major pilot-scale treatment system components, such as pumps/filters and treatment vessels, were mounted on skids for ease of transportation to each test site. Each treatment system consisted of an extraction well, an influent storage tank, an influent pumping station, an ion-exchange treatment system, an effluent storage tank, an effluent pumping station, and a groundwater return well to dispose of treated groundwater (Figures 2-2 and 2-3). High-pressure hoses with quick-disconnect fittings were used to connect the components and to transfer the groundwater through the system. The following sections discuss the pilot-scale system in more detail.

Power for the well pumps, process pumps, and other equipment requirements was furnished by portable diesel-powered generators. A process water tank filled with raw water was used for backwashing requirements, and bottled water was used for sanitary water requirements. Trailers were provided for field test personnel. Tents provided protection for the treatment system from weather. Parts were stored in onsite CONEX boxes.

2.3.1.1 Extraction Wells. The extraction wells were fitted with 480-V, three-phase submersible pumps. At the B-5 Reverse Well site a 3-hp pump was used, and at the BY-Cribs site a 1 1/2-hp pump was used. The pumping rate from the well was monitored by a paddle-type flowmeter with remote indication. The groundwater was monitored for temperature, pH, dissolved oxygen, and turbidity. A carbon steel manifold was mounted to a flange welded to the extraction well casing, and an air release valve was mounted in the manifold to vent air back into the well casing until groundwater flow was established. A low-level interlock was installed in the extraction well to shut off the well pump on low level.

- **2.3.1.2** Storage Tanks. To store the influent and treated effluent, 31,000-L (8,200-gal) storage tanks were used. The tanks were fitted with level indicators and control interlocks to shut off the following pumps:
 - The well pump on high level in the influent tank
 - The feed pump(s) to the treatment system on influent tank low level and effluent tank high level
 - The effluent pumps on effluent tank low level or disposal well high level.

The tanks were also fitted with a clear polyvinyl chloride sight gauges as backup for the level indicators. The sight gauge was also used to perform pump flow calibrations by measuring the rate of rise. In addition to the level interlocks, each tank used secondary overflow control involving the installation of overflow piping back to either the extraction or disposal well, as appropriate.

2.3.1.3 Influent Pumping Station. The skid-mounted influent pumping station was used to transfer groundwater from the influent storage tank through prefilters, through the ion-exchange treatment system, and into the effluent storage tank. The influent pumping station consisted of two centrifugal pumps, a filter system, a pipe manifold connecting the pumps and filter system together, and a locally mounted control panel. One pump was operated at a time; the other was redundant. The pumping station included a flow meter to measure the instantaneous flow rate and provide totalized volume of water treated. The flow rate was manually adjusted. Flowswitch interlocks were used to protect the pumps from loss of flow, and differential pressure instrumentation was used to indicate differential pressure across the filters. Flowswitches were also used to monitor breaching of the hoses and were interlocked to stop the pumps and thus mitigate potential spills. Valving and quick-disconnect fittings for an air hose were available to blow down the system from several locations in the piping.

The filter system was used to remove suspended solids from the groundwater before the adsorbent/ion-exchange treatment system. The filter system used polypropylene bag filters with 2.5-to 5- μ m rating housed in a stainless steel housing with a basket filter support. Four filters were manifolded together with individual valves on the inlets and outlets such that one or all four filters could be online.

2.3.1.4 Process Treatment System. The treatment system consisted of fiberglass adsorbent/ion-exchange columns that were 76 cm (30 in.) diameter by 183 cm (72 in.) high, each having a volume of 0.74 m³ (26 ft³) (738 L or 195 gal). The B-5 Reverse Well system used three ion-exchange columns, and the BY-Cribs system used two ion-exchange columns. The B-5 Reverse Well system consisted of a bone char column followed by a clinoptilolite column followed by a mixed bed column of bone char and clinoptilolite, in series flow. The BY-Cribs system consisted of two columns with Dowex 21K resin, in series flow. A GAC column was added to each treatment system shortly after startup to remove trace organic constituents found in the used 31,000-L (8,200-gal) tanks. The GAC columns were added to the end of each treatment train. The adsorbent/ion-exchange system(s) was plumbed to allow for series or parallel flow but operated only in series flow. Hoses connected the influent pumping station to the process treatment system. Two stainless steel process manifolds were used for mounting instrumentation and were located next to the ion-exchange columns. The inlet manifold contained instruments for monitoring process pH, flow, and pressure before the adsorbent/ion-exchange columns. Differential pressure transmitters were used to monitor the pressure drop across each adsorbent/ion-exchange column. After passing through the adsorbent/ion-exchange

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columns, the groundwater went through the second stainless steel pipe manifold mounted beneath the inlet manifold to monitor the outlet pH. The treated groundwater exited the outlet manifold through a 5-cm (2-in.) high-pressure hose to the effluent tank(s).

- 2.3.1.5 Effluent Pumping Station. The effluent pumping station was used to pump treated water from the effluent storage tank through a set of cartridge filters to the disposal well through a 2-in. hose. The effluent pumping station was fabricated similar to the influent pumping station, except the centrifugal pumps required less head to deliver the flow to the well, and the effluent filters were cartridge type. A flowswitch located in the pump suction piping was interlocked to stop the pump on loss of flow. Differential pressure transmitters monitored the differential pressure across the filters, and a dissolved oxygen analyzer and a thermocouple with a temperature indicator monitored the treated water discharged to the disposal well.
- **2.3.1.6 Disposal Well**. A flange was welded to the protective casing of the disposal well to mount the disposal manifold, like the extraction well. An air release valve was also used to vent air. The turbidity of the treated water was monitored. A level probe was installed in the disposal well and interlocked to shut off the effluent pump on high level.

2.3.2 Laboratory-Scale Tests

Laboratory-scale tests consisted of resin selection tests, batch equilibrium tests, and small-scale flow-through tests (mini-column tests).

2.3.2.1 Resin Selection Tests. Before initiation of laboratory-scale tests, the groundwater samples received by the TDL from the Hanford Site were tested for homogeneity and characterization. Samples were submitted to Quanterra Laboratory in St. Louis, Missouri, for initial characterization analysis. Resin selection tests consisted of batch equilibrium and small-scale flow-through tests.

Batch equilibrium tests were performed by weighing gram (± 0.001 g) quantities of different ion-exchange media/adsorbents into 2,000-mL polyethylene bottles. Resin/adsorbent doses were typically 0.2, 0.5, 2.0, and 20.0 g/L of groundwater. Two thousand grams of groundwater was added to each bottle. The bottles were then sealed and agitated by shaker table or magnetic stirrer for 24 h. When the studies were complete, the processed groundwater was decanted or filtered from the ion-exchange medium/adsorbent. The liquid was aliquoted and sent to the Quanterra Laboratory in Richland, Washington, for analysis. Some of the liquid was also used to perform in-house analyses for preliminary results.

The small-column test used a 1.3-cm- (0.5-in.) diameter glass column fed by a variable-speed tubing pump. The column was packed with 20 mL of clinoptilolite and backwashed with approximately 13 BV of deionized water until the deionized water coming off the top of the column was clear. The column was then allowed to repack before the feed, at 3 mL/min, was introduced downflow to the column. The duration of the column study was 1,000 BV. During the run, grab and composite samples were taken for analysis to determine effluent quality. These samples were analyzed for total strontium.

2.3.2.2 Batch Equilibrium Tests. The Hanford Site batch equilibrium tests were performed by weighing gram (± 0.01) quantities of adsorbent into 1-L glass beakers. Next, 750 mL of sample groundwater was added to the beakers used in the first test group and 1,000 mL was added to the

beakers used in the second test group. The test samples were then stirred in a Phipps & Bird Jar Test device or a toxic characteristic leach procedure (TCLP) tumbler for 24 h. After 24 h, the stirrers were stopped and the contents allowed to settle. The supernatant was filtered via a 1-µm filter, sampled, and analyzed for the specific contaminants of interest. Because of the low concentrations of the radionuclides in the residual liquid, a large volume (750 to 1,000 mL) of sample was required to obtain an adequate analysis. The feed was filtered groundwater from the pilot-scale treatment system just before the first adsorbent/ion-exchange column. The feed was drawn into a 757-L (200-gal) polyethylene storage tank for dissemination to the test containers.

2.3.2.3 Mini-Column Tests. The mini-column test apparatus at each site consisted of a 200-gal influent feed storage tank, a 1- μ m prefilter, two (BY-Cribs site) or three (B-5 Reverse Well site) 2.54-cm- (1-in.) diameter by 1.52-m- (60-in.) long glass columns, and a 757-L (200-gal) effluent collection/storage tank (see Figure 2-1). The filter, columns, pumps, and other components were connected with plastic tubing and tubing fittings. Variable-speed tubing pumps were used to provide flow of groundwater through the columns. Teflon three-way valves were installed in the plastic tubing to provide sampling valves at the inlets and outlets of each column. A pressure gauge was installed in the inlet to the first column of each test setup to monitor backpressure of the system and indicate the potential for plugging. The columns were mounted to a fabricated wood structure with laboratory hardware and clamps.

The columns were constructed of heavy wall glass with ground spherical ball joint ends. Glass spherical ball joint (ground joint) adapters were fitted to both ends to allow installation of stainless steel tubing with which to connect the plastic tubing. High-vacuum silicone grease was used to seal the spherical ball joint adapters to the glass column ends. A support media was installed in the bottom of the columns over which 5 to 8 cm (2 to 3 in.) of 3-mm-diameter glass beads were placed. The adsorbent was added from the top to a depth of approximately 91 cm (36 in.) (bed volume ~460 mL).

2.3.3 Groundwater Monitoring

Continuous water-level measurements were made by installing an in situ water-level transducer in selected monitoring wells. Each transducer was lowered to a predetermined level (top of pump in extraction wells and 0.30 to 0.61 m (1 to 2 ft) below static water level in injection and observation wells) and secured to the top of the wellhead. The transducers were then connected via signal cable to a central datalogger, which recorded and stored water-level readings at programmed levels. Data were downloaded to a lap-top computer every 2 weeks via an RS-232 port and brought into the office for analysis.

Groundwater samples were collected at a sampling valve on the pump column immediately adjacent to the wellhead. The valve and sampling port were purged before collection to ensure that representative sampling was obtained.

2.4 SAMPLING AND ANALYSIS

Sampling and analysis were performed in accordance with the *Pilot-Scale Treatability Test Plan for the 200-BP-5 Operable Unit* (DOE-RL 1995c), the *Workplan and Procedures for Bench-Scale Testing for the 200-BP-5, Unit #2 Groundwater Operable Unit* (BHI 1995d), the *Workplan and Procedures*

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for Bench-Scale Testing of Potential Absorbents for the 200-BP- Groundwater Operable Unit (BHI 1995e), the Environmental Investigation Procedures (BHI 1994) noted below, and the existing IT TDL procedures. Analyses were performed at the 222-S Laboratory in the 200 Areas of the Hanford Site and at the Quanterra Laboratories in Richland, Washington and St. Louis, Missouri, in accordance with laboratory procedures.

2.4.1 Pilot-Scale Treatment System

Pilot-scale treatment system sampling and analysis focused on providing data to demonstrate treatment system effectiveness and to monitor the impacts, if any, of normal groundwater constituents (i.e., anions and cations) on the effectiveness of the treatment system.

Sample valve arrangements had been provided at appropriate locations during construction of the treatment system to allow sampling during system operation (see Figure 2-2 for locations). Before samples were taken, the sample valves and tubing were purged to ensure that a representative sample was obtained. Purgewater was collected and recycled back into the influent storage tank for reprocessing. Treatment system effectiveness was determined by sampling the influent and effluent and calculating the overall removal efficiency as shown in Section 2.2.3.1. In addition, samples were taken of the inlet and outlet of the prefilters and the first ion-exchange column to determine solids removal efficiency and to monitor breakthrough of the lead adsorbent/ion-exchange column.

Samples were generally taken every other day when processing and were analyzed at the 222-S Laboratory for ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr at the B-5 Reverse Well site and ⁶⁰Co and ⁹⁹Tc at the BY-Cribs site. After the adsorbent was changed, more emphasis was placed on monitoring for ⁹⁰Sr because of the interest in getting the effluent to the MCL of 8 pCi/L. Moreover, as a cost reduction measure at the BY-Cribs site, the sampling frequency was altered to a rotating frequency of three samples every other day from five possible locations.

Periodically, duplicate samples were sent to an offsite laboratory for a QC check of the analysis. In addition, samples were taken periodically from the treatment system to monitor typical groundwater parameters through the treatment system to determine whether any particular ion was being preferentially removed, thereby interfering with the removal of the radionuclides of interest.

Spent adsorbents taken from the ion-exchange columns were also sampled and analyzed for waste designation. These data also provided a check on the amount of target contaminants removed by the adsorbents.

Samples were handled in accordance with BHI-EE-01, Volume 1, Environmental Investigations Procedures, EIP 5.1, "Chain of Custody," EIP 5.4, "Field Cleaning and/or Decontamination of Equipment," and EIP 5.11, "Sample Packaging and Shipping" (BHI 1994).

2.4.2 Laboratory-Scale Tests

2.4.2.1 Resin Selection Test. Two groundwater samples were used in the bench-scale study. Approximately 2,000 mL of each sample was required for characterization. Each sample was split into equal portions; one portion was submitted to Quanterra Laboratories for analyses "as received" and the other portion was filtered through a 0.45-µm glass fiber filter before analysis. All samples

were collected and analyzed in accordance with approved Quanterra laboratory procedures. The untreated groundwater from the BY-Cribs and B-5 Reverse Well plumes was analyzed by the Quanterra-St. Louis Laboratory. the samples of the groundwater resulting from the batch equilibrium, or isotherm, tests were sent to the Quanterra-Richland Laboratory for radionuclide analyses (e.g., 60 Co, 137 Cs, $^{238/239/240}$ Pu, $^{89/90}$ Sr, and 99 Tc). These results were used to verify in-house screening data. Samples of effluent from the column study were collected and analyzed in-house at TDL for 137 Cs by gamma spectroscopy and total strontium by GFAA. Selected samples were sent to the Quanterra-Richland Laboratory for isotopic plutonium analysis.

- 2.4.2.2 Batch Equilibrium Tests. As mentioned previously, two groups of batch equilibrium tests were conducted at the B-5 Reverse Well site. All tests were done in duplicate and analyzed at the 222-S Laboratory. During the first group of batch equilibrium tests, duplicate samples of feed water were also placed on the agitating test apparatus as controls and analyzed with the rest of the post-contact solutions. In addition, a field blank was also processed like the other samples. Samples were analyzed for ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr in the first group of batch equilibrium tests. During the second group of batch equilibrium tests, duplicate feed samples were placed on the jar test apparatus for each set as a control. Samples were analyzed for ⁹⁰Sr and total strontium. Duplicate samples were taken for performing total strontium analyses.
- **2.4.2.3** Mini-Column Tests. Mini-column test samples were taken from the feed and after each column after processing approximately 250 BV (which occurred approximately every 24 h) until 1,500 BV had been processed. The sampling frequency was then increased to every 100 to 150 BV processed, or on a 12-h frequency. The samples were sent to the 222-S Laboratory for analyses, and every fifth sample was sampled for duplicate analyses at an offsite laboratory as a QC check. Samples were analyzed at the 222-S Laboratory for ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr from the B-5 Reverse Well site and for ⁶⁰Co and ⁹⁹Tc from the BY-Cribs site.

2.5 DATA MANAGEMENT

Pilot-scale system operational data were recorded on log sheets and in the field log book specifically assigned to this task. The log book(s) are part of the project file and are preserved according to EIP 1.5, "Field Logbooks" (BHI 1994). Field screening data were handled as operational data. Analytical data for both the pilot-scale and laboratory-scale tests from the 222-S Laboratory and the offsite laboratories were managed in accordance with EIP 2.0, "Sample Event Coordination" (BHI 1994). Analytical results were entered into the HEIS database.

2.6 DEVIATIONS FROM THE TEST PLAN

A number of deviations to the treatability test plan (DOE-RL 1995c) occurred following its implementation. The deviations and resulting impact on test objectives are described below. The most notable deviations from the treatability test plan included changes to the treatment system design and approaches to drilling. The treatability test plan (Rev. 0) submitted to meet the milestone represented the expected state of conditions that guided treatment system design. Production tests performed during the spring of 1995 revealed an unexpectedly thin zone of saturated sediments at the BY-Cribs site and diminished projections of extraction rates. A project review by the Environmental Restoration Contractor (ERC) produced several major changes to system design. The project review and important changes are defined in Appendix A of the treatability test plan (DOE-RL 1995c).

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2.6.1 Geohydrological Testing

Lower than expected production from wells in the BY-Cribs plume was demonstrated during production tests in the spring of 1994. Well 699-50-53A, which was most capable of supplying high concentrations of contaminated water, was ultimately selected as the extraction well but did not produce water at rates exceeding 15 L/min (4 gal/min). Continued low flow led to a recommendation for further evaluations of the BY-Cribs aquifer and a cost benefit analysis of moving the treatment system versus piping or trucking water to the existing location. Changes to the original test plan were detailed in a 200 NPL Agreement/Change Control Form (BHI-00203), dated November 9, 1994, that specified hydrogeologic investigations at the BY-Cribs site and also at the B-5 Reverse Well contaminant plume. Specific items included (1) pump test and groundwater analysis at well 699-55-57, (2) groundwater contaminant analysis at well 699-55-60A, (3) in situ groundwater velocity flow measurements, and (4) tracer tests at the B-5 site injection well. Results are detailed in Chapter 4.0 of this test report.

In addition, several activities proposed in the 200-BP-5 treatability test plan were not performed. Specifically not performed/developed were (1) a computer model of the local groundwater flow and primary contaminant distributions, (2) optimized extraction well cycling data, and (3) refined knowledge of the aquifer contaminant distributions. As explained in Section 2.1.1.2, a computer model was to be developed if the test data warranted continued pump-and-treat activities. Extraction well pumping/cycling data were not required due to successful contaminant extraction at both sites. A refined understanding of aquifer contaminant distribution was not acquired due to difficulties with the drilling program explained in Section 2.6.2.

2.6.1.1 Well 699-55-57 Tests. Groundwater monitoring data from the April 1994 200-BP-5 Operable Unit sampling event indicated that well 699-55-57 had relatively high levels of ⁹⁹Tc (2,100 pCi/L) and also penetrated a 1.5 to 2.1-m (5- to 7-ft) thickness of unconfined aquifer. This well was initially regarded as the most suitable alternative if the system were moved, and tests were proposed to ascertain the amount of water it could produce. Initial reviews of information provided in Ledgerwood (1993) indicated that well construction did not favor high pumping rates because the casing had been perforated using saw cuts. A preliminary pump test indicated a 8-L/min (2-gal/min) extraction rate.

It was proposed that aquifer communications could be improved, and a shot perforation technique, available through onsite sources, was procured. The shot perforation system was overcharged and destroyed most of the casing in the aquifer. Following this, running sand conditions were encountered. A 10-cm (4-in.) telescoping screen was installed and the well jetted free of sand several times. An 8-h production test was performed in late February 1995, and five samples were taken to measure the concentration of the primary contaminants of concern. Test results are discussed in Section 4.3.2.

2.6.1.2 Well 699-55-60A Testing. Wells 699-55-60A and 699-55-60B were identified as two of only a few wells lying west of the expected BY-Cribs plume location near the Gable Gap area. Further, no samples had ever been taken from these locations. The wells were both constructed as 36-cm (14-in.) completions for cooling water supply to the 200 North Area in 1944. Both wells were considered for sampling, but a site inspection showed that the amount of debris in well 699-55-60B would not permit reliable sampling. Well 699-55-60A was sampled in early March, in a cooperative

venture with Pacific Northwest National Laboratory (PNNL). Results are presented in Section 4.4.2.2.

2.6.1.3 In Situ Flow Velocity Measurements. In situ flow velocity measurements were proposed to determine the rate and direction of groundwater flow in the BY-Cribs plume region, in and north of the 200 East Area. The technique used the KV Engineering Company's In Situ Flow Meter device owned by Westinghouse Hanford Company's (WHC) Groundwater Data Services organization. The device has been modified by WHC to use stouter rods, which ensure accurate north-south directional alignment. The device emits a heat pulse in the groundwater and measures voltage change at eight thermistors because of temperature changes. The device is accurate down to 0.012-m/day (0.04-ft/day) groundwater flow velocities.

A series of 14 wells, the construction specifications of which met the instrument's requirements, were identified and tested in late May/July timeframe. The wells were tested to determine groundwater directions and velocities and the data used to verify previous plume migration directions. Tests were also considered at the B-5 Reverse Well, but radionuclide contamination raised concerns that the device might become contaminated. Additional test information and results are given in Section 4.3.3.

2.6.1.4 B-5 Reverse Well Injection Well Change. The reinjection well proposed in Section 4.1.2 of the 200-BP-5 treatability test plan (DOE-RL 1995c) was questioned before startup of the system. An ERC regulatory analysis interpretation required that treated water could not be reinjected outside the existing contaminant plume unless the contaminant concentrations were less than the MCLs. For the BY-Cribs plume, the nearby wells were within the plume; therefore, no problems existed with the proposed return well.

At the B-5 Reverse Well site, the plume was much smaller in area and did not include the preselected injection well 299-E28-1. The injection piping was pulled from this well and relocated to well 299-E28-7, 18 m (60 ft) upgradient from the extraction well. Concerns were raised regarding recirculating treated water and treating it again with much lower radionuclide concentrations through the system; tracer tests described in the next section were performed.

2.6.1.5 Tracer Tests at the B-5 Reverse Well. Two tracer tests were conducted around the B-5 Reverse Well during the course of system operations to test for recirculation of treated groundwater. In both cases, a bromide tracer was mixed into treated effluent water and reinjected at the start of 5 days of continuous extraction/treatment operations. Hourly samples were taken from the extraction well and analyzed with a bromide detector. Extraction rates during both tests were 95 L/min (25 gal/min).

The first test was run using well 299-E28-23 as the extraction well and well 299-E28-7 as the injection well. These wells are approximately 18 m (60 ft) apart. This arrangement provided a measure of flow between an injection well upgradient from the extraction well. The second test was run with the same extraction well location but with the injection well moved to well 299-E28-25, located approximately 8 m (25 ft) northwest and downgradient from the injection well. Test results are described in Section 4.3.4.

2.6.1.6 Cyclic Pumping at the B-5 Reverse Well System. Section 4.2.1 of the 200-BP-5 treatability test plan proposed cyclic pumping and testing of the B-5 Reverse Well. It was expected that, per the conceptual model, rapid declines in the extraction concentrations of ¹³⁷Cs, ^{239/240}Pu, and

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⁹⁰Sr would occur shortly after test startup. It was also expected that after 1 to 2 months of operation, the extracted water would be depleted of high concentrations of contaminant, the system would be turned off, and the groundwater would be allowed to reequilibrate to pretest levels. Testing would then resume 3 to 4 months later. However, because of the continuously high extraction concentrations of ¹³⁷Cs and ⁹⁰Sr, this approach to groundwater extraction and operations was not needed.

2.6.2 Drilling Activities

The recognition of a thin aquifer and poor extraction rates at well 699-50-53A and adjacent wells initiated a program to drill several wells between it and the likely injection well at 699-49-55A. The goal was to locate greater thicknesses of groundwater and greater contaminant concentrations. Two wells were to be evenly spaced between the existing wells and would serve as either extraction, injection, or monitoring wells depending on production testing and groundwater analysis results. Additionally, the use of a heavyweight cone penetrometer exploration system was proposed to ascertain depth to, and thickness of, groundwater, and to then guide drilling accordingly.

Following the ERC project review, revision 0 of the 200-BP-5 treatability test plan proposed the use of a sonic drilling technique using heavy cone penetrometer rods to investigate aquifer thickness and contaminant distribution between the BY-Cribs and well 699-50-53A. Results of this activity would then support drilling a network of eight extraction/injection/monitoring wells around the most productive location found. The sonic drilling system was used in August 1994 and successfully provided aquifer information at three of eight locations. At three other locations holes were drilled, but hard drilling conditions were encountered and groundwater was not reached (see Section 4.4.1). The drilling confirmed a limited ⁹⁹Tc plume south of the extraction well and did not support pursuing drilling activities in FY 1994.

A second set of wells was proposed for FY 1995 in the area northwest of well 699-50-53A. In an effort to better direct the drilling program, a proposal was made to use high-resolution seismic reflection techniques to locate suitable drilling locations. The seismic reflection program was to be used to locate erosion channels in the basalt that were regarded as possibly providing both a thicker aquifer with high concentrations of the target contaminants. To be effective, this proposal relied on downhole seismic velocity data not then available. The overall expense of the seismic reflection program would have severely limited the resulting drilling program and was not assured of providing the needed data. Plans for drilling were dropped for FY 1995. A monitoring well was then proposed at the B-5 Reverse Well plume site. This well was intended for characterization but was not regarded as essential to the treatability test.

2.6.3 Changes in Pilot-Scale System Design/Operation

Design changes and several incidents led to either decreased system effectiveness or delays in operations of both systems.

2.6.3.1 Tank Contamination. System operations began on August 29, 1994, with extraction of groundwater, but after repeated low pH readings at the BY-Cribs system, operations were shut down to allow investigation of the problem. The 31,000-L (8,200-gal) tanks obtained from B Plant and that were being used as influent and effluent storage tanks were found to contain residual material that had

not been removed before installation. This was detected during startup and acceptance testing activities when unusually low pH readings were noted during calibration of pH instruments at the BY-Cribs site. A review of B Plant history indicated the tanks had been used for storage of aluminum nitrate nonhydrate, ethyldiaminetetracetic acid, and nitric acid. The tanks had not been properly cleaned before transfer from B Plant. Cleaning the tanks and disposing of the cleaning solutions for both sites caused a 3-month delay.

Initial samples taken from the tanks also indicated organic contamination. Therefore, to mitigate the potential for organics being entrained into the treated water that would be discharged into the disposal wells, GAC systems were designed and installed for removal of organics. In addition, during tank cleaning a small quantity of elemental mercury was discovered in one tank and mercury vapor was detected in another tank. The elemental mercury was cleaned up and properly disposed of. Tank cleanup and verification sampling were completed, and the treatment systems started again on November 28, 1994.

- **2.6.3.2** Winterization. After approximately 1 week of operations in early December 1994, testing was shut down for 6 weeks on December 6, 1994, to complete winterization activities. This resulted from an ERC management order following an incident resulting from frozen piping in the 100 Areas. System operations at the B-5 Reverse Well site resumed on January 17, 1995, and at the BY-Cribs site on January 18, 1995.
- 2.6.3.3 Change in Resin for Strontium-90 Removal. Resin for the B-5 Reverse Well system was changed from Rohm & Haas IRC-718 to clinoptilolite after laboratory test results indicated that the clinoptilolite appeared to have a higher removal efficiency for ⁹⁰Sr than the IRC-718. The resin change caused several operational problems because the treatment systems were not designed for the higher operating pressures and flows that clinoptilolite, by virtue of its higher specific gravity and smaller size, required. In addition, the finer grain size of the material required finer mesh screens in the columns and refabrication of screens at either end of the columns. The higher specific gravity limited sufficient backwashing of the clinoptilolite, since the original system design did not anticipate the 1,300 to 1,500 L/min (350 to 400 gal/min) flow rates and associated pressure required to achieve the manufacturer's recommended 50% bed expansion for the clinoptilolite. The fines also caused a corresponding increase in system pressures. During backwash, the backwash flow rate could achieve only 57 to 76 L/min (15 to 20 gal/min), which was insufficient to remove the fines. The backwash solutions were passed through filters for removal of the fines. This operation took several days to complete. In addition, because of insufficient backwashing, significant quantities of fines tended to clog downstream in-line filters during the first several weeks of processing operation.
- 2.6.3.4 Ion-Exchange Column Changes. The ion-exchange columns originally selected were thought to have a pressure rating that was too low to meet the requirements of the B-5 Reverse Well system. It also appeared that their delivery would not meet scheduled startup; therefore, procurement of other, higher pressure-rated vessels was initiated. Smaller volume fiberglass vessels were located and ordered. Tradeoffs on the column sizes and resins resulted in operating the systems at 95 L/min (25 gal/min) to provide the minimum 5-minute residence time per column for the contaminants of concern. This was one-half the originally estimated process rate of 189 L/min (50 gal/min).

2.6.4 Preliminary Engineering Assessment of Treatment Alternatives

A preliminary engineering assessment of treatment alternatives study was conducted to identify and evaluate alternate technologies that might be more effective or less costly than ion exchange. The document is being released as a Bechtel Hanford, Inc. (BHI) report.

2.6.5 Addition of the 216-A-25 Gable Mountain Pond to the Risk Assessment

As part of the RBDA and per agreement between DOE, EPA, and ERC, the ⁹⁰Sr plume at the 216-A-25 Gable Mountain Pond was examined for hazard to offsite personnel. As explained in more detail in Section 5.1.2.4, Gable Mountain Pond received a large portion of the cooling waters generated at the 200 East facilities, primarily the PUREX Plant, over the years. In the course of operations, a ⁹⁰Sr plume built up under the pond following an accidental release at the PUREX Plant in 1964. Based on the relative risk assessment performed in the 200 East Groundwater Aggregate Area Management Study Report (DOE-RL 1993a), it was considered to be the third most significant plume in the 200-BP-5 Operable Unit for risk to human health, following the B-5 Reverse Well and BY-Cribs plumes. The risk assessment discussion is presented in Section 5.4.

2.6.6 Treatment Systems Shutdown

The proposed 6-month test period specified in the 200-BP-5 treatability test plan (DOE-RL 1995c) was reduced to 4.5 months following delays resulting from tank cleanout and winterization. In anticipation of the reduced testing program, the field-based mini-column tests were proposed as a means of gathering breakthrough data for the initially selected resins. Modifications were later made to the B-5 mini-column tests to examine several options for enhanced °Sr removal. A 200 NPL Agreement/Change Control Form (BHI-00334) was approved by DOE, EPA, and Ecology on May 15, 1995, and implemented a May 30, 1995 date by which to cease all operations. The BY-Cribs pilot system was shut down on May 22, 1995, and the B-5 Reverse Well system was shut down on May 29, 1995.

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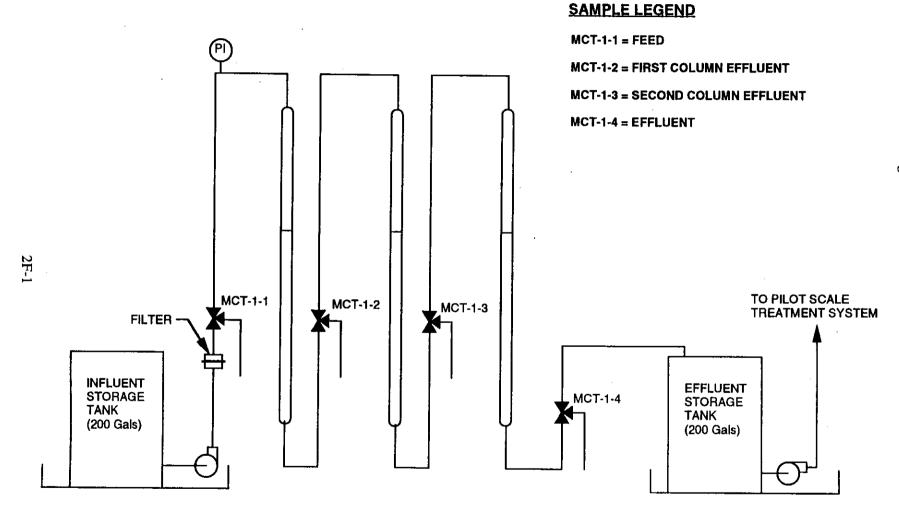


Figure 2-2. 216-B-5 Reverse Well Treatment System Equipment Arrangement.

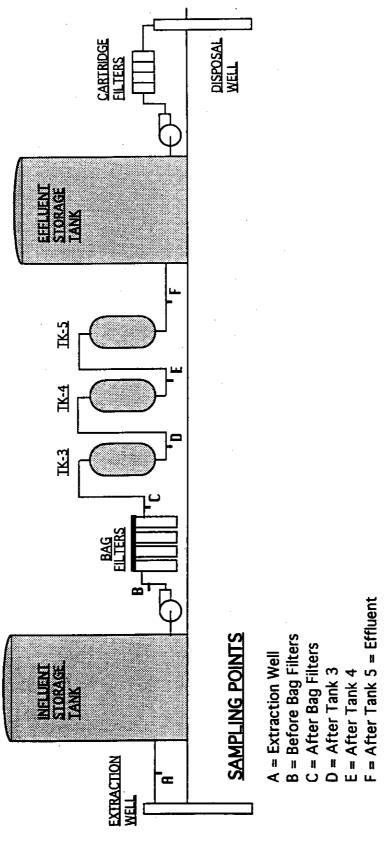
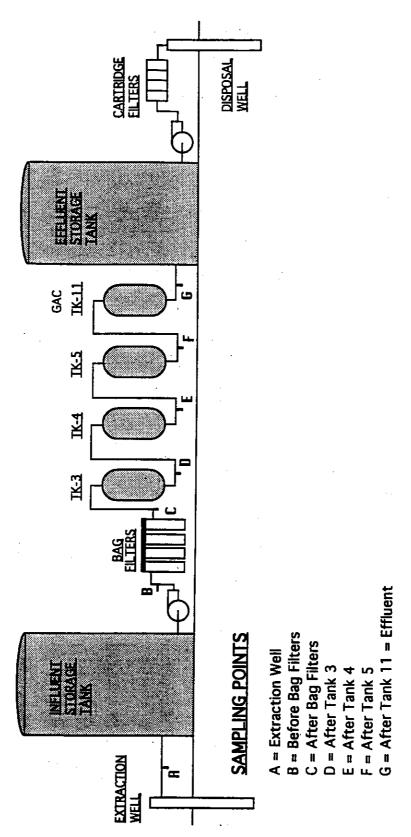


Figure 2-3. 216-BY Cribs Treatment System Equipment Arrangement.



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Table 2-1. Data Quality Objectives for Laboratory-Scale Testing.

Activity	Conduct chemistry tests of ion-exchange resin(s), coupled with pre-treatment system chemistry, if necessary, that is suitable for removing 60Co and 99Tc or 137Cs, 239/240Pu, and 90Sr from groundwater.		
Objectives	Identify the preferred ion-exchange resin(s) effective at capturing ⁶⁰ Co and ⁹⁹ Tc (or ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr) from groundwater samples.		
Prioritized Data Uses	Support pilot-scale treatment system design (i.e., resin selection and pre-treatment requirements).		
Appropriate Analytical Level or Implementation Guidelines	Protocols follow established industrial standards or U.S. Environmental Protection Agency methods. Some analyses may require method modification and/or development (Level V).		
Parameters to be Obtained	Because of the limited scope of the laboratory testing program, the parameters to be obtained focus on the effectiveness of the treatment system as determined by sampling the influent and effluent concentrations of contaminants of concern from the ion-exchange resin columns and potential pre-treatment processes as follows:		
	Resin Selection: Concentrations of primary and secondary contaminants pH, temperature, oxidation/reduction potential Batch equilibria Kinetics, rate of uptake.		
	Pre-treatment: • Effect of pH adjustment on removal efficiency • Resin capacity and effectiveness in removing 60Co.		
Required Detection or Measurement Limits	Analytical detection limits and data quality objective (DQO) requirements (precision, accuracy, representativeness, completeness, comparability parameters) will be identified in the sampling and analysis plan (Appendix C) and the quality assurance project plan (QAPjP) (Appendix B). These requirements will focus on the effectiveness of the processes being tested. The DQOs for Level III analyses are listed in the QAPjP (Appendix B).		
Critical Samples or Values	⁶⁰ Co and ⁹⁹ Tc or ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr concentrations in pre- and post-treatment samples.		
Constraints	Representative samples of groundwater are required from each of the two plumes.		

Table 2-2. Data Quality Objectives for 200-BP-5 Operable Unit Plumes Pilot-Scale Treatment Testing. (2 Sheets)

	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1		
Activity	Pilot-scale treatability test.		
Objectives	Assess effectiveness, operating parameters, and cost of using ion exchange to remove ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr; or ⁶⁰ Co and ⁹⁹ Tc from extracted groundwater.		
Prioritized Data Uses	Support the selection of a preferred interim action alternative for the plume.		
Appropriate Analytical Level or Implementation Guidelines	Level I and II screening analyses will be used for process monitoring. As a minimum, ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr; or ⁶⁰ Co and ⁹⁹ Tc concentrations will be verified by limited Level III or V analyses. No validation (Level IV) data will be required because only interim action decisions are being supported. Requirements will be refined in the process monitoring sampling and analysis plan (Appendix C) and the quality assurance project plan (QAPjP) (Appendix B).		
Parameters to be Obtained	Effectiveness: Influent and effluent concentrations of ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr; or ⁶⁰ Co and ⁹⁹ Tc. Develop breakthrough curves for adsorbents.		
	Operating Parameters: Flow rate pH, temperature, turbidity, dissolved oxygen Operating pressures, both differential and point. Resource Needs: Equipment/materials Personnel and maintenance requirements Secondary waste volumes and characteristics Power and chemical usages Onstream factors Health and safety requirements, including field radiation monitoring		
	 Other cost elements (e.g., mobilization, sample transport, analytical services, decontamination, residuals transport/treatment/disposal). 		
Required Detection or Measurement Limits	Effectiveness: Analytical detection limits for 137 Cs, $^{239/240}$ Pu, 90 Sr, and nitrate; or 60 Co and 99 Tc must be able to detect expected effluent concentrations after 90% removal. Accuracy should be sufficient to support calculation of removal efficiency to $\pm 1\%$. Other supporting documentation such as equipment data sheets may also affect final data quality objectives (DQOs). The DQOs and other precision, accuracy, representativeness, completeness, comparability parameters are specified in the QAPjP (Appendix B).		

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Table 2-2. Data Quality Objectives for 200-BP-5 Operable Unit Plumes Pilot-Scale Treatment Testing. (2 Sheets)

Required Detection or Measurement Limits (Continued)	Operating Parameters: Process chemistry will be measured primarily to detect significant and/or unanticipated secondary impacts on system operation and efficiency. Analytical protocols will follow established vendor standards, industrial standards, or EPA practices. Other operating parameters will be measured as follows:				
	Parameter Instrument Range Accuracy				
	Flow rate	meter	10 to 60 gal/min	±1 gal/min	
	pН	probe	0 to 14	±0.1	
	Temperature	thremocouple or thermometer	0 to 100 °C	±1 °C	
	Turbidity	meter	0 to 100 NTU ^a	±5.0 NTU	
	Differential transmitter or 0 to 30 lb/in² ±0.5 lb/ Pressure gauge				
	Point Pressure transmitter or 0 to 100 lb/in ² ± 1 lb/in ² gauge				
	Resource Needs: Resources will be monitored in accordance with normal recordkeeping practices (e.g., inventory, workhours) specific to each resource type. U.S. Environmental Protection Agency (EPA) guidance (EPA 1992) calls for an accuracy of +50% to -30% in estimating implementation costs.				
Critical Samples or Values	 Cesium-137, ^{239/240}Pu, and ⁹⁰Sr; or ⁶⁰Co and ⁹⁹Tc concentrations in influent and effluent streams at a frequency proportional to the rate of change in the primary contaminants. Chemical and radiological concentrations in spent resins and discarded filter cartridges. Operating costs (e.g., materials, personnel). Secondary waste disposal costs. 				
Constraints	 It is necessary that resin breakthrough be measured in a timely manner for ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr; or ⁶⁰Co and ⁹⁴Tc. Representative samples are required of process water streams, discarded filter cartridges, and spent resin. Groundwater monitoring is required to assess transport of contaminants in response to pumping. Some resin and filter cartridge samples may have high radionuclide content that requires special sampling and handling methods. 				

^aNTU = nephelometric turbidity unit(s).

Table 2-3. 200-BP-5 Treatability Test Objectives and Approach.

Test Objective	Approach		
Effectiveness			
Identify optimum ion- exchange resin	Conduct laboratory-scale batch equilibrium tests of various resins to select the best candidate based on performance and cost.		
Determine ion-exchange system effectiveness for removal of ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr at B-5 Reverse Well and ⁶⁰ Co and ⁹⁹ Tc at BY-Cribs from extracted groundwater	Monitor pilot-scale ion-exchange column influents versus effluents for concentrations of ¹³⁷ Cs, ^{239/240} Pu, and ⁹⁰ Sr at B-5 Reverse Well and ⁶⁰ Co and ⁹⁹ Tc at BY-Cribs using field screening and laboratory analyses. Conduct laboratory-scale mini-column tests to provide additional resin breakthrough data and evaluation of resin kinetics (effectiveness at different flow rates).		
	Operating Parameters		
Refine operational requirements/procedures	Use observational approach during pilot-scale testing to refine operating requirements. Conduct dry-run ion-exchange resin changeout. Revise operating procedures based on observations.		
Assess impacts of "other" groundwater constituents on operational efficiency	Monitor pilot-scale process chemistry including suspended solids, calcium carbonate, chloride, nitrate, phosphate, and sulfate. Use observational approach to detect significant impacts on system operation and efficiency. Revise operations accordingly.		
Assess operating parameters to optimize treatment efficiency	Monitor pilot-scale system flow rate, pH, temperature, turbidity, and pressures. Use observational approach to optimize treatment efficiency. Use laboratory-scale flow-through column tests to perform kinetic studies to evaluate removal efficiencies at various flow rates (residence times), and to provide supplemental resin loading data. Conduct laboratory-scale batch equilibrium tests of resins at various pHs to assess impact of pH on loading effectiveness.		
Demonstrate operational reliability and safety	Operate pilot-scale system for approximately 5 months. Identify and allocate needed resources. Conduct site safety inspections and meetings to ensure health and safety of personnel.		
	Resource Needs		
Develop cost estimates of significant cost components	Track significant expenditures by the following categories: s engineering, construction, and startup; operations; maintenance; analytical; waste management; and project management through the Financial Data System.		
Refine health and safety requirements	Refine Site Safety Plans and Radiation Work Permits based on lessons learned during operations, safety inspections and meetings, and monitoring the work environment as needed. Establish hazardous and radiological control zones as required.		

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Table 2-4. Data Quality Objectives for 200-BP-5 Operable Unit Plumes Contaminant Extraction Testing. (2 Sheets)

	I no		
Activity	Pilot-scale contaminant extraction testing.		
Objectives	Develop a pumping schedule in terms of time pumping is on, time pumping is off for reequilibration or partitioning of primary contaminants, pumping locations, and return locations. Estimate total (dissolved and adsorbed) contaminant mass that can interact with groundwater to reestablish a plume after completion of interim action. Develop estimate of primary contaminants.		
Prioritized Data Uses	Determine contaminant recovery effectiveness and system efficiency of pump-and-treat alternative. Develop numerical model of aquifer and primary contaminant response to pumping.		
Appropriate Analytical Level or Implementation Guidelines	Level I and II screening analyses (gross alpha, beta, or spectral gamma radiation) will be used for preliminary determination of pumping schedule, to be confirmed by limited Level III and V analyses for specific radionuclides.		
Parameters to be	Effectiveness:		
Obtained	 Concentrations of ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr in extracted groundwater Pumping rates and locations, times for pumpage on and off Estimates of dissolved and adsorbed ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr in aquifer and on aquifer materials. 		
	Operating Parameters:		
	 Aquifer hydraulic properties and contaminant distribution properties Estimates of total primary contaminant quantities Optimized pumpage cycling (pumping rates, aquifer reequilibration times, and moving pumping among available wells) to maximize recovery Computer model input parameters for local groundwater flow and contaminant phase transformation and transport. 		
	Resource Requirements:		
	 Electrical costs of pumping Installation of additional wells Technical personnel time to interpret process effectiveness Analytical costs Other significant cost contributors. 		

Table 2-4. Data Quality Objectives for 200-BP-5 Operable Unit Plumes Contaminant Extraction Testing. (2 Sheets)

Required Detection or Measurement Limits	Parameters for field or mobile laboratory screening methods will be developed in the test sampling and analysis plan (Appendix C). Parameters for all analytical measurements will be based on usual limits of normal analytical services as expressed in Table 3-3 and the quality assurance project plan (Appendix B). Screening methods are used because of the need for fast turnaround time to interpret analytical data. Costs will be monitored in accordance with normal recordkeeping practices. Estimates of total (adsorbed and dissolved) primary contaminant mass in aquifer matrix and groundwater will be made through a calibrated computer model and are likely to be accurate to about a half an order of magnitude.	
Critical Samples or Values	Primary contaminant concentrations during pumpage and after reequilibration.	
Constraints	Timing of extraction testing, pumping rates, and pumping procedures cannot be estimated beyond first cycle. Remainder of extraction testing will depend on observed response of groundwater system during initial testing.	

Table 2-5. Data Quality Objectives for Hydrogeologic Assessment.

Activity	Install new wells. Complete aquifer properties testing in the aquifers utilizing pumping and slug tests. Obtain water level measurements from existing and new wells.		
Objectives	New wells will help define aquifer properties, groundwater flow, nature and extent of contamination, and groundwater geochemistry, and will provide additional wells for groundwater extraction and return. Aquifer testing results address general data needs for identifying aquifer properties necessary to evaluate groundwater flow and contaminant mobility. Water level measurements are needed to calculate horizontal and vertical hydraulic gradients required to evaluate groundwater flow.		
Prioritized Data Uses	Support groundwater flow characterization and contaminant transport evaluation needed to refine the operable unit conceptual model and support project treatability test operations.		
Appropriate Analytical Level or Implementation Guidelines	Wells will be installed according to procedures outlined in the Environmental Investigations and Site Characterization Manual, EII 6.8 (WHC 1988) and Washington Administrative Code (WAC) 173-160 and following description of work (DOWs). Aquifer tests will be completed according to procedures outlined in the Environmental Investigation Procedures, EIP 7.1, "Aquifer Testing" (BHI 1994) and follow-on DOWs. Water level measurements and related tape calibration will be completed according to procedures outlined in the Environmental Investigation Procedures, EIP 7.1, "Aquifer Testing" (BHI 1994).		
Parameters to be Obtained	Hydraulic conductivity, transmissivity, and depth-to-water measurements for existing and new wells.		
Required Detection or Measurement Limits	None.		
Critical Samples or Values	 Wells must be completed in the aquifer to allow sample collection for analysis and to allow aquifer testing. Wells need to show maximum contaminant values and the extent of contaminant plumes. Single-well slug tests should be supplemented by limited numbers of slug interference and/or pumping tests with observation wells to characterize test results variability. Periodic water level measurements to assess aquifer gradient and response to pumping and changes in B Pond disposal practices. 		
Constraints	 Well design should consider contingencies for multiple use (i.e., pump-and-treat systems). Emphasize slug tests in contaminated areas. Accurate surveys of measurement points are required. 		

Table 2-6. Data Quality Objectives for Groundwater Monitoring.

Activity	Groundwater sampling and analysis.		
Objectives	 Establish a baseline on major contaminants and co-contaminants before the start of treatability testing. Determine the effect of groundwater extraction and return on concentration of primary contaminants and co-contaminants within the plume and in the vicinity of the return well. Determine hydraulic impacts in the area influenced by the test Provide remedial design input. 		
Prioritized Data Uses	Support treatability test feed water requirements and provide input parameters for numeric model calibration and remedial action design.		
Appropriate Analytical Level or Implementation Guidelines	Field screening (Level II) will be used as a quick indicator of test effectiveness. Data from an offsite laboratory (Level IV) will be used to make final decisions regarding the test. See the quality assurance project plan (Appendix B) for required detection or measurement limits.		
Parameters to be Obtained	 Concentration of primary contaminants (⁶⁰Co and ⁹⁹Tc for the BY-Cribs plume, ¹³⁷Cs, ^{239, 240}Pu, and ⁹⁰Sr for the B-5 Reverse Well plume). Interfering nontarget groundwater constituents (anions) before, during, and/or after test(s). General groundwater quality parameters to include gross alpha, gross beta, pH, temperature, conductivity, and oxidation/reduction potential. Groundwater flow direction before, during, and/or after testing within the network. 		
Required Detection or Measurement Limits	None.		
Critical Samples or Values	Groundwater should be sampled before the test begins, as needed during the test (e.g., pore volume removed, pumping cycle completed, significant contaminant reduction), and at completion of the treatability test to evaluate test effectiveness.		
Constraints	 Representative groundwater samples are required. Accurate water level measurements are required. 		

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Table 2-7. Mini-Column Tests at the 216-B-5 Reverse Well.

Test 1	MCT-1	MCT-2	
Columns	Bone char-clinoptilolite- clinoptilolite, run at 5- to 6-minute contact	Bone char-clinoptilolite- clinoptilolite (with strontium chloride addition), run at 5- to 6-minute contact	
Test 2	2MCT-1	2MCT-2	2MCT-3
Columns	Continuance of Test 1 MCT-1 (Bone char-clinoptilolite-clinoptilo lite)	Bone char-clinoptilolite- clinoptilolite, run at 12-minute contact	Bone char-clinoptilolite- UOP A-51, run at 12-minute contact

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3.0 RESULTS AND DISCUSSION

3.1 PILOT-SCALE TESTS

3.1.1 Pilot-Scale Treatment System Effectiveness

3.1.1.1 216-B-5 Reverse Well. Two tests were conducted with the pilot-scale treatment system at the B-5 Reverse Well site, designated as "Run 1" and "Run 2". Tables C-1, C-2, and C-3 (Appendix C) present the ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr analytical results for the ion-exchange treatment system for Run 1. Figure 3-1 is a plot of the lead column results for ⁹⁰Sr removal, and Figure 3-2 is a plot of the overall system results for ⁹⁰Sr removal. Figure 3-3 is a plot of the lead column results for ¹³⁷Cs removal. Figure 3-4 is a plot of the overall system results for the removal of ^{239/240}Pu. Because the system was not able to achieve the target ⁹⁰Sr level after 2 months of operations, testing was terminated and the clinoptilolite and mixed-bed (clinoptilolite and bone char) media removed.

Run 1 average inlet concentrations (see Tables C-1, C-2, and C-3, Appendix C) were 1,354 pCi/L for ¹³⁷Cs, 19.4 pCi/L for ^{239/240}Pu, and 3,937 pCi/L for ⁹⁰Sr. The average effluent concentrations from the lead column(s) were less than 59.2 pCi/L ¹³⁷Cs, less than 5.6 pCi/L ^{239/240}Pu, and 1,785 pCi/L ⁹⁰Sr. This results in average removal efficiencies of greater than 95.6% for ¹³⁷Cs, greater than 77% for ^{239/240}Pu, and 54.7% for ⁹⁰Sr.

The ¹³⁷Cs and ^{239/240}Pu effluent results reported are actually an average of the detection limits together with actual values from the 222-S Laboratory. Most of the reported effluent cesium and plutonium values are the detection limits for the 222-S Laboratory, reported as "less than" values. The 222-S Laboratory has high background levels that affect analytical results for some radionuclides in that the detection limits vary and may be quite high. For example, in the tables in Appendix C if a value is an actual value, there is either no "less than" (<) symbol or there is a "Y" in the column for detectable. If the value is at or below the detection limit, there is a < or N in the detectable column. As can be seen, there are times that effluent values are higher than influent values because of higher background yielding a high detection limit at the time the analysis was run, whereas the detection limit may have been lower (lower background) when the influent sample was analyzed and reported as an actual value. Because of this, the data and resulting curve plots appear sporadic. For this reason, it is more important to look at trends rather than the actual values and to interpret results accordingly. If the treatment system can achieve an effluent lower than detection limits of the laboratory, the removal efficiencies reported herein underestimate actual removal efficiencies.

Offsite laboratory detection limits are somewhat lower, indicating that actual effluent values for ¹³⁷Cs may be approximately 8 pCi/L and approximately 1.6 pCi/L for ^{239/240}Pu (the detection limit for two samples was 11 and 4.8 pCi/L for ¹³⁷Cs and 1.5 and 1.6 pCi/L for ^{239/240}Pu). Based on these values, the ¹³⁷Cs and ^{239/240}Pu removal efficiencies would be 99.5% and 92%, respectively, and would indicate that the treatment system may be capable of meeting the 1.2-pCi/L MCL for drinking water for ^{239/240}Pu. In any case, the treatment system appears capable of easily meeting the 120-pCi/L MCL for ¹³⁷Cs.

It should be noted that the lead column for plutonium removal is the first column in line (the bone char column), and the lead column for strontium and cesium removal is the second column in line (the first clinoptilolite column). Thus, referring to the "lead" column for strontium and cesium

removal actually includes the bone char and the first clinoptilolite column. It can be seen (see Table C-1, Appendix C) that for the first 500 BV, bone char removed significant amounts of 90 Sr ($\sim 20\%$). Figure 3-1 is a plot of 90 Sr removals versus bed volumes treated for the "lead column" and shows that for the first 500 BV, 90 Sr removal through the bone char and clinoptilolite columns averaged 47 pCi/L. However, after 500 BV, a sharp increase in effluent concentration is seen, which over the course of Run 1 yielded an average effluent of 1,785 pCi/L. This resulted in an average lead column removal efficiency for 90 Sr removal of 54.7%.

System removal efficiency for ⁹⁰Sr is shown in Figure 3-2 where the average effluent concentration averaged 936 pCi/L. Table C-1 (Appendix C) shows that the mixed-bone char-clinoptilolite column reduced the average ⁹⁰Sr concentration from 1,785 pCi/L to 936 pCi/L, resulting in an overall removal efficiency of 76.2%. Figure 3-3 presents removal results for the lead column versus bed volumes treated for ¹³⁷Cs, and Figure 3-4 presents the ^{239/240}Pu removal results for the treatment system versus bed volumes treated.

A total volume of 2,284,000 L (603,362 gal), or 4,033 BV, of groundwater was processed in Run 1. The mass amount removed by the <u>lead</u> columns is determined by the definition of Specific Activity (SpA) of a radionuclide, in the following relationship:

$$SpA = \frac{(\ln 2) \times N = Ci/g}{T_{1/2}}$$

where N is the number of radioactive atoms per unit mass (6.0225 x 10^{23} /atomic mass) and T_{1/2} is half-life of the radionuclide.

For 90 Sr, the specific gravity is 141 Ci/g or 141 x 10^{14} pCi/g. For 137 Cs, the specific activity is 87 Ci/g and for $^{239/240}$ Pu the specific activity is taken as 0.0886 Ci/g.

For a radionuclide, calculating the grams removed by the adsorbent is determined by the formula:

(Influent Concentration - Effluent Concentration) x (Volume of Water)/(SpA)

The results for lead column removal are presented in Table 3-1.

At the conclusion of Run 1, the clinoptilolite and clinoptilolite-bone char were removed from tanks 4 and 5, and both were recharged with clinoptilolite. The bone char was not changed because it was still demonstrating good ^{239/240}Pu removal. Thus, Run 2 commenced with the original bone char column followed by two columns with new clinoptilolite. The GAC column remained online at the end of the treatment train, but was not regularly monitored. The process flow rate was reduced from approximately 95 L/min (25 gal/min) in Run 1 to 57 L/min (15 gal/min) to increase the contact time to 8 minutes per column (versus the 5 to 6 min of Run 1).

Run 2 analytical results are presented in Tables C-4 and C-5 (Appendix C) for ¹³⁷Cs and ⁹⁰Sr. Because the emphasis of Run 2 was ⁹⁰Sr removal, no plutonium and only limited ¹³⁷Cs data were obtained. The ¹³⁷Cs data were not plotted. Figures 3-5 and 3-6 present the ⁹⁰Sr results versus bed volumes treated for the lead column and the overall system, respectively. Near the end of Run 2, a tracer test was performed, and the flow was increased to 106 to 114 L/min (28 to 30 gal/min), which reduced contact time to 5 minutes.

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Run 2 concluded after processing approximately 1,448, 559 L/min (382,670 gal), or 3,198 BV. The system provided an average effluent concentration of 267 pCi/L ⁹⁰Sr and 58.5 pCi/L ¹³⁷Cs. The corresponding removal efficiencies for ¹³⁷Cs and ⁹⁰Sr were 96% and 94.5%, respectively. As previously mentioned, no plutonium data were obtained for Run 2. It should be noted that there was a significant increase in the influent ⁹⁰Sr concentration from an average of 3,695 pCi/L for Run 1 to 4,896 pCi/L for Run 2. The respective influent ¹³⁷Cs values were 1,354 pCi/L for Run 1 and 1,457 pCi/L for Run 2.

As shown in Figure 3-5, the longer contact time did improve ⁹⁰Sr removal. The quality of the effluent from the lead clinoptilolite column decreased over time, with concentrations ranging from a low of 56 pCi/L at approximately 100 BV to approximately 1,000 pCi/L ⁹⁰Sr at 1,850 BV. At 2,150 BV, coincident with the onset of the tracer test, the effluent concentration increased to 2,200 pCi/L ⁹⁰Sr. The lead column removed an average of 980 pCi/L over the course of Run 2 for a removal efficiency of 80%. For ¹³⁷Cs, the removal efficiency was unchanged at 96% from Run 1. The increase in influent concentration and reduction in residence time during the tracer test probably caused some degradation of removal efficiency for the ⁹⁰Sr.

As shown by the removal efficiency curve in Figure 3-5 (dashed line) before the tracer test, removal efficiencies seem to follow a weekly "stair-step" pattern. For example, after 200 BV, the efficiency declined until 570 BV were reached on a Friday. The removal efficiency then appeared to recover on the following Monday. Again, at approximately 970 BV (Friday), the efficiency was 87%, down from 94% at the beginning of the week (Monday). At approximately 1,000 BV (Monday), the efficiency was back up to 90%. This pattern is seen throughout the test from 350 to 1,850 BV and follows an overall downward trend from week-to-week. This seems to suggest that clinoptilolite may be better suited for batch operations. During the tracer test, a kinetic effect was observed, in that shorter contact times and continuous operation (24 h/day) resulted in increased effluent concentrations, increasing to approximately 2,500 pCi/L by the end of the test (3,000 to 3,200 BV). As mentioned above, longer contact times and daily operations seemed to show improved ⁹⁰Sr removals.

Another observation was that the prefilters (bag filters) periodically demonstrated good ⁹⁰Sr removal values. It is thought that fines from backwashing the clinoptilolite are responsible. The backwash solutions were sent back to the influent storage tank and processed through the system for removal by the prefilters. The fines are the small-sized clay fraction of the clinoptilolite. Because some clays are known to provide good cation removal, it is thought that the clinoptilolite fines residing in the influent storage tank are able to adsorb the ⁹⁰Sr and are periodically resuspended, drawn into the system, and removed by the filters. It also seemed that there was a small amount of material periodically released by the filters because the filter effluent ⁹⁰Sr concentrations are up to 13% higher than the feed. This may be a sampling or analysis anomaly, but a 13% difference is significant (see Table C-4, Appendix C, for a comparison of the data).

The removal efficiency of the second clinoptilolite column for 90 Sr was 13% at 94 BV, but then increased to 90% by 425 BV (see Table C-4, Appendix C, and Figure 3-6). The corresponding feed concentrations of 90 Sr (effluent from lead column) were 56 pCi/L and 289 pCi/L 90 Sr, and the respective effluent concentrations were 48.6 pCi/L and 29.3 pCi/L. The effluent from the second clinoptilolite column is the final effluent.

This indicates that the clinoptilolite performs poorly with a dilute feed, and was not capable of meeting the target MCL of 8 pCi/L 90Sr effluent concentration at an 8-minute contact time. After the

tracer test began, the removal efficiency decreased to approximately 56% at 2,700 BV and was 55% at the end of the test (see Table C-4, Appendix C).

Considering the overall treatment system effectiveness, the average removal efficiency for ⁹⁰Sr was approximately 97%, achieving an average effluent ⁹⁰Sr concentration of 77.8 pCi/L until 1,850 BV; it then decreased to 68% by the end of the test with an effluent concentration of 1,160 pCi/L. While operating with an 8-minute contact time per column, the effluent concentration ranged from 28 pCi/L to 134 pCi/L ⁹⁰Sr. At the end of Run 2 of the pilot-scale test, the effluent had increased to 1,160 pCi/L ⁹⁰Sr.

First-order rate constants (K values) were calculated from the pilot-scale and mini-column test data for 90 Sr to separate the effect of residence time from bed performance. The rate constants were calculated from the following equation.

 $Kt = -ln(Effluent {}^{90}Sr conc./Influent {}^{90}Sr conc.)$

where K is the kinetic constant and t is the empty bed contact time. The resulting rate constants are tabulated in Table 3-2 for each run.

Inspection of the rate constants shows generally good agreement from the first to second bed. The K values are generally higher in the mini-columns relative to the pilot plant. This is probably due to the more efficient contacting in small columns produced by the higher length/diameter ratios and the lower tendencies for small-diameter columns to form short-circuiting channels in the resin.

Based on the K values calculated above, a conservative extrapolation of the data suggests that a three-bed system with approximately 18 minutes of contact time should allow achievement of 8 pCi/L ⁹⁰Sr in the effluent.

Run 2 processed a total volume of 1,448,559 L (382,670 gal), or 3,198 BV, of groundwater. The mass amount removed by the <u>lead</u> columns is calculated as described above for Run 1 and is presented in Table 3-1. In addition, the mass removed by the system is presented in Table 3-3.

3.1.1.2 216-BY Cribs. Based on Beck's work (WHC 1994) with Dowex 21K resin on groundwater containing chromate, nitrate, and uranium, Dowex 21K was selected for use for the 200-UP-1 pilot-scale treatability test. From the results of the *Treatability Test Report for the 200-UP-1 Operable Unit - Hanford Site* (DOE-RL 1995a), Dowex 21K demonstrated good removal of ⁹⁹Tc, to saturation of the lead ion-exchange column at approximately 2,600 BV. The loading curve demonstrated sharp breakthrough beginning at between 1,800 to 2,000 BV. It appeared that the Dowex 21K favored uranium over technetium, but there is no uranium at the BY-Cribs. Therefore, it was expected that Dowex 21K would process approximately 5,000 BV before saturation/breakthrough.

Table C-6 (Appendix C) presents ⁶⁰Co and ⁹⁹Tc data from the first, or lead, ion-exchange column of the BY-Cribs pilot-scale treatment system. The average inlet ⁶⁰Co and ⁹⁹Tc concentrations were 154 pCi/L ⁶⁰Co and 9,008 pCi/L ⁹⁹Tc, respectively. The average treated effluent concentrations from the lead column were less than 38.1 pCi/L ⁶⁰Co and 164 pCi/L ⁹⁹Tc, respectively. This results in an average removal efficiency of greater than 75% for ⁶⁰Co and 98% for ⁹⁹Tc for the lead ion-exchange column. The offsite ⁶⁰Co detection limit was 10.5 pCi/L; therefore, ⁶⁰Co removal efficiency may be on the order of 93.2%. A total volume of 1,426,200 L (376,760 gal), or 3,014 BV, of groundwater

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was processed without reaching saturation or breakthrough, although the data plot (Figure 3-7) indicates some amount of breakthrough. The mass of ⁶⁰Co and ⁹⁹Tc removed during the treatability test by the lead column and the whole treatment system is presented in Tables 3-1 and 3-3, respectively. It can be seen that Dowex 21K is capable of providing a discharge that meets or is lower than the MCL targets of 200 pCi/L ⁶⁰Co and 4,000 pCi/L ⁹⁹Tc.

As noted, the ⁶⁰Co results from the lead column effluent (Table C-7, Appendix C and Figure 3-8) were reported as less-than values, below the detection limit of the 222-S Laboratory. Offsite detection limits are between 10 and 20 pCi/L. Using the data in Table C-7 (Appendix C), the ⁶⁰Co removal effectiveness would be 75.3%; but using the offsite data point of 10.5 pCi/L, ⁶⁰Co would result in a 93% removal.

In the treatability test plan (DOE-RL 1995c), it was hypothesized that the cobalt in the groundwater might be complexed with cyanide in a metal-cyanide complex to form an anion rather than the cobalt cation. Because the cobalt was readily adsorbed on the anion resin, Dowex 21K, it is certain that the cobalt was in anionic form, and most probably in the cobalt-cyanide complex, as was previously hypothesized. However, because no analyses for cyanide were performed as it moved through the treatment system, there is no indication of the complex. Analysis of the resin from the lead ion-exchange column at the end of the campaign indicates a cyanide loading of 9.8 mg cyanide per gram of resin. Because this loading is not proportional to the ⁶⁰Co that was loaded, this indicates other anions were complexed with the cyanide, probably iron, aluminum, and zinc as indicated by somewhat high loadings of these metals on the resin.

3.1.2 Operating Parameters

Operating parameters were monitored to refine operational requirements/procedures, assess the impact of "other" (secondary) groundwater constituents on operational efficiency, and assess operating parameters to optimize treatment efficiency. Because the B-5 Reverse Well system and the BY-Cribs system were similar in design, the operating parameters were also comparable, as noted in the following sections.

3.1.2.1 216-B-5 Reverse Well. The results of the secondary contaminants analyses (Appendix C) show that the clinoptilolite adsorbed calcium, magnesium, and potassium, and released sodium and iron until an equilibrium was reached. After this equilibrium was reached, the concentrations remained fairly constant throughout the system. Samples collected around the bag filters indicate the filters removed iron, zinc, and chloride, apparently originating from the feed tank, but released sulfate and nitrate. The sulfate and nitrate were then absorbed by either the bone char or clinoptilolite. Data collected immediately before the tracer test (1,850 BV) showed the bone char was releasing iron and zinc, but the clinoptilolite was readsorbing these concentrations. Alkalinity, fluoride, chloride, and sulfate concentrations (except data around the bag filters) remained fairly constant throughout the test.

With the available data, it is difficult to determine whether the adsorption and/or release of the contaminants discussed could have affected the treatment efficiency. It is possible that the calcium, magnesium, and potassium occupied potential ⁹⁰Sr sorption sites, thus decreasing the treatment efficiency. However, closer monitoring of the secondary contaminant concentrations is needed for a more definitive conclusion.

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The flow rate, tank levels, differential and point pressures, and pH were monitored hourly to ensure that the treatment system was operated within its design parameters. The flow rate was monitored from the extraction well, the process treatment system, and the groundwater return (injection) system. Point pressures were measured upstream and downstream of each ion-exchange column, GAC column, and influent (bag) and effluent (cartridge) filter sets. The differential pressure was measured across each of these point pressures. The differential pressure across the post-column filters (in-line between the last ion-exchange column and the GAC column) was estimated by measuring the point pressure downstream of the last ion-exchange column and upstream of the GAC column.

The differential pressure across the filters increased as the filters became loaded with solids. The filters were replaced when the differential pressure reached a specified reading, as follows:

Bag filters: 207 kPa (30 lb/in²)
 Post-column filters: 138 kPa (20 lb/in²)

Cartridge filters: see below.

The treated groundwater was gravity-fed from the effluent tank back into the aquifer. With clean cartridge filters, an injection flow rate of approximately 170 L/min (45 gal/min) could be maintained. A 57- to 76-L/min (15- to 20-gal/min) decrease in flow rate was an indication that the filters were loaded, and was, therefore, used as the criterion for filter replacement.

Extraction rates averaged approximately 106 L/min (28 gal/min); therefore, when operating at either 95 L/min (25 gal/min) (Run 1) or 57 L/min (15 gal/min) (Run 2), it was necessary to extract groundwater only during operating hours. If circumstances arose that caused the influent tank to be at a low level at the end of an operating day, extraction occurred overnight.

The pH was noted to increase between the extraction well and the ion-exchange columns, and then to slightly decrease after passing through the ion-exchange columns. The average pH measured at the extraction wellhead was approximately 7.69, increasing to approximately 8.13 before the ion-exchange columns, and decreasing to approximately 8.04 at the ion-exchange columns effluent, before the GAC column. Sufficient sampling of secondary contaminants was not performed with which to determine the cause of the pH variations, which could be due to a variety of factors, including the pH probes. For example, removal of some cations or anions (and even alkalinity) from solution is known to affect the effluent from an ion-exchange column. Additionally, removal of the cations or anions from the resin in the ion-exchange process itself is known to affect the pH of the discharge.

Various problems were encountered while backwashing the clinoptilolite columns. The problems are detailed in the site logbook, kept while backwashing the columns before starting Run 2. This log provides the record for this discussion. Similar problems were experienced when backwashing before Run 1, but a detailed log was not available to support a separate discussion. Backwashing is designed to remove the fines contained within the media and to grade the media with respect to density and/or size. The system was configured so that the backwash water, pumped from the effluent tank, ran through the column (bottom to top), through a portable rotameter, through the influent bag filters, to the feed tank. The bag filters were used to remove the clinoptilolite fines that had been washed from the column so that the feed tank would not be overburdened with solids. A flow rate of approximately 1,893 L/min (500 gal/min) was specified by the clinoptilolite manufacturer for backwashing. The pilot-scale system had not been designed for this high flow rate as it was not designed for using clinoptilolite as the ion-exchange media. Because of high differential pressures across the bag filters, the backwash flow rate had to be maintained at a low level, averaging

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approximately 30 L/min (8 gal/min) (measured at the rotameter). The bag filters had to be replaced seven times during backwashing of each of the clinoptilolite column to relieve the high differential pressure still induced by the approximately 30-L/min (8-gal/min) flow rate. Once the water passing through the rotameter appeared to be clear, the backwashing was stopped. Backwashing the two columns took a total of 5 days. Although it was evident that a completely thorough backwash could not be performed given the physical characteristics of clinoptilolite, the job was considered sufficient given the time constraints of Run 2.

The operational problems experienced after the backwashing of the clinoptilolite were further evidence that fines were not removed during backwashing. The post-column filters had to be replaced ten times, and the bag filters were also replaced once within the first four days of operation because of high differential pressures. The frequency of filter changes then decreased for the remainder of the test.

3.1.2.2 216-BY Cribs. The results of the secondary contaminants analyses (Appendix C) show that the Dowex 21K adsorbed iron, zinc, and copper. However, the other contaminant concentrations remained fairly constant throughout the test, indicating that they had little or no effect on treatment efficiency. Although it appears the adsorption of the iron, zinc, and copper did not affect the treatment efficiency, it is difficult to reach a definitive determination with the available data. Closer monitoring of the secondary contaminant concentrations is needed for a definitive conclusion.

The flow rate, tank levels, differential and point pressures, and pH were monitored hourly to ensure that the treatment system was operated within its design parameters. The flow rate was monitored from the extraction well, the process treatment system, and the groundwater return (injection) system. Point pressures were measured upstream and downstream of each ion-exchange column, GAC column, and bag and cartridge filter sets. The differential pressure was measured across each of these point pressures.

The differential pressure across the filters increased as the filters became loaded with solids. The filters were replaced when the differential pressure reached a specified reading, similar to that at the B-5 Reverse Well bag filters [207 kPa (30 lb/in²) for cartridge filters]. A 57- to 76-L/min (15- to 20-gal/min) decrease in flow rate was used as a criterion for cartridge replacement.

Treated groundwater was originally pumped from the effluent tank into a truck, which then delivered the water to the injection well. A hose was later installed that gravity-fed the treated groundwater from the effluent tank to the injection well. An injection flow rate of approximately 95 L/min (25 gal/min) was maintained.

The extraction flow rate was the limiting factor at the BY-Cribs. The flow rate averaged approximately 13.2 L/min (3.5 gal/min), so the system had to be operated on a batch-like processing schedule. Approximately 15,100 L (4,000 gal) was processed each day with the extraction pump running continuously. Well maintenance activities identified minor biofouling on the well pump. However, the minimal extent of the biofouling was not considered to be a contributing factor to the low extraction rate. No significant increase in production rate was noted following well maintenance.

Water at the extraction wellhead, bag filter effluent, and ion-exchange column effluent was monitored for pH. The pH averaged approximately 7.7 at the wellhead, approximately 7.8 after the bag filters, and remained relatively constant after passing through the ion-exchange columns.

No problems were experienced while backwashing the Dowex 21K. The manufacturer specified a backwash flow rate of approximately 3 gal/min/ft², which equated approximately 102 L/min (27 gal/min). This flow rate was maintained until the backwash water was clear, which took approximately 30 minutes.

3.1.2.3 Resin Disposal. During the early years of Hanford operations, process waste containing significant quantities of uranium was stored in the existing 241-Tank Farms. To reduce tank construction requirements and to recover this uranium, the Uranium Recovery Program (URP) was initiated at the 221/244-U Buildings. This recovery process extracted most of the contained uranium but also concentrated significant quantities of fission products in the effluent returned to the tank farms. Evaporators in the Uranium Recovery Process and at the T and B Tank Farms were not able to keep up with the processing rate, and there was a net loss in tank space (Anderson 1990). An in-plant scavenging process that added potassium ferrocyanide to waste stream before going to the BY Tank Farms was included in post-September 1954 waste treatment operations.

Liquid effluents sent to the 241-BY Single-Shell Tank Farms prior to in-plant fission product removal were transferred to the 241-CR Vault for ferrocyanide scavenging. Upon return, the waste was cascaded through a three-tank system with most of the precipitated cesium dropping out in the first tank. Overflow liquids were then discharged to the soil column at the BY-Cribs through the 216-B-43 to 216-B-49 Cribs between November 1954 and December 1957. Subsequent discharges were made at the BC-Cribs

Cobalt-60 was not recognized in the groundwater until January 1956 when it was detected in groundwater samples near the BY-Cribs (Thomas et al. 1956). The presence of ⁶⁰Co in the groundwater was surprising as it was generally known to readily sorb onto soils and to be significantly retarded in moving through the soil column. Cobalt was assumed to have complexed with a large molecule or to exist in a nonionized form. Cyanide is highly mobile in groundwater, and soil and was present in the waste solutions discharged to the BY-Cribs (Thomas et al. 1956). By the time ⁶⁰Co had been detected, the BY-Cribs had been closed to further waste discharges because of high ¹³⁷Cs concentrations reported in the same timeframe. ⁹⁹Tc was not detected in the groundwater until 1985 (Law et al. 1986). Contaminant plume data are presented in Table 1-1.

Spent resins removed after treatability testing were drummed, sampled, and analyzed for target contaminants, metals, anions, volatile and semivolatile organics, mercury, polychlorinated biphenyls, and radionuclides. Based on the presence of radionuclides, all resins were given waste designations of "low-specific activity" and are suitable for burial onsite. Current plans call for disposal at the Environmental Restoration Disposal Facility when that facility opens in 1996. Cyanide was removed by DOWEX 21K and was found to be present at 10 times the concentration expected from the ⁶⁰Co found in the resins. This suggests that other forms of cyanides are present in the groundwater than the ⁶⁰Co ferrocyanide complex alone.

3.1.3 Resource Requirements

Cost and manhours expended are presented in Table 3-4. The following is a summary of these costs. Total operational costs are \$3,571.5K from September 1994 through June 1995. Total project cost from March 1994 to June 1995 is \$5,555,300. Indirect charges amounted to 29.3% of the total budget. Additional expenses were incurred for other 200-BP-5 activities.

- 3.1.3.1 Operations. Total operational costs accounted for approximately 44.4% of the total operational budget expended through June 1995. The major contributors in the operational costs were from exempt employee charges, personnel assigned to operate the system, analytical expenses, and miscellaneous expenses (see "other" in Table 3-4). Actual costs from operations of the systems during the months of mid-January to May 1995 are approximately 30.6%. With mini-column tests included, field activities encompassed 53.6% of the operational budget. The miscellaneous charges include rental charges for the support trailer and generator, fuel charges, and miscellaneous support from WHC Site Services.
- 3.1.3.2 Analytical Costs. Analytical costs total only 6.9% of the total operational budget through June 1995 for pilot system operations. With mini-column testing, analytical costs reached 13.1% of operational budget. This does not include labor and materials for sample acquisition. Analytical costs related to tank cleanout and geosciences accounted for an additional 1.5% of operational budget. This reflects the emphasis placed on process control using onsite (222-S Laboratory) analytical support rather than offsite laboratory analysis. Also, not all analytical cost have been accrued for samples submitted through May. There is a lag time for these charges to be billed by the offsite laboratory and accrued on the cost account.
- 3.1.3.3 Waste Management. Waste management costs tracked through June 1995 are minor but have not completely accrued. The costs for disposal of some, but not all, spent resins are valued at 1% of operational budget. Sample costs for waste designation are contained with each system's operational analytical subheading.
- 3.1.3.4 Engineering, Design, and Construction. The design and construction of the treatment system accounts for 35.7% of the total FY 1994 and FY 1995 budget expended. The majority of the material costs for construction are actually accrued under the miscellaneous category because the items were purchased by a site contractor tasked with this function, but some materials were purchased by the Design Engineering task.
- 3.1.3.5 Maintenance and System Standby. Cost breakout between the two categories is difficult to separate due to the funding of craft support by supplemented Work Order for much of FY 1995. Total craft support comprised 4.2% of the operational budget. This covers the costs of tank cleanout, winterization, and regular maintenance, which is a small portion of this category. Based primarily on labor and indirects, additional costs incurred by delays to the two systems operations amounted to 13.8% of the operational budget.
- **3.1.3.6 Project Management**. Project management expenditures account for 10.2% of the total expended budget. The majority of the costs accrued come from exempt labor charges and miscellaneous support charges (see "other").
- 3.1.3.7 Project Support. Project support covers technical labor support for multiple aspects of the treatment systems' operations in FY 1995. Higher burn rates are noted during months for tank cleanout and winterization. Project support for FY 1994 is contained primarily within the Design Engineering heading. FY 1995 project support accounts for 15.8% of the operational budget.
- **3.1.3.8** Groundwater Characterization. Costs associated with performing preoperational and operational geohydrologic tests amount to 3.9% of the operational budget. This includes FY 1994 fitness-for-use testing, sonic push activities, tracer tests, and additional groundwater sampling/analysis. Labor associated with the tracer tests is included under operations.

3.2 LABORATORY-SCALE TESTS

As discussed in Section 2.2.3, laboratory-scale tests consisted of resin selection tests, batch equilibrium tests, and mini-column tests. The results of these tests are presented in the following sections.

3.2.1 Resin Selection Tests

Samples of groundwater were collected in late June 1994 from wells in the B-5 Reverse Well and BY-Cribs plume and sent to the IT Corporation Laboratory in Knoxville, Tennessee. The samples were characterized with the following results:

• B-5 Reverse Well: ${}^{90}Sr = 11,400 \text{ pCi/L}$ ${}^{137}Cs = 2,550 \text{ pCi/L}$ ${}^{239/240}Pu = 20.1 \text{ pCi/L}$

• BY-Cribs: $^{99}\text{Tc} = 1,480 \text{ pCi/L}$ $^{60}\text{Co} = 28.3 \text{ pCi/L}$ Cyanide = 149 ppb Nitrate = 20.3 ppm.

3.2.1.1 216-B-5 Reverse Well Groundwater. A series of batch equilibrium tests was performed to evaluate Rohm and Haas IRC-718 ion-exchange resin: two sources of bone char (Resintech and a sample procured for use in the B-5 pilot-scale treatment system - TIGG Corporation), and two sources of clinoptilolite (Resintech and American Resources Corporation) by IT Corporation on characterized samples of B-5 Reverse Well groundwater. The results of those tests are presented in Table C-10 (Appendix C). Isotherm plots are shown in Figures 3-9 through 3-17.

IRC-718 is a chelant-based cation-exchange resin that was tested on the B-5 Reverse Well groundwater. This resin has a preference for the heavy metal, multivalent cations. Results for cesium, plutonium, and strontium are shown in Table C-10 (Appendix C), and an isotherm plot for ⁹⁰Sr is shown in Figure 3-9. The isotherm data show that this resin had little affinity for cesium and, at the lower doses, strontium was not removed. Strontium was probably displaced from the resin by other cations (calcium, iron, magnesium, and other metals) that were more strongly adsorbed or present in higher concentrations. Cesium is a monovalent cation that was not expected to adsorb on this resin. Plutonium loading data were not consistent, and the IRC-718 showed no loading capacity at concentrations below 7.7 pCi/L, which indicates that meeting the MCL criteria of 1 to 2 pCi/L would be difficult with this resin. No isotherm plots are presented for ¹³⁷Cs and ^{239/240}Pu loading on IRC-718.

Bone char has been used in the 200 Area processing plants to remove plutonium from wastewater and was expected to have some capacity to remove strontium. As shown on the data summary and isotherms, the Resintech bone char did not work very well on strontium or cesium, although it was fairly effective on plutonium. Isotherm tests were also repeated on a sample of the actual bone char (TIGG/Tate & Lyle) that was to be used in the pilot test. The pilot plant (Hanford Site) bone char was coarser than the Resintech material. The results of these tests are shown in Table C-10 (Appendix C), and isotherm plots for plutonium loading are shown in Figures 3-10 and 3-11.

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Isotherm tests were also conducted on acid-washed bone char to determine if strontium removal could be increased. The purpose of the acid wash was to clean the surface of the bone matrix (calcium hydroxy apatite) and, it was hoped, increase its activity. Results of this testing indicated no improvement in strontium removal.

The plutonium isotherm for the Resintech bone char (see Figure 3-11) showed that the loading at 7 pCi/L was higher than at 16 pCi/L. This was not expected, and the last data point is suspect. Splitting the difference on the last two points and extrapolating to the inlet groundwater concentration of 20.1 pCi/L plutonium would yield a saturation loading of 38 pCi/g of bone char. Based on 75% of saturation as the effective bed loading capacity, this results in a run length of 1,007 BV. A more conservative loading of 26 pCi/g (actual loading equivalent to a feed concentration of 6.88 pCi/L) resulted in an expected run length of 688 BV. Extrapolation of the Hanford Site bone char isotherm gives a predicted saturation loading of 42 pCi/g, which translates to a run length of 1,113 BV.

Clinoptilolite is a naturally occurring zeolite that removes contaminants by both cation exchange and adsorption because of its molecular sieve properties. It was tested on the B-5 Reverse Well groundwater. The first batch of clinoptilolite, from Resintech, was a high-calcium clinoptilolite. It showed good adsorption for cesium, but tailed off at the high doses and had poor capacity for strontium. A second batch, from AmReC, had a much higher capacity for strontium and better cesium removal at the low concentrations. This is not unusual, because clinoptilolite deposits from different mines vary in their composition and adsorptive capacity. These results are shown in the data summary (see Table C-10, Appendix C), and isotherm plots (Figures 3-13 through 3-16). Because the loading capacity of the bone char for plutonium was not as high as expected, a series of batch equilibrium tests, designed to produce a plutonium isotherm on the AmReC clinoptilolite, were added to the test program. This isotherm showed that the AmReC clinoptilolite has poor loading capacity for plutonium.

At the lowest dose used in the AmReC clinoptilolite isotherm tests (0.2 g clinoptilolite/L of reverse well groundwater), cesium loading was 11,195 pCi/g of clinoptilolite. Extrapolation of the cesium/clinoptilolite isotherm to the inlet concentration (Co) of 2,550 pCi/L would result in a very high saturation loading (~48,000 pCi/g). To be conservative, an effective bed loading of 18,750 pCi/g was assumed, which was only 50% higher than test loading at an equilibrium concentration of 232 pCi/L. This resulted in a predicted run length of 6,470 BV for cesium breakthrough. Extrapolating the strontium isotherm gave a bed saturation loading of 47,000 pCi/g of clinoptilolite or an effective bed capacity of 35,250 pCi/g. This resulted in a predicted run length of 2,700 BV for full loading of strontium. Early breakthrough of low levels of strontium are expected because of low loading at concentrations less than 500 pCi/L. Plutonium was not effectively adsorbed on the clinoptilolite.

- 3.2.1.2 216-BY Cribs Groundwater. Batch equilibrium tests were performed on characterized samples of BY-Cribs groundwater to evaluate the ability of Dowex 21K ion-exchange resin to remove ⁹⁹Tc. The results of those tests are presented in Table C-11 (Appendix C). Isotherm plots are shown in Figure 3-17. Technetium-99 was present in the initial BY-Cribs groundwater at 1,480 pCi/L, which is above the IRM limit of 900 pCi/L. If the isotherm is extrapolated to the inlet groundwater concentration of 1,480 pCi/L, a saturation capacity of 13,500 pCi/g resin is predicted. Based on achieving 75% of saturation loading, the expected run length would be 4,400 BV.
- 3.2.1.3 Clinoptilolite Flow-Through Column Tests. In the laboratory-scale flow-through column testing conducted at IT Laboratory in Knoxville, Tennessee, a 1,000 BV column test with a single

column using clinoptilolite was completed. It confirmed that low levels of strontium rapidly bleed through the column. All strontium analyses for this flow-through column study were conducted as total strontium by GFAA. Removal efficiency for total strontium and ⁹⁰Sr should be equivalent. Data from this study are shown in Table C-12 (Appendix C) and Figure 3-18. Initial column effluent contained 1 ppb of strontium, which is 99.4% removal (based on 155 ppb feed). At 500 BV, effluent strontium was 10 ppb (93.5% removal). At 1,000 BV, effluent was 21 ppb, which is 86.5% removal. Average effluent over the 1,000 BV was about 10 ppb (735 pCi/L). Clinoptilolite (in a single-stage column with a 7.33-minute empty-bed contact time) was not able to get strontium down to the 8-pCi/L (0.11-ppb) DWS. A two-stage system or column with higher contact time should be more effective. Isotopic dilution may also improve removal in a second-stage column. A two-stage system with isotopic dilution might give 1,200 to 1,500 BV with 98% removal of strontium (effluent 2.5 ppb or 184 pCi/L).

Cesium in the column effluent was less than 50 pCi/L after 1,000 BV. Samples of the column effluent showed plutonium at 16.8 pCi/L after 500 BV and at 20.8 pCi/L after 1,000 BV. This shows rapid breakthrough of plutonium and clinoptilolite's lack of affinity for plutonium.

3.2.2 Batch Equilibrium Tests

Batch equilibrium tests are performed at varying dose rates of adsorbent (generally two or more) to provide equilibrium loading data with which to predict the adsorption of constituents from solution. The models proposed by Freundlich and Langmuir are commonly used (Freundlich 1926, Langmuir 1918). The most commonly used model is the Freundlich adsorption isotherm:

$$\frac{X}{m} = KC^{1/n}$$

where:

x = mass of constituent adsorbed (adsorbate) on solid phase, pCi

m = mass of solid (adsorbent) being used for adsorption, g

C = equilibrium concentration of residual groundwater sample, pCi/L

1/n = an empirical exponent used to linearize the data (Freundlich 1926)

K = a constant, sometimes called the Freundlich adsorption coefficient

In log form the equation becomes:

$$\log \frac{X}{M} = \frac{1}{n} (\log C) + \log K$$

which is in the form: y = mx + b.

Plotting the log of x/m versus the log of C should provide a straight line with the intercept of log K and 1/n the slope of the straight line isotherm curve.

Freundlich (1926) proposed that the amount of a solid, or a gas, adsorbed per gram of adsorbing material would be proportional to the equilibrium concentration (for a solid), or the equilibrium pressure (for a gas), raised to some fractional exponent. The Freundlich isotherm, albeit empirical, is

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used to linearize batch equilibrium data as straight line curves when plotted on log-log graphs. These curves then can be used to compare the results of different adsorbents for the intended use.

In addition to the information discussed in evaluating the results of batch equilibrium test data, isotherm plots provide the following:

- The slope of the straight line gives a general indication of an adsorbent's ability for use in either a batch or column operation. Steeper slopes indicate good adsorption from solutions at higher concentrations, but not at lower ones; they also indicate higher capacity and better suitability for flow-through column operation. More gradual slopes indicate more even adsorption over the entire range of concentration of the solution, and the ability to achieve a high-quality effluent, or low concentration of ion in the effluent, may be better suited for batch operations depending on the requirements.
- The curve can be projected to intersect a vertical line constructed at the initial concentration
 (C_o) of the groundwater sample to give an approximate loading capacity of the adsorbent for
 the contaminant.

3.2.2.1 Second Group of Batch Equilibrium Tests. The test program for this work is described in Workplan and Procedures for Bench-Scale Testing of Potential Adsorbents for the 200-BP-5 Groundwater Operable Unit (BHI 1995e). Six adsorbents were evaluated in this test. Results of the second group of batch equilibrium tests, performed at the B-5 Reverse Well, are summarized in Table C-13 (Appendix C), and plots of strontium adsorption isotherms are shown in Figures 3-19, 3-20, 3-21, and 3-22. Isotherm plots are presented for only four of the adsorbents showing adequate 90Sr removal (UOP A-51, clinoptilolite, IRC-718, and UOP E-95). In addition, cesium and plutonium data were not plotted because the emphasis of this test was to find a strontium removal adsorbent. The six adsorbents in this test included UOP A-51, UOP E-95, IRC-718, Durasil 230, bone char, and clinoptilolite. The batch equilibrium tests were conducted with the addition of varying amounts (0.2. 2, and 5 ppm) of SrCl₂ as a source of nonradioactive strontium. The addition of strontium chloride is an isotopic dilution technique that increases the total strontium levels to increase the driving force for mass removal on an adsorbent. If there is sufficient elemental strontium present in the groundwater. there may be sufficient driving force for adsorption. However, the target end concentration of 8 pCi/L of 90Sr only represents a mass concentration of 5.67 x 10⁻⁸ ppb of strontium. Addition of nonradioactive strontium greatly increases the mass loading and in theory should promote adsorption of the radioactive as well as the nonradioactive strontium. Clinoptilolite was also evaluated without the addition of SrCl₂, and the results are plotted in Figure 3-20 with the other clinoptilolite data. Bone char was evaluated to determine whether it was a good choice for plutonium removal. The plutonium data were not plotted as an adsorption isotherm because the residual ^{239/240}Pu concentrations in the batch equilibrium tests were below the detection limit for plutonium and would not provide a representative curve.

The MCL, or 4% of the derived concentration guides, for ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr are 120 pCi/L, 1.2 pCi/L, and 8 pCi/L, respectively. The detection limit for plutonium analysis at the 222-S Laboratory is above the MCL, generally ranging around 4.5 to 8 pCi/L, because of the high background in the laboratory. The detection limit at offsite laboratories is much better (to < 1 pCi/L), but the turnaround time (15 to 45 days) did not provide a timely response and costs were prohibitive.

In the absence of adsorption isotherm plots, batch equilibrium data still provide significant information. In evaluating the results of batch equilibrium tests, the lower dose levels (i.e., 0.2 and 0.4 g/L) give a good approximation of the loading capacity of an adsorbent. The higher dose levels give an indication of the ability of the same adsorbent to achieve a high-quality effluent, assuming kinetics are not a factor (assumes that the adsorbent and the liquid are in equilibrium in the batch test). As can be seen from the plutonium data in Table C-13 (Appendix C) for bone char, the lower dose of 0.4 g/L resulted in a range of plutonium loading between 22.2 to 35.7 pCi/g, and the bone char was able to achieve less than 4 to less than 6 pCi/L residual ^{239/240}Pu. Two other adsorbents indicated good plutonium removal values. UOP A-51 resulted in a plutonium loading of 11 to 12 pCi/g and a residual ^{239/240}Pu concentration ranging from less than 4.6 to less than 7.3 pCi/L at the higher doses; IRC-718 resulted in a plutonium loading of 26.9 to 29 pCi/g and a residual concentration ranging from less than 3.8 to less than 7.3 pCi/L.

From the batch equilibrium data and ⁹⁰Sr adsorption isotherm plots for UOP A-51 and IRC-718 (see Figures 3-19 and 3-21), it appears both have the ability to achieve a high-quality effluent, resulting in a low average residual ⁹⁰Sr concentration of 7 and 4.7 pCi/L, respectively, at the higher doses of adsorbent. Interestingly, the IRC-718 achieved an average 4.7 pCi/L concentration at a 4-g/L dose as well. This would be an indication that IRC-718 might be capable of achieving low concentrations over a range of concentrations as described previously. The UOP A-51 isotherm has a steeper slope, indicating that it might be better suited for column operation. Strontium chloride addition appeared to depress the loading capacity of the UOP A-51 (see Figure 3-19), as witnessed by a decrease in the slope of the isotherm plot. However, SrCl₂ appeared to improve the loading capacity of the IRC-718 (see Figure 3-21). IRC-718 isotherm did have a shallower slope than UOP A-51, indicating that it might be better suited for batch operation. From these data, both IRC-718 and UOP A-51 appear to be good candidates for ⁹⁰Sr removal.

The UOP E-95 and clinoptilolite results are similar in that both achieve residual 90 Sr concentrations between 40 to 60 pCi/L at the higher doses of adsorbent. The isotherm slopes are similar, and SrCl₂ appears to interfere with 90 Sr loading. Referring to the isotherms (see Figures 3-20 and 3-22) it can be seen that the lowest strontium chloride dose of 0.2 ppm provided the lowest 90 Sr concentrations. Both adsorbents might be good for column operation, but would require some type of polishing step to achieve a low 90 Sr concentration. Figure 3-20 also shows the isotherm plot for clinoptilolite where no SrCl₂ was added; it falls between the 2- and 5-ppm plot, which seems to indicate that a SrCl₂ addition of 0.2 ppm might improve 90 Sr removal for clinoptilolite.

Durasil 230 was tested to evaluate its potential for cesium removal. Table C-13 (Appendix C) shows that Durasil 230 is comparable to clinoptilolite for 90 Sr and 137 Cs removal. The Durasil resulted in an average residual 90 Sr concentration of 54 pCi/L and an average 90 Sr loading of 3,075 pCi/g. The results for Durasil regarding 137 Cs indicate an average residual 137 Cs concentration of 57 pCi/L and an average 137 Cs loading of 3,155 pCi/g. The addition of strontium chloride had little affect on 137 Cs removal by Durasil 230.

3.2.2.2 Third Group of Batch Equilibrium Tests. This third group of batch equilibrium tests were conducted to determine the following:

• If lowering the pH by two points would improve/affect the removal of %Sr

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• If the addition of SrCl₂ in a test simulating column operation, where the addition of SrCl₂ is added to the second column (or in the case of this test, the second part of batch Test 2b, described below) would improve 90Sr removal.

The results of the third group of batch equilibrium tests are summarized in Table C-14 (Appendix C) and plotted as adsorption isotherms in Figure 3-23.

In Test 1a, the batch equilibrium tests were run at the normal pH of the feed groundwater, approximately 8.3. In Test 1b, the pH was adjusted to 6.3 with 20% hydrochloric acid to determine the effects of lower pH on the adsorption of 90 Sr. The final pH of the solutions was measured before sampling. The pH had increased by approximately 0.5 pH (nominally 8.5 to 9) in Test 1a and 1.5 pH (nominally 7.5 to 8.0) in Test 1b. It was noted that the higher clinoptilolite doses caused a higher terminal pH, and that the higher doses also required more acid to lower the pH. This is probably due to the highly alkaline nature of the clinoptilolite because of relatively high concentrations of calcium, magnesium, sodium, and potassium oxides. In addition, the groundwater has an alkalinity of approximately 100 ppm.

In Test 2a, the batch equilibrium tests were run at two dosages, 2 and 20 g/L, and then sampled. In Test 2b, the residual solutions from Test 2a were combined, and fresh clinoptilolite in doses of 2 and 20 g/L were added to the appropriate previous doses of 2 and 20 g/L. Then, 0.26 ppm (as strontium) SrCl₂ was added to each beaker and placed back on the stirrers for another 24-h contact.

The results for Test 1 indicate that lowering the pH reduced the clinoptilolite's ability to achieve a low effluent concentration. At the 20-g/L dose, the average residual %Sr concentration of the normal pH batch test was 24.4 pCi/L, whereas the average residual concentration at the lower pH was 46.9 pCi/L. The slopes of the adsorption isotherms, as shown in Figure 3-23, are similar, except the curve for the lowered pH batch test is depressed. However, the %Sr loadings at the 0.2-, 2-, and 20-g/L doses are very close, averaging 3,012.5 pCi/g at the 0.2-g/L dose, 2,061.2 pCi/g at the 2-g/L dose, and 237.3 pCi/g at the 20-g/L dose. Projecting the two curves to the initial concentration, Co of 4,780 pCi/L, and reading from the Y axis (%Sr loading) would indicate that the equilibrium capacity of approximately 3,000 pCi/g was reached.

Test 2, at longer contact and with addition of SrCl₂, had curious results. Test 2a was like Test 1a, except only two doses (2 and 20 g/L) were run having similar results (2,514.5 pCi/g and 275 pCi/g at the 2- and 20-g/L doses, respectively). In Test 2a, the feed ⁹⁰Sr increased to 5,395 pCi/L with different results from Test 1a (which was 4,780 pCi/L). The 20-g/L dose was not able to achieve the low residual concentration of Test 1a (62.4 versus 24.4 pCi/L). Furthermore, the slope of the isotherm plot increased significantly. In Test 2b, the higher dose (60 g/L) indicated the ability to achieve a lower effluent concentration, although not as low as the normal pH batch of Test 1a. The curious observation is that the loading for both the 6- and 60-g/L doses is nearly the same at approximately 850 pCi/g, which seems to indicate that a longer contact or the addition of SrCl₂ does not improve the loading; in fact, some metathesis or desorption may have been occurring.

3.2.3 Flow-Through Column Tests (Mini-Columns)

To adequately evaluate the effectiveness of an ion-exchange treatment system, it is desirable to obtain multiple sets of performance data. This can be accomplished in a short period of time by performing small-scale ion-exchange column tests. The objective is to process sufficient volumes of contaminated

groundwater to load the adsorbent bed to saturation (where the column effluent equals the influent) two or three times to obtain sufficient data. These data then can be used to obtain the shape of the breakthrough curve, the amount of contaminant(s) that can be removed, and the volume of groundwater that can be processed. When the volume of groundwater processed in either pilot-scale or mini-column tests is converted to bed volumes and the flow rate to bed volumes per unit of time, the data can be directly related, compared, and used to scale-up to larger sized systems. For this test, mini-column data were also intended to supplement data collected from the pilot-scale system, since it was anticipated that breakthrough in the pilot-scale system beds would not occur within the desired time frame. To achieve breakthrough data more quickly than with the pilot-scale system, mini-columns were run 24 h/day.

Small-scale flow-through ion-exchange column, or mini-column, tests were performed at both the B-5 Reverse Well site and the BY-Cribs site. Two groups of mini-column tests were performed at the B-5 Reverse Well site. The first group of tests was intended to perform the following:

- Test MCT-1 was to provide supplemental loading and breakthrough data from a flowsheet comparable (i.e., same contact time) to the pilot-scale treatment system (a bone char column followed by a clinoptilolite column followed by a clinoptilolite column in a series-flow arrangement) for the removal of ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr
- Test MCT-2 was to evaluate a strontium dilution method for enhancing the removal of ⁹⁰Sr with the addition of SrCl₂.

From the results of the first group of tests, it appeared that the flowsheet was capable of good removals of ¹³⁷Cs and ^{239/240}Pu, but ⁹⁰Sr removals resulted in an effluent that was still higher in concentration than the target MCL of 8 pCi/L. The emphasis of the second group of tests focused primarily on ⁹⁰Sr removal to accomplish the following:

- Evaluate whether longer contact times on clinoptilolite would improve ⁹⁰Sr removals (Test 2MCT-2)
- Evaluate an alternate adsorbent (UOP A-51) that had demonstrated good ⁹⁰Sr removals in the batch equilibrium tests described above (a bone char column followed by a clinoptilolite column followed by a UOP A-51 column) (Test 2MCT-3)
- Continue the non-SrCl₂ addition test from the first group of tests to achieve either saturation with respect to ⁹⁰Sr or breakthrough of ¹³⁷Cs (Test MCT-1 continued).

One group of mini-column tests was performed at the BY-Cribs site using Dowex 21K ion-exchange resin. Because it appeared that the pilot-scale treatment system would not be able to reach saturation for ⁹⁹Tc or ⁶⁰Co, supplemental loading data needed to be obtained by performing two mini-column tests. Both mini-column tests were run at the same contact time of the pilot-scale treatment system (~5 min per column) in a two-column (lead and polish column series-flow arrangement) configuration.

3.2.3.1 216-B-5 Reverse Well Group 1. Because it appeared that ⁹⁰Sr would be the most difficult contaminant to remove, the emphasis of both groups of mini-column tests performed at the B-5 Reverse Well site was to evaluate/enhance the effectiveness of clinoptilolite for the removal of ⁹⁰Sr. However, it was also necessary to obtain data to supplement the pilot-scale treatment system on

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effective ¹³⁷Cs and ^{239/240}Pu removals. These data were obtained during the operation of the first group of mini-column tests. In addition, during the second group of mini-column tests, as described above, ¹³⁷Cs data were collected to look for either breakthrough of ¹³⁷Cs or saturation of ⁹⁰Sr.

The results of the first group of B-5 Reverse Well site mini-column tests (MCT-1, MCT-2) are shown in Tables C-15 and C-16 (Appendix C) and plotted in Figures 3-24 through 3-29. Tables C-17 and C-18 (Appendix C) and Figures 3-30 through 3-33 give the results for the second group of tests (2MCT-2, 2MCT-3). All mini-column sample analysis results are reported in Appendix C.

Data reported in the tables are organized by bed volume and sample port. Sample ports are distinguished by the first and second numbers in the labels (e.g., "MCT-1-1" means mini-column test one, sample port one) (see Figure 2-1 for locations of sample ports).

Blank spaces in the tables and missing points on the curves are a result of missing or lost data. As previously discussed, some samples analyzed by the 222-S Laboratory were below the laboratory detection limits. The detection limits averaged approximately 60 pCi/L for ¹³⁷Cs, approximately 10 pCi/L for ^{239/240}Pu, and approximately 4 pCi/L for ⁹⁰Sr. Data below the detection limit are shown with a " < " (less than) in the column to the left of the data in the tables. These data do not present a precise picture of removal effectiveness in that it cannot be determined whether the target treatment goals were met. However, they can be used to demonstrate the general ability of the treatment system to remove the contaminants. Offsite laboratory results, because of lower background, provide the data necessary to show the ability to meet target treatment goals.

Because the influent concentrations of the contaminants varied, the trends of the effluent concentrations do not always represent how the system is performing. Therefore, the results will be presented in terms of removal efficiencies with mention of influent and effluent concentrations. Removal efficiencies are calculated by the following equation:

(Influent - Effluent) x 100%

After processing 3,500 BV, both Tests MCT-1 and MCT-2 were stopped. Review of the data indicated that the addition of SrCl₂ (in Test MCT-2) appeared to degrade the performance of clinoptilolite for ⁹⁰Sr removal. Further, because of the gradually increasing (not sharp) breakthrough curve for both tests (MCT-1 and MCT-2), it could not be determined when breakthrough of the clinoptilolite would occur (see Figure 3-28).

Two new tests were proposed. First, it was decided to perform a kinetic evaluation of clinoptilolite to see if a longer (12-minute) contact time would improve ⁹⁰Sr removal (Test 2MCT-2). Second, it was decided to evaluate an adsorbent selected from the batch equilibrium tests, UOP A-51, in combination with clinoptilolite in a bone char column, clinoptilolite column, and a UOP A-51 column test (Test 2MCT-3). Third, it was decided to continue the clinoptilolite test (without SrCl₂) at the previous contact time to see if saturation with respect to ⁹⁰Sr or breakthrough of ¹³⁷Cs could be achieved (continued test of MCT-1). These tests are summarized in Table 2-7 and discussed in Sections 2.2.3.4 and 3.2.3, 3.2.4.

Before stopping the tests at 3,500 BV, the columns experienced minor biofouling by algae. While MCT-1 operations were interrupted, algae increased, becoming visible in the standing water above and within the top 10 cm of the clinoptilolite and in sections approximately 30 cm from the top of the

bed in both the lead and second clinoptilolite beds. Once the test was restarted, the algae continued to grow throughout the column, covering the clinoptilolite, column inner walls, and water above the clinoptilolite, with algae in both the lead and second clinoptilolite columns. An increase in growth was noticeable after a weekend, or when the test was stopped for more than a day, and as the weather became warmer. A small (approximately 1 cm in diameter) growth was visible on the top of the bone char bed. The algae were visible throughout the tubing of the system and continually clogged the precolumn filter. Growth was also visible on the bottom of the mini-column influent tank after restarting MCT-1.

The algae's growth created a back pressure in the columns, which was revealed by the pressure gauge installed before the bone char column (see Figure 2-1). The flow rate was reduced to relieve the back pressure. As discussed in the following paragraphs, it appeared that the only measurable effect the biofouling had on the removal efficiency of the clinoptilolite. However, the resulting reduction in flow rate did influence ⁹⁰Sr removal. This is also discussed in the following sections.

At the conclusion of the test, a sample of the algae was obtained and dried to check for radioactivity. The algae was found to be emitting beta radiation at about 8,000 dpm and was concluded to be from uptake of 90 Sr. Biological organisms are known to assimilate strontium as if it were calcium or a calcium substitute. Radiation measurements taken through the glass column in the upper region of the clinoptilolite bed during the test indicated beta radiation levels twice that of the dried algae. To determine if the algal growth interfered with 90 Sr removal effectiveness, a comparison of the results between two lead clinoptilolite columns of separate mini-column tests (2MCT-2, which had significant algae growth and 2MCT-3, which had no algae growth) indicated no measurable effect. This is evident by looking at the effluent results from each lead column (see Figures 3-30 and 3-32).

3.2.3.1.1 MCT-1 Results. The results for Test MCT-1 are presented in Table C-15 (Appendix C) and plotted in Figures 3-24, 3-25, 3-26, and 3-27. The data presented in the table and figures include the initial, as well as the continuation of Test MCT-1. As mentioned previously, the initial run was terminated at approximately 3,500 BV, and then continued for another approximately 4,580 BV (total of $\sim 8,100$ BV). Sampling during the continuation run was intended to monitor for 90Sr saturation or 137Cs breakthrough, whichever occurred first. In looking at the lead column effluent curve (see Figure 3-24), immediately after 3,500 BV up to 4,800 BV, it appears that the lead column approached saturation for ⁹⁰Sr. During this time, the test was run 24 h/day (see Figure 3-24). Additionally, when the test was begun, it ran continuously and was sampled once per day until reaching 1,500 BV, at which time operations were reduced to 8 h/day because the test plan (BHI 1995e) called for increased sampling frequency after 1,500 BV. This could only be accommodated by running 8-h days and sampling every other day. At approximately 1,000 BV, there seemed to be a sharp "breakthrough." When the test was operated discontinuously (8 h/day), a kinetic effect appeared in that removal of ⁹⁰Sr by clinoptilolite seemed to recover after continuous operations were stopped. In addition, at 4,800 BV and again at 6,500 BV, the algal growth caused a reduction in flow (with attendant increase in contact time of 11.5 min after 6,500 BV) and there also was a "recovery." This "stair-step" removal/recovery effect seems to suggest that %Sr removal by clinoptilolite has a dependence on contact time and/or discontinuous operation, or rather, has a kinetic dependence and may be best suited for batch operation. This also could suggest suitability for continuous operation at a long contact time. The effluent concentration reached a low of 1,820 pCi/L at 7,500 BV when the flow rate was reduced, resulting in a contact time of approximately 21 minutes. Generally, the results indicate a better quality effluent when the system was run discontinuously, or in a batch-type operation, and when the contact time was increased.

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The bone char column, as a roughing column for 90 Sr, removed a significant portion of the feed 90 Sr for the first 500 BV. But after 500 BV, the bone char was apparently saturated and started to "leak" small amounts of 90 Sr, causing somewhat sporadic results in the lead clinoptilolite column's test results. The bone char effluent was not monitored for 90 Sr during the mini-column tests because pilot-scale test data indicated that the bone char became loaded after 500 BV and was no longer effective.

The lead clinoptilolite bed (with help from the bone char) removed 95% of the ⁹⁰Sr for the first 500 BV. After the bone char became saturated, the ⁹⁰Sr concentration from the lead clinoptilolite column increased to 1,060 pCi/L at 1,000 BV, resulting in a removal efficiency of approximately 75%. The removal efficiency dropped to around 70% at 1,500 BV and averaged 70% to 3,500 BV. After 3,500 BV, the removal efficiency of the clinoptilolite again dropped, averaging 40% to 45% for the rest of the test. This clearly indicates that clinoptilolite cannot, with any confidence, achieve the MCL target effluent of 8 pCi/L at short contact times.

The second clinoptilolite bed effluent, or system effluent, was monitored at the same frequency as the lead clinoptilolite effluent during the first 3,500 BV. However, because it was determined that adequate data would not be obtained to properly determine the 90 Sr breakthrough curve due to the short duration of this test, the objective of the continuation phase of this test was to look for 90 Sr saturation (effluent equals effluent) or 137 Cs breakthrough. For this reason, overall system removal data was not obtained with any regularity for this portion of this test.

The second clinoptilolite column achieved 90% ⁹⁰Sr removal to 500 BV, but then dropped to 74% by 1,500 BV (see Figure 3-25). The removal efficiency averaged about 75% to 3,500 BV and decreased to between 50% and 60% by the end of the test. For the first 500 BV, this provided the overall system with fair ⁹⁰Sr removals, with one data point less than 8 pCi/L. After 500 BV, however, the effluent increased sharply to an average 605 pCi/L ⁹⁰Sr (a range of 305 to 768 pCi/L).

The ¹³⁷Cs results seen in Table C-16 (Appendix C) show less-than-detectable values (~60 pCi/L) from the lead clinoptilolite column (MCT-1-3 column in table) effluent except for three data points at 4,800, 6,900, and 7,700 BV. The influent concentration showed a steadily increasing trend starting at approximately 4,200 BV, yet the ¹³⁷Cs removals held constant (see Figure 3-26). At 7,700 BV, there was a sudden increase in influent concentration with a parallel increase in effluent concentration of 230 pCi/L, and the two data points after this were again below the detection limit following the decrease in influent concentration. Regardless, no breakthrough was seen relative to ¹³⁷Cs after processing 8,100 BV when the test was terminated. It appears from this that clinoptilolite is a good adsorbent for ¹³⁷Cs.

Plutonium-239/240 values were below the 222-S Laboratory detection (~10 pCi/L) in the bone char effluent to 3,500 BV. The ^{239/240}Pu effluent concentration was not checked again until 8,100 BV had been processed, and the value was still below detection. As discussed in Section 3.1.1.1, because of high background levels at the 222-S Laboratory the detection limits for low-level analyses of plutonium and cesium varied from day-to-day. This produced varying results. For example, referring to Table C-15 (Appendix C) and Figure 3-27, at 3,100 BV, the effluent concentration appears to be greater than the influent. However, the effluent concentration of 35 pCi/L is below detection limit, whereas the influent concentration of 14.3 pCi/L is a detectable value. It can be seen that the data plot in Figure 3-27 is sporadic and indicates inconsistent data from which to draw a conclusion.

A sample was collected at 2,600 BV for analysis at an offsite laboratory as a QC check against the 222-S Laboratory data (these data are shown in Table C-15, Appendix C). The ⁹⁰Sr concentration from the lead clinoptilolite column effluent from the offsite laboratory was 1,180 pCi/L, and the corresponding value from the 222-S Laboratory was 1,120 pCi/L. The ¹³⁷Cs value from the 222-S Laboratory from the same location was below detection (60.5 pCi/L), and the value from the offsite laboratory was -1.59 pCi/L. The negative value represents a value that is less than the background found at the laboratory. The ^{239/240}Pu value from the 222-S Laboratory was less than detection (10.7 pCi/L) and 0.742 pCi/L from the offsite laboratory. The lower detection limit of the offsite laboratory shows that the system was capable of treating to values less than the DWS of 1.2 pCi/L ^{239/240}Pu and 120 pCi/L ¹³⁷Cs. No other offsite laboratory samples were taken for this test.

3.2.3.1.2 MCT-2 Results. This test was done to evaluate the effect of adding a 2-ppm spike of SrCl₂ to each of the clinoptilolite columns for the removal of ⁹⁰Sr (BHI 1995e), and was designated as "MCT-2. Because of an error in making up the strontium solution (it was to be 80 ppm SrCl₂ as strontium) plus difficulties with the metering pump, the strontium concentrations were 0.6 to 0.8 ppm instead of 2 ppm. After the total strontium was found to be low, the feed rate was adjusted, and at 2,600 BV the feed strontium was up to 1.4 ppm. At 3,500 BV it was at 2.12 ppm. Comparing the total strontium (0.167 ppm) with the ⁹⁰Sr concentration (4,205 pCi/L, or 2.982 x 10⁻⁸ ppm) in the feed groundwater shows that the ⁹⁰Sr concentration is an insignificant portion of the measured total strontium. Therefore, the ⁹⁰Sr removed by the clinoptilolite in the lead column does not affect the value of total strontium measured after the second SrCl₂ addition.

The addition of approximately 1 ppm total strontium in the feed streams to the clinoptilolite columns appeared to reduce the effectiveness of the clinoptilolite for ⁹⁰Sr removal. The MCT-2 lead clinoptilolite column effluent concentrations followed the same trends as that of MCT-1 but were consistently about 300 to 400 pCi/L higher (see Table C-16, Appendix C, and Figure 3-28). The removal efficiencies of the two tests were similar while the bone char was actively removing ⁹⁰Sr (to 500 BV), but the MCT-2 lead clinoptilolite column effluent concentrations increased and stayed consistently higher by approximately 300 to 400 pCi/L. Both tests were otherwise run identically.

Figure 3-29 compares overall system removals of ⁹⁰Sr for the MCT-1 and MCT-2 tests. Although the second MCT-2 clinoptilolite column removal efficiency was nearly the same as that of MCT-1, the effluent concentration stayed consistently approximately 300 pCi/L higher than the effluent from MCT-1 because of the higher inlet concentration.

The addition of $SrCl_2$ did not appear to affect the performance of the clinoptilolite for ^{137}Cs or $^{239/240}Pu$ removal. Effluent concentrations for both radionuclides were below the detection limit ($\sim 60~pCi/L$ ^{137}Cs and $\sim 10~pCi/L$ $^{239/240}Pu$) throughout the test.

Offsite laboratory results were consistent with the 222-S Laboratory for feed ^{239/240}Pu. Offsite laboratory results for a sample and a duplicate for ¹³⁷Cs were 1.96 and 2.02 pCi/L, respectively; the 222-S Laboratory result was less than 58 pCi/L. The offsite laboratory reported 0.524 pCi/L and 0.491 pCi/L for a ^{239/240}Pu sample and duplicate. The 222-S Laboratory reported less than 5.24 pCi/L. These results again indicate that the system is capable of treating to less than the target MCLs for ¹³⁷Cs and ^{239/240}Pu.

3.2.3.2 216-B-5 Reverse Well Test Group II. After evaluation of the data obtained from mini-column tests MCT-1 and MCT-2, and because of the poor performance of the clinoptilolite for ⁹⁰Sr removal, it was decided to perform another group of mini-column tests. MCT-1 was to be

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continued to look for saturation of 90 Sr or breakthrough of 137 Cs. The second test, 2MCT-2, was to be a kinetic evaluation of clinoptilolite to see if a longer (12 min) contact time would improve 90 Sr removal. This is equivalent to running the pilot-scale system at 38 L/min (10 gal/min) as compared to the normal processing rate of 95 to 114 L/min (25 to 30 gal/min). In the third test, another adsorbent, UOP A-51, was selected from the batch equilibrium tests for evaluation in a mini-column test (designated 2MCT-3). These tests were performed simultaneously.

The equipment used for MCT-2 was cleaned and reused for the 2MCT-2 test. However, residual algae from the feed tank or other source apparently entered the 2MCT-2 equipment because it became infested with algae during this test. Test 2MCT-3 consisted of all new equipment and materials. Test 2MCT-2 showed no biological growth until near the end of the test. The growth, appearing on the top of the UOP A-51 bed, did not appear to be algae, but possibly a bacterial growth that was white and slimy in appearance. This contrasted with the definite green color of the algae. There was no discernible difference in the results of either test; therefore, it appears that the growths had no significant impact.

3.2.3.2.1 2MCT-2 Results. 2MCT-2 was run in the same configuration as MCT-1 (bone char, clinoptilolite, clinoptilolite), but at a 12-minute contact time per column. The test was run 24 h/day for 6 weeks, processing a total of approximately 2,900 BV. The test was operated 24 h/day to process the maximum amount of groundwater possible in the 6 weeks of available testing time.

The lead clinoptilolite column ⁹⁰Sr effluent concentration followed the same "stair-step" path with an overall downward trend (see Table C-17, Appendix C, and Figure 3-30) as was noted in the pilot-scale system (discussed in Sections 3.1.1.1 and 3.2.4.1.1). The data again indicate the same kinetic dependence as noted in Test MCT-1. The high effluent concentrations occurred at the end of the week (i.e., either a Thursday or Friday) and the low effluent concentrations occurred at the beginning of the week, after recovering over the weekend.

For the first 200 BV, the bone char acted as a roughing column for 90 Sr. After 630 BV, the bone char effluent was not monitored for 90 Sr because the bone char column was assumed to have reached saturation. At approximately 1,500 BV, the bone char removed approximately 46% of the 90 Sr. However, at 1,900 BV it appeared to be releasing some of the 90 Sr. It appears that after the bone char becomes saturated, some cation is being preferentially loaded, thereby pushing off the 90 Sr.

The second clinoptilolite column demonstrated a ⁹⁰Sr removal efficiency greater than 90% to 1,000 BV. Beyond 1,250 BV, it demonstrated an average removal efficiency of 90%. At the 1,250 BV sampling event for both 2MCT-2 and 2MCT-3, the samples were mislabeled when turned in to the laboratory. The laboratory corrected the error; however, these data are suspect and appear anomalous (see Figure 3-31). The overall system achieved a ⁹⁰Sr removal efficiency that ranged from approximately 99.9% through 1,000 BV to 96.8% at 2,900 BV. For the first 1,000 BV, the effluent concentration was less than the target MCL of 8 pCi/L ⁹⁰Sr. It appears that longer contact time improved ⁹⁰Sr removals.

3.2.3.2.2 2MCT-3 Results. 2MCT-3 was set up similarly to 2MCT-2, except that a UOP A-51 column was added at the end instead of clinoptilolite and consisted of bone char, clinoptilolite, and UOP A-51 (a synthetic zeolite/molecular sieve) columns. 2MCT-3 was run 24 h/day for 6 weeks. A total of 2,900 BV were processed at a contact time of 12 minutes per column.

The bone char again removed a significant percentage of the ⁹⁰Sr to 200 BV and then became saturated. The sample data at 1,250 BV indicate a 99% ⁹⁰Sr removal by bone char (see Table C-18, Appendix C, and Figure 3-32). As in the 2MCT-2 results, however, there were some mislabeled samples at this data collection (1,250 BV) that may be considered anomalous.

For the first 200 BV, while the bone char bed was actively removing 90 Sr, the lead clinoptilolite bed was removing >99% of its influent (the bone char effluent). Once the bone char became saturated, the lead clinoptilolite bed's efficiency dropped to approximately 90% until 1,250 BV, and then averaged 80% through the end of the test.

The removal efficiency of the lead column followed the same downward-trending "stair-step" pattern as 2MCT-2 (see Figure 3-30), but was not as pronounced. The removal efficiency curve (dashed line) again indicates low and high removals corresponding with the ends and beginnings of the week, respectively. At 2,500 BV (Monday) the removal efficiency did not recover as in previous weeks. The mini-column tests were restarted on Sunday morning after being shut down Friday afternoon, which allowed only one day to sit idle. This may have contributed to the lower recovery observed on Monday. The removal efficiency did not increase throughout this week (through 2,900 BV) as in previous weeks. A leak in the line at the feed pump was found Friday morning (2,900 BV) and repaired before sampling. The amount of water lost (all of which was held within the containment structure) indicated that 2MCT-3 had been without a feed stream for 1 to 2 h. Sampling of 2MCT-3 occurred approximately 1.5 h after the feed stream was reestablished. The pause in operations, which allowed a lengthy contact time within the columns, may have contributed to maintaining the level of removal efficiency.

The UOP A-51 maintained a removal efficiency greater than 90% to the end of test. A downward trend in the removal efficiency curve after 1,250 BV is illustrated in Figure 3-33. The last data point indicated a removal efficiency of 89.9%.

The system removal efficiency with the UOP A-51 adsorbent was greater than 95% for the duration of the test. The overall system effluent concentration was less than the target MCL of 8 pCi/L for 90 Sr for the first 1,000 BV, although it was sporadic because of the detection limits.

3.2.3.3 216-BY Cribs Mini-Column Tests. Based on previous batch equilibrium studies by Barney (WHC 1994) and the successful removal of technetium in mini-column and pilot-scale treatment tests at 200-UP-1 Operable Unit, Dowex 21K, a strong base anion gel-type resin, was selected for testing in the BY-Cribs pilot-scale treatability test. Because it did not appear that sufficient loading data would be provided from the pilot-scale treatment system, mini-column tests were conducted at the BY-Cribs site to provide more data. These used Dowex 21K at a contact time simulating the pilot-scale system. The 200-UP-1 results for 99Tc indicated that Dowex 21K became saturated at approximately 2,600 BV in the presence of uranium. However, because there was little or no uranium in the BY-Cribs groundwater, it was expected that 99Tc would be able to process approximately 4,000 to 5,000 BV before saturation. Additionally, because the geology of the BY-Cribs site was different from that at the 200-UP-1 site, other anions or cations may preferentially load on the Dowex 21K, thereby reducing or interfering with 99Tc loading.

The two mini-column tests conducted at the BY-Cribs site processed approximately 5,900 BV each (bed volume = 460 mL). The results are shown in Tables C-19 and C-20 (Appendix C). Figures 3-34 through 3-37 are plots of the ⁹⁹Tc data. Figures 3-34 and 3-36 show the ⁹⁹Tc results for the lead columns of both tests. Both curves are the typical sharp breakthrough curves normally

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experienced by a resin that performs well. The curves indicate that 99 Tc saturation occurred at between 4,500 to 5,000 BV. A 30% breakthrough occurred at between 3,300 to 3,500 BV, and 50% breakthrough occurred at 3,800 BV. The average 99 Tc removal for both lead columns was 90% to 30% breakthrough and 82% to 84% to 50% breakthrough.

Figures 3-38 and 3-39 are plots of the ⁶⁰Co data for the lead columns of Tests 1 and 2. Unfortunately, the 222-S Laboratory's detection limit for ⁶⁰Co averaged approximately 38 pCi/L, but when used for an effluent value, the lead column demonstrated a removal efficiency of greater than 75%. Because the ⁶⁰Co effluent from the lead column was below the detection limit, the polish column feed and effluent results were expected to be, and were, the same. Thus, these results were not plotted.

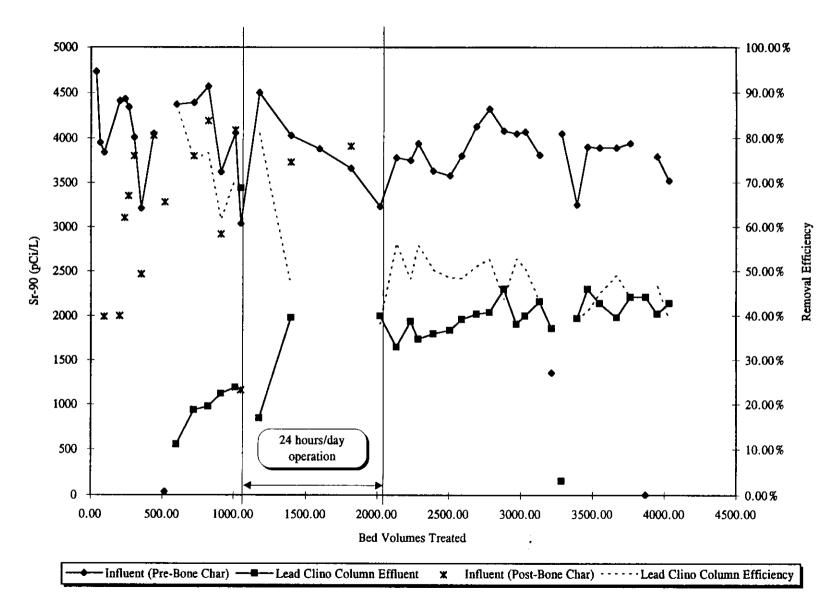
The offsite detection limits for both ⁶⁰Co and ⁹⁹Tc are 10.5 and 2.5 pCi/L, respectively (with 95% upper confidence limit [UCL]). Comparing one offsite ⁶⁰Co sample result with a similar sample result (taken at same time and point) from the 222-S Laboratory shows a detection limit of 10.5 pCi/L compared to 42.7 pCi/L at the 222-S Laboratory. It should be noted that the offsite laboratory result was actually 2.93 pCi/L, but this was at a lower confidence level than the minimum detectable activity/amount (which is the 95% UCL detection limit). In any case, with a feed concentration of 168 pCi/L and a 10.5 pCi/L effluent, the resulting removal efficiency would be 93.8%. If the 2.93 value were used, it would be 98.3% removal.

3.2.4 Strontium-85 Data for 100-N Area

An unpublished report by PNNL documents data developed for the 100-N Area ion-exchange pump-and-treat system. Four mini-column, single-bed, flow-through adsorption tests were run using AmReC clinoptilolite. Three tests were run at residence times of 13.4 min/column with about 30 mg/L of calcium added to the influent to approximate groundwater quality conditions. Test conditions and breakthrough of ⁸⁵Sr, used as a replacement for ⁹⁰Sr, are given in Table 3-5. The other test was run with the same calcium content and had an even longer residence time. A fifth test was done with 27-minute empty-bed residence time, but with about six times more calcium added. Breakthrough occurred almost immediately. PNNL concluded that clinoptilolite-exchange columns could effectively adsorb ⁹⁰Sr from groundwater if the groundwater had the lower ionic strengths resulting from the observed calcium content.

3-24

Figure 3-1. Pilot-Scale System Lead Column Strontium-90 Effluent Versus Bed Volumes Treated, Run 1.



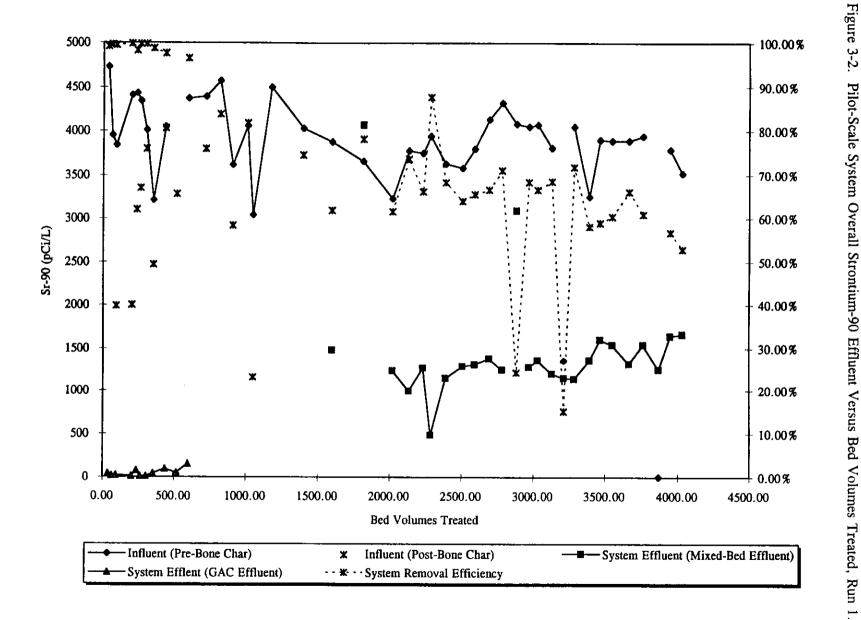


Figure 3-3. Pilot-Scale System Lead Column Cesium-137 Effluent Versus Bed Volumes Treated, Run 1.

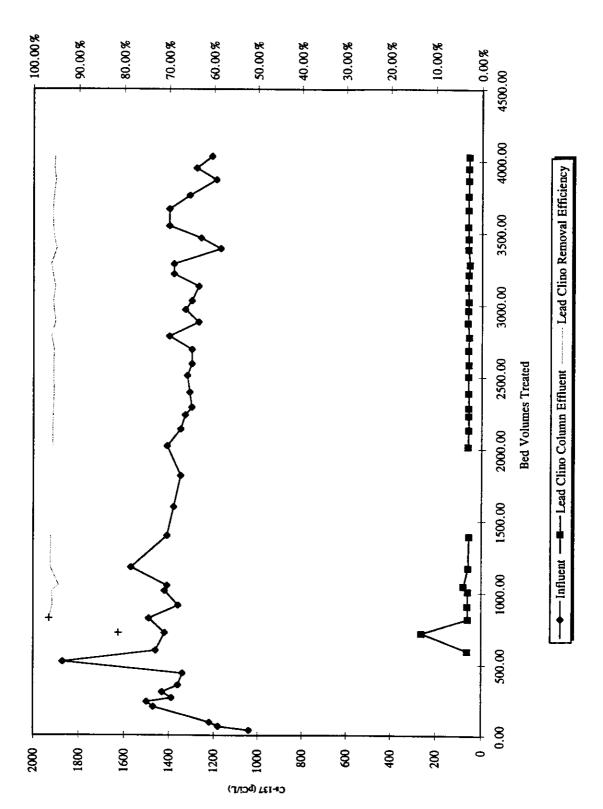
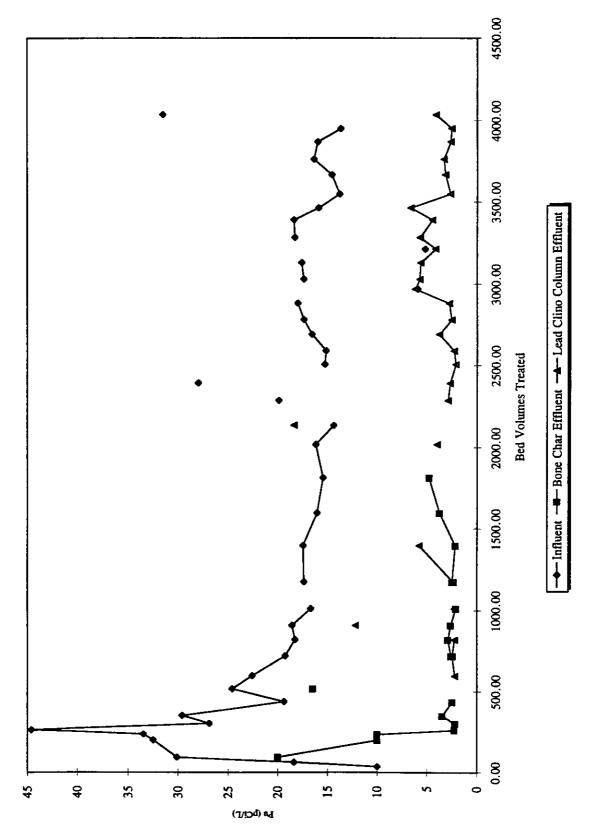


Figure 3-4. Pilot-Scale System Plutonium-239/240 Effluent Versus Bed Volumes Treated, Run 1.





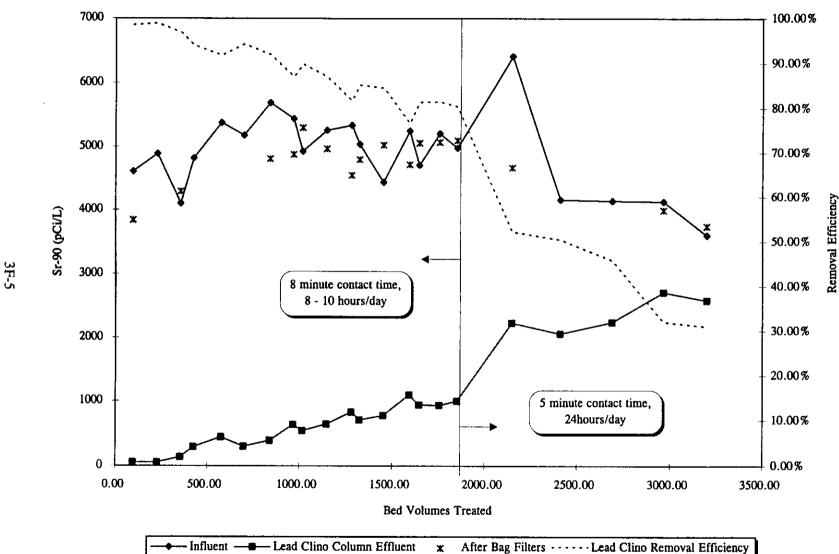
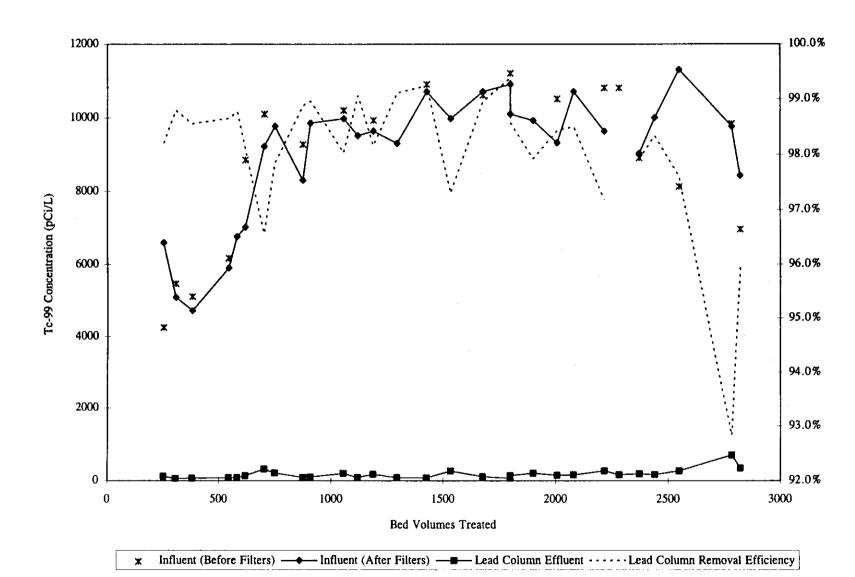


Figure 3-5. Pilot-Scale System Lead Column Strontium-90 Effluent Versus Bed Volumes Treated, Run 2.

7000 100.00% 90.00% 6000 80.00% 5000 70.00% 60.00% 4000 Sr-90 (pCi/L) x 50.00% 3000 40.00% 30.00% 2000 20.00% 1000 10.00% 0.00% 0.00 500.00 1000.00 1500.00 2000.00 2500.00 3000.00 3500.00 Bed Volumes Treated -- Influent After Bag Filters —■— System Effluent (Second Clino Effluent) · · System Removal Efficiency - Second Clino Removal Efficiency

Figure 3-6. Pilot-Scale System Overall Strontium-90 Effluent Versus Bed Volumes Treated, Run 2.



Pilot-Scale System Lead Column Technetium-99 Effluent Versus Bed Volumes Treated.

Figure 3-8. Pilot-Scale System Lead Column Cobalt-60 Effluent Versus Bed Volumes Treated.

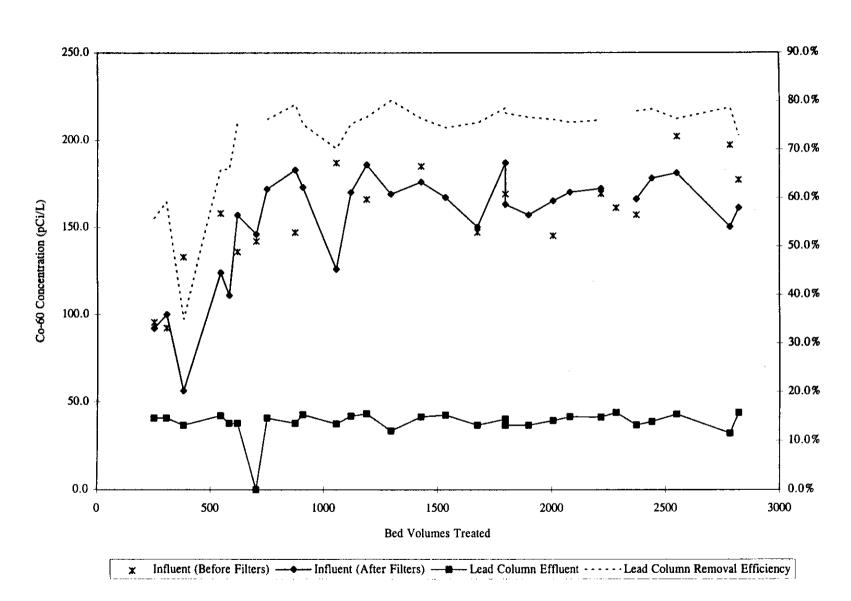


Figure 3-9. IRC-718 Isotherm for Strontium-90.

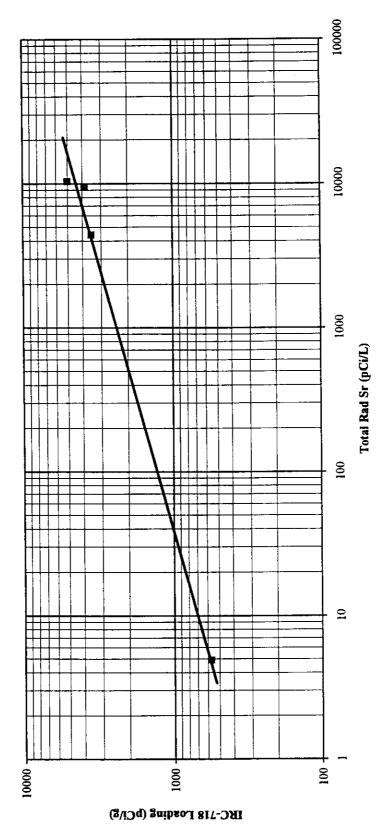
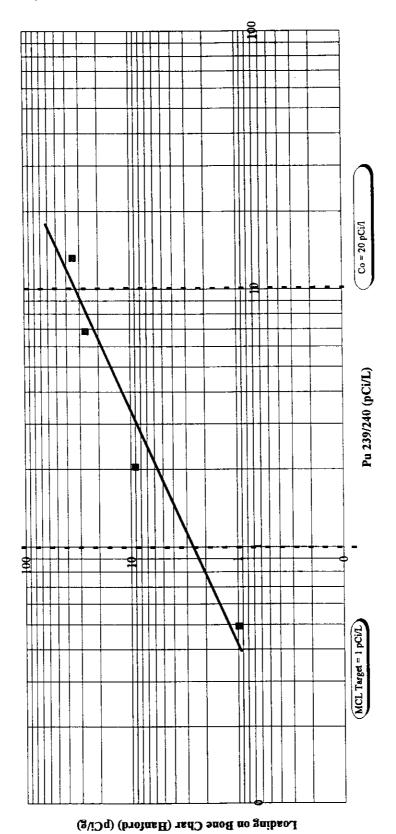


Figure 3-10. Hanford Bone Char Isotherm for Plutonium.



3F-10

Figure 3-11. Resintech Bone Char Isotherm for Plutonium.

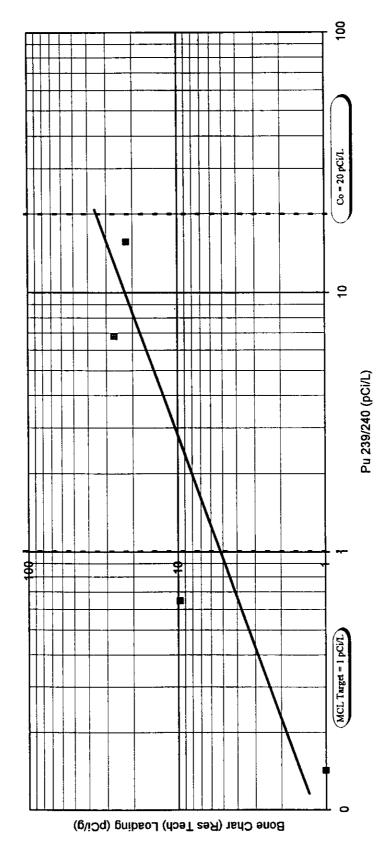


Figure 3-12. Resintech Bone Char Isotherm for Strontium-90.

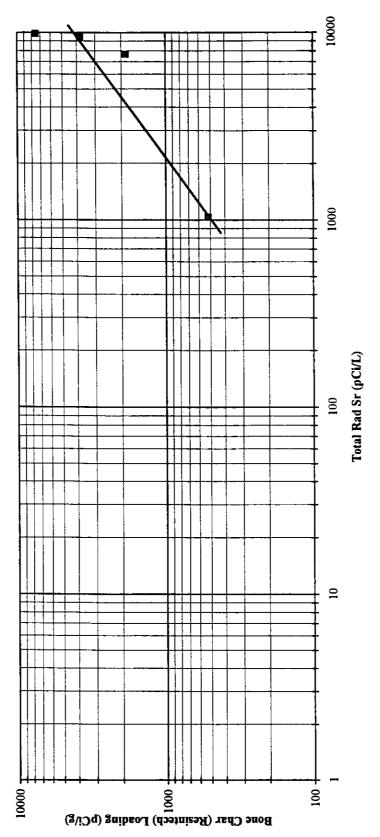


Figure 3-13. Resintech Clinoptilolite Isotherm for Strontium-90.

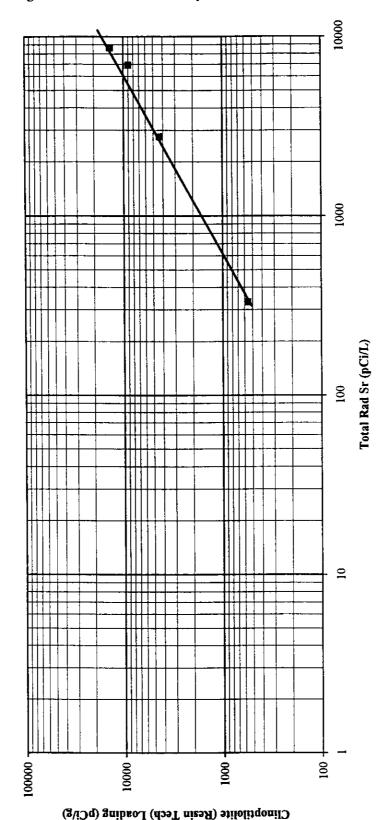


Figure 3-14. Resintech Clinoptilolite Isotherm for Cesium-137.

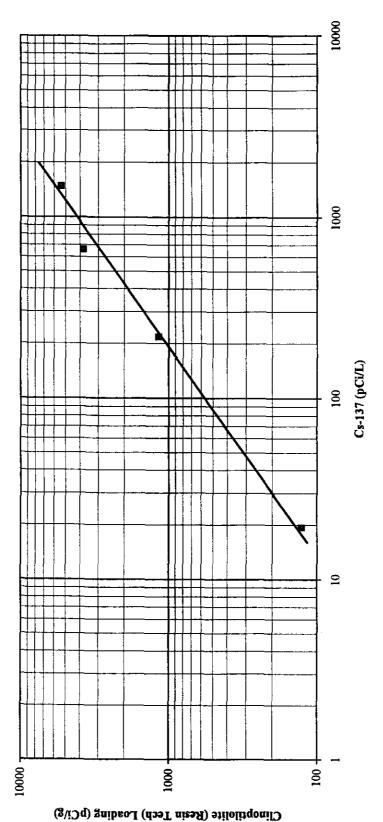


Figure 3-15. American Resources Corporation Clinoptilolite Isotherm for Strontium-90.

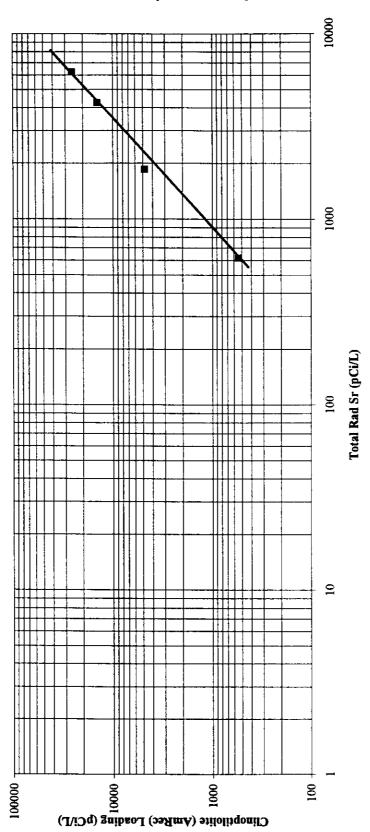


Figure 3-16. American Resources Corporation Clinoptilolite Isotherm for Cesium-137.

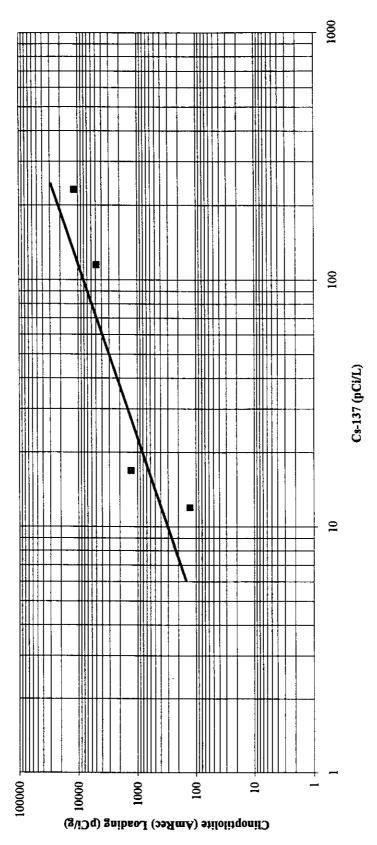
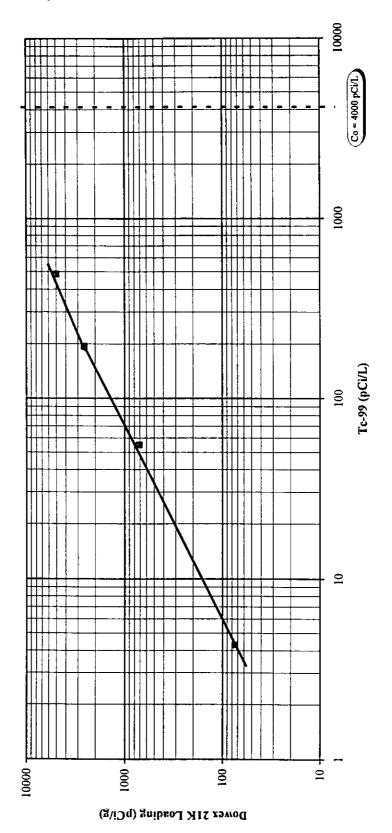


Figure 3-17. Dowex 21K Isotherm for Technetium-99.



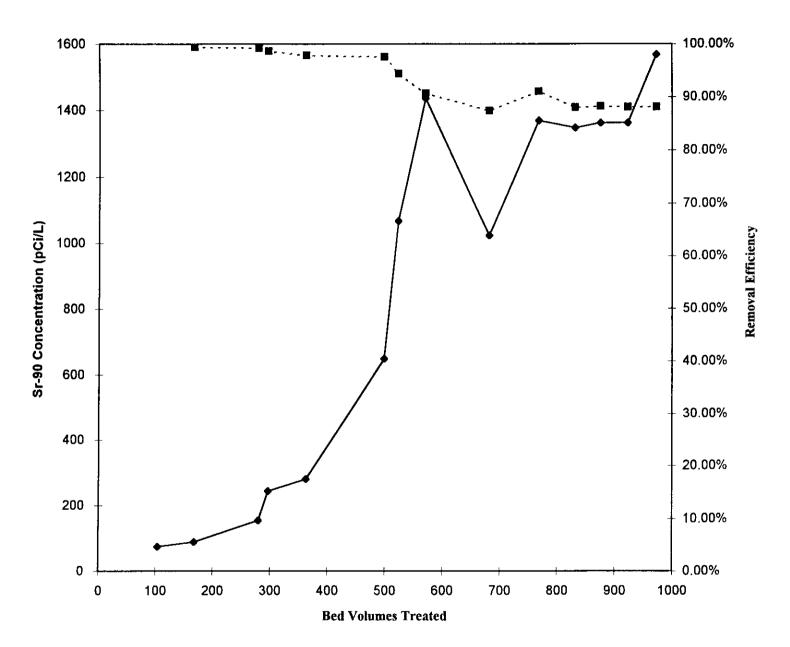
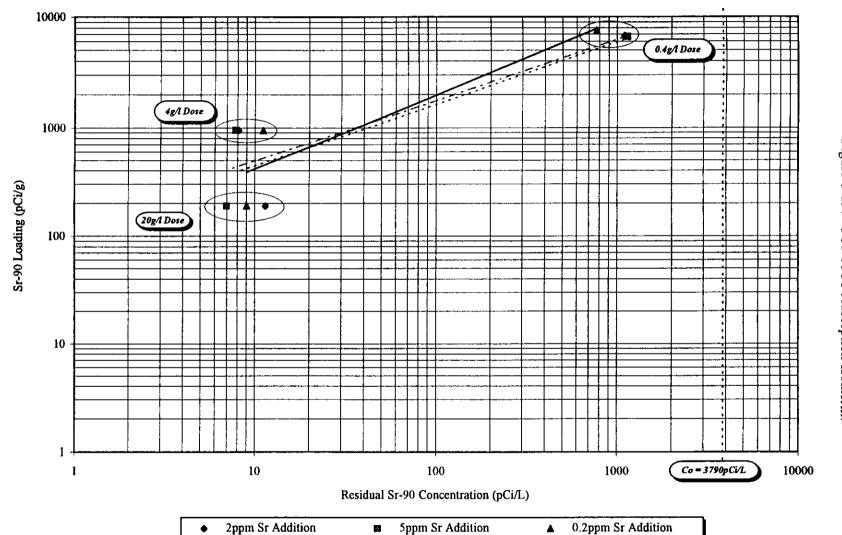


Figure 3-18. Clinoptilolite Flow-Through Column Test Plot (Knoxville Tests).

3F-19



- 2ppm Addition Best Fit ---- Sppm Addition Best Fit

0.2ppm Addition Best Fit

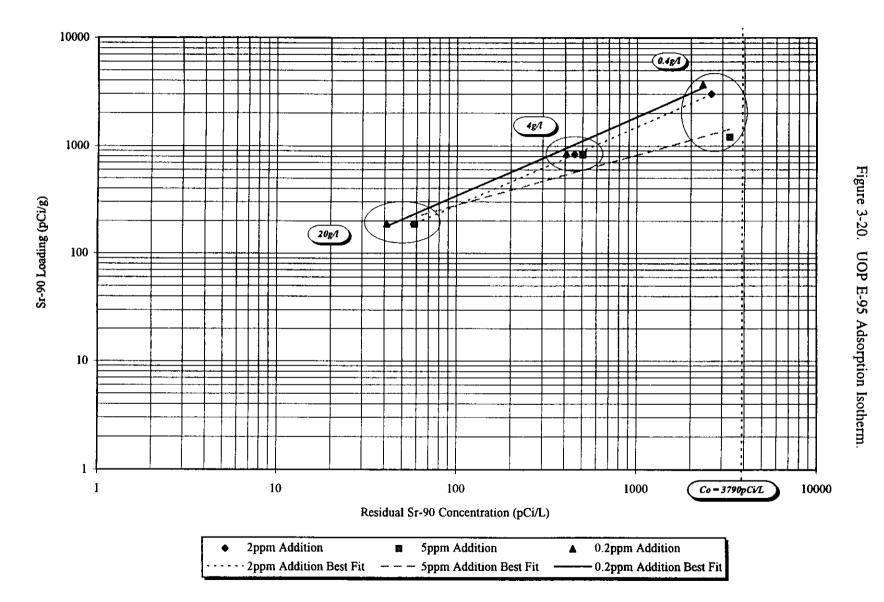
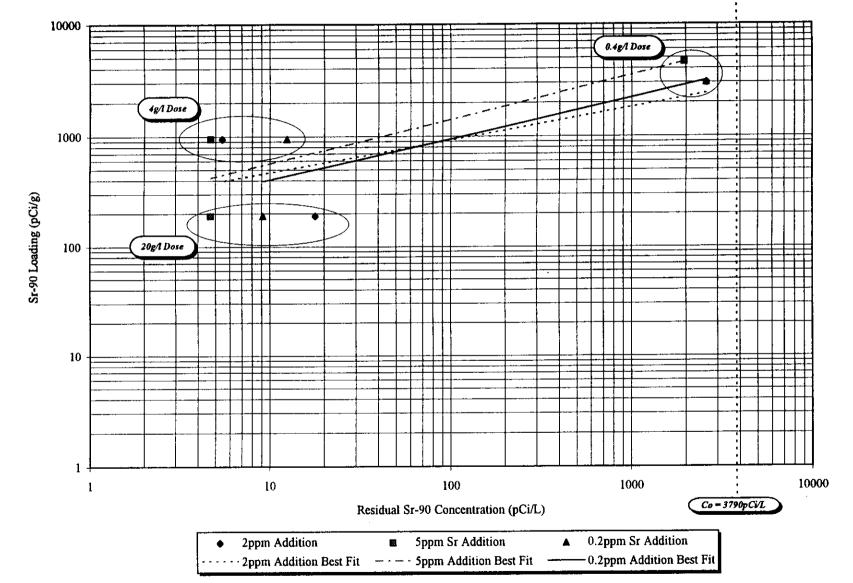
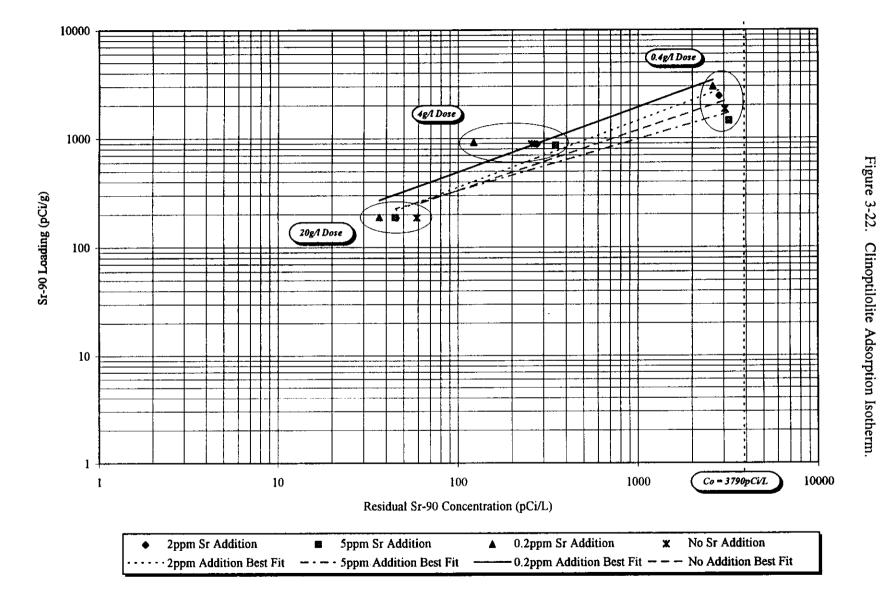
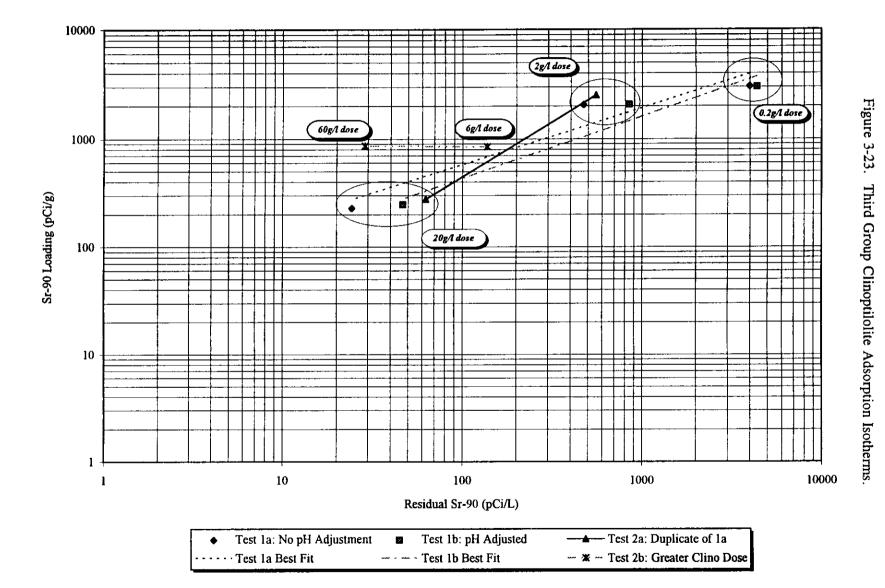


Figure 3-21.

IRC-718 Adsorption Isotherm.







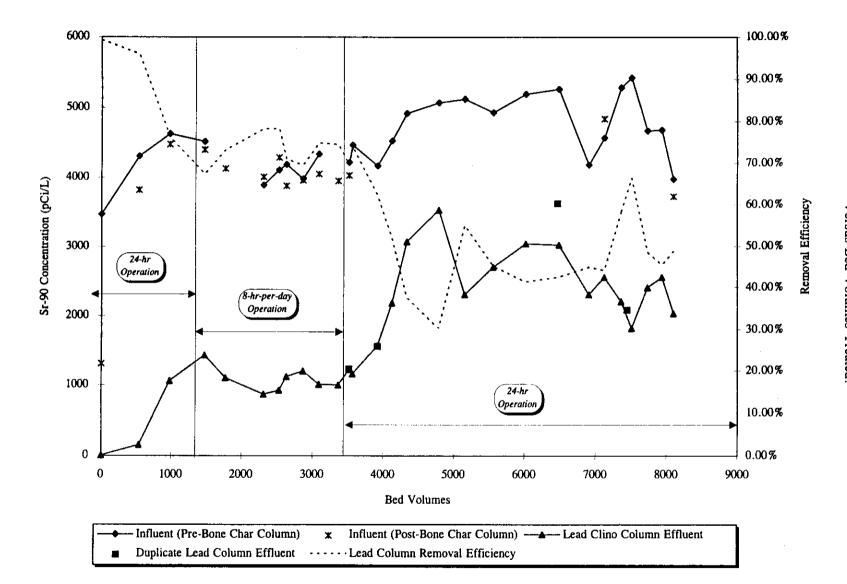


Figure 3-24. MCT-1 Lead Clinoptilolite Column Strontium-90 Removal Versus Bed Volumes Treated.

Figure 3-25. MCT-1 Overall System Strontium-90 Removal Versus Bed Volumes Treated.

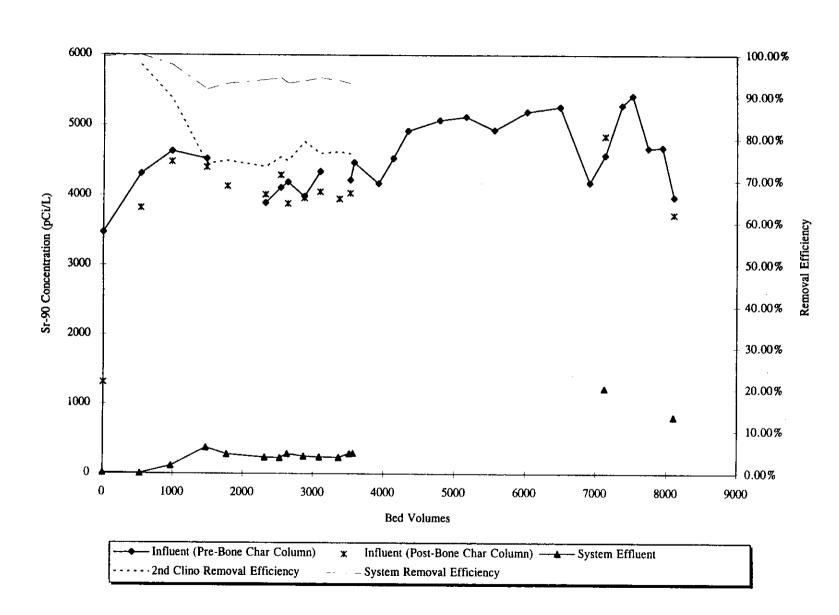


Figure 3-26. MCT-1 Lead Clinoptilolite Column Cesium-137 Removal Versus Bed Volumes Treated.

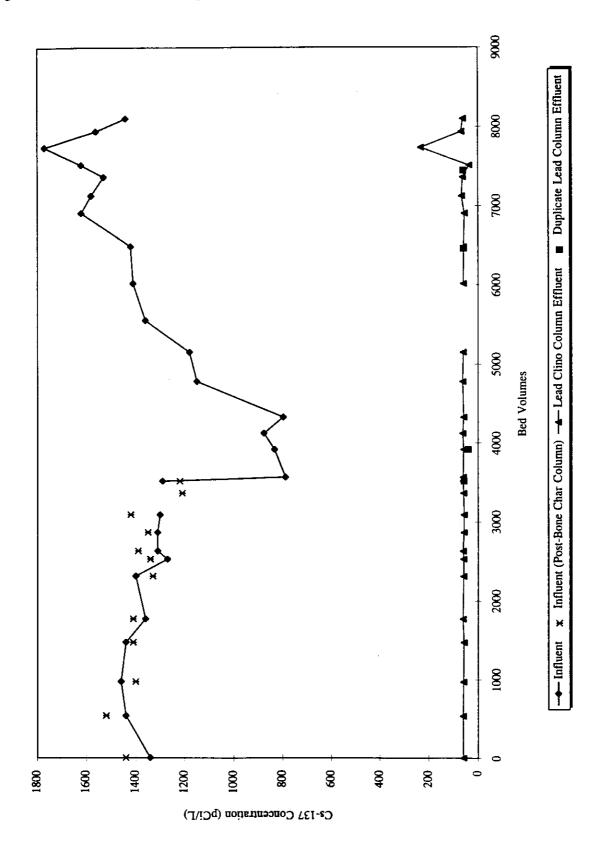


Figure 3-27. MCT-1 Bone Char Column Plutonium-239/240 Removal Versus Bed Volumes Treated.

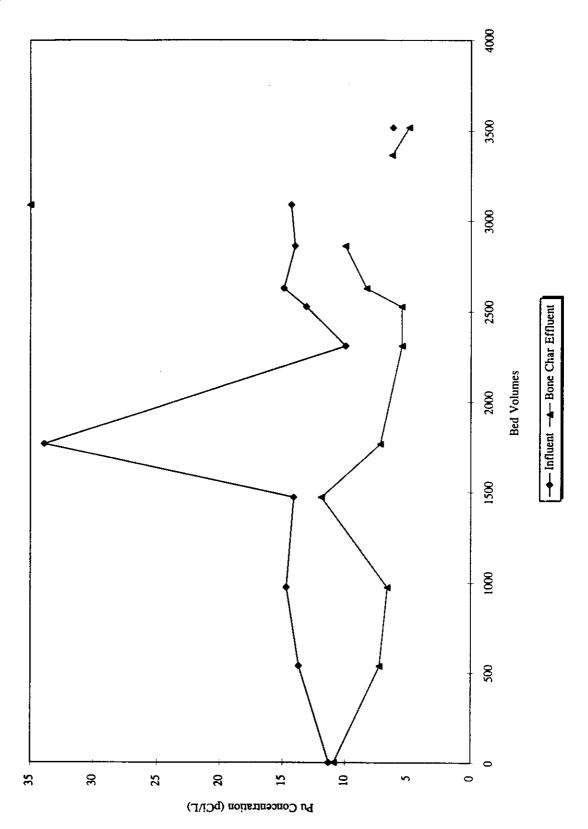


Figure 3-28. MCT-1 and MCT-2 Lead Column Strontium-90 Removal Versus Bed Volumes Treated

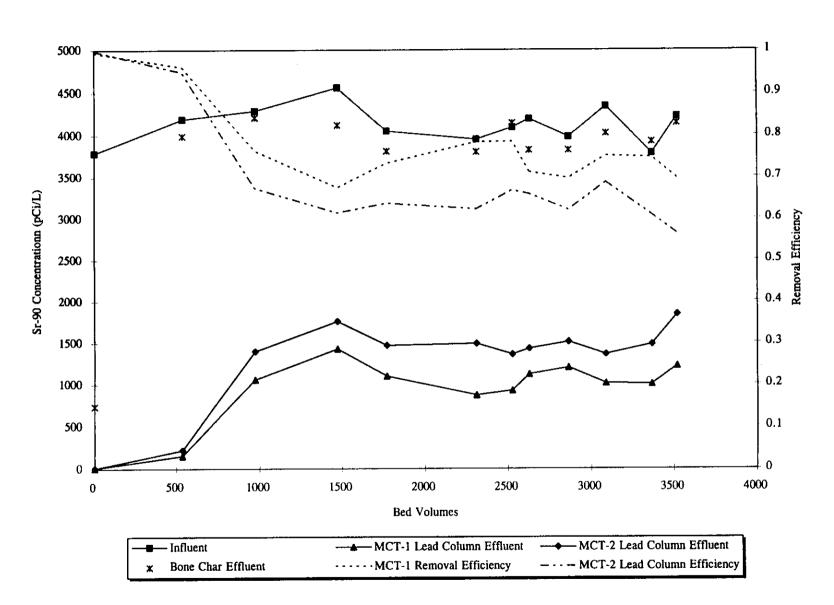


Figure 3-29. MCT-1 and MCT-2 Overall Strontium-90 Removal Versus Bed Volumes Treated.

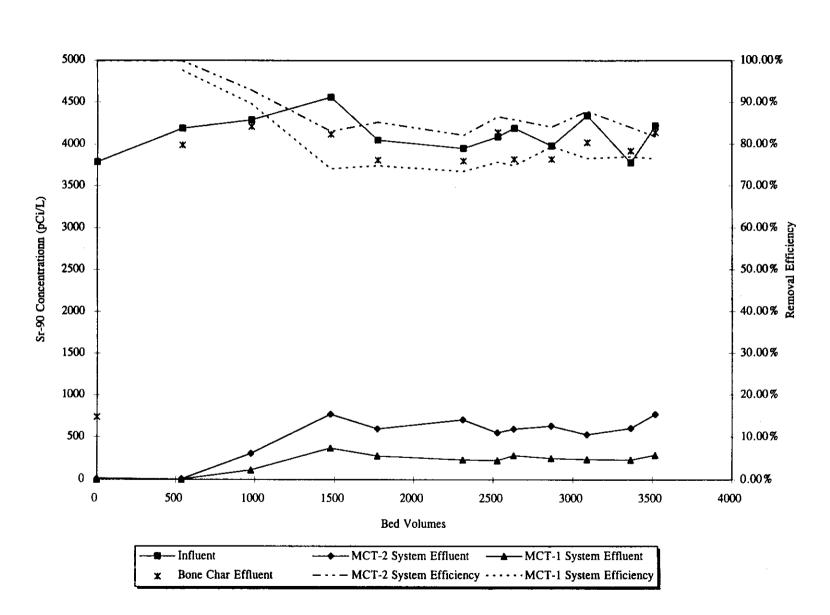
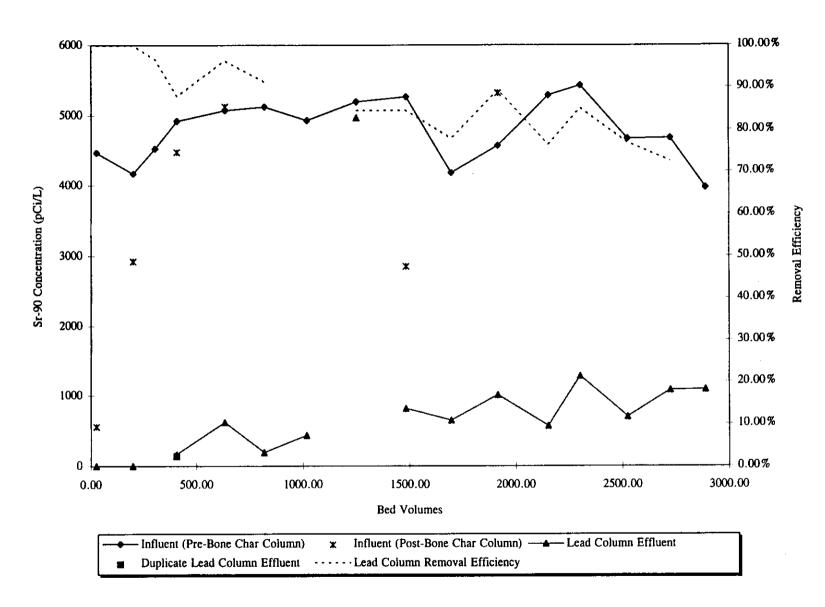


Figure 3-30. 2MCT-2 Lead Clinoptilolite Column Strontium-90 Removal Versus Bed Volumes Treated.



6000 100.00% 90.00% 5000 80.00% 70.00% 4000 Sr-90 Concentration (pCi/L) 60.00% 3000 50.00% × 40.00% 2000 30.00% 20.00% 1000 10.00% 0.00% 0.00 500.00 1000.00 1500.00 2000.00 2500.00 3000.00 **Bed Volumes** Influent (Pre-Bone Char Column) Influent (Post-Bone Char Column) —▲— System Effluent Duplicate System Effluent -----2nd Clino Col. Removal Efficiency --- System Removal Efficiency

Figure 3-31. 2MCT-2 Overall System Effluent Strontium-90 Removal Versus Bed Volumes Treated.

Figure 3-32. 2MCT-3 Lead Clino Column Strontium-90 Removal Versus Bed Volumes Treated

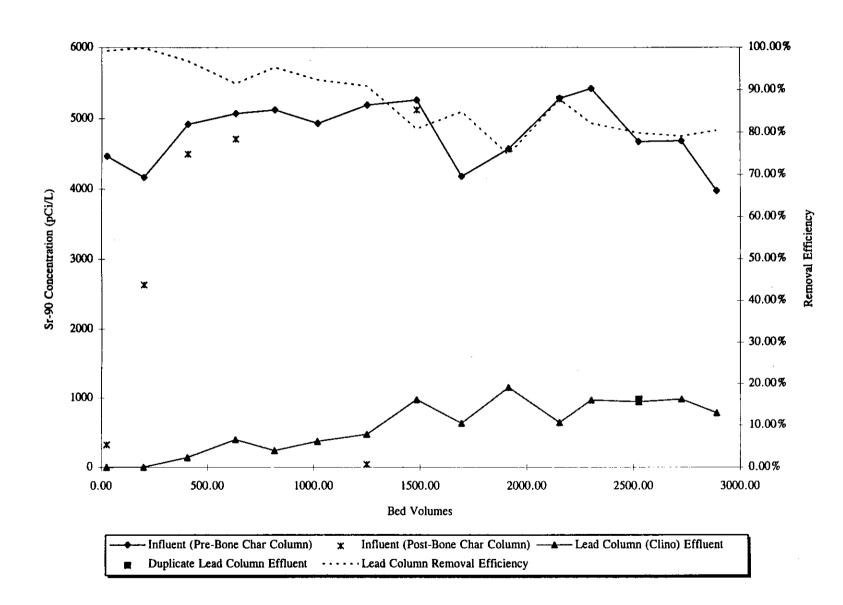
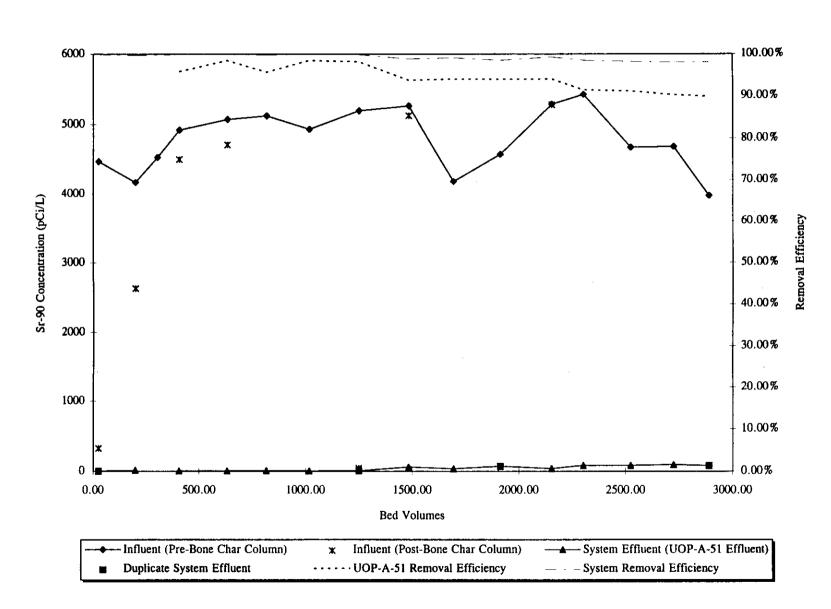


Figure 3-33. 2MCT-3 Overall System Strontium-90 Removal Versus Bed Volumes Treated.



3F-33

Figure 3-34. Lead Mini-Column Technetium-99 Effluent Versus Bed Volumes Treated, Test 1.

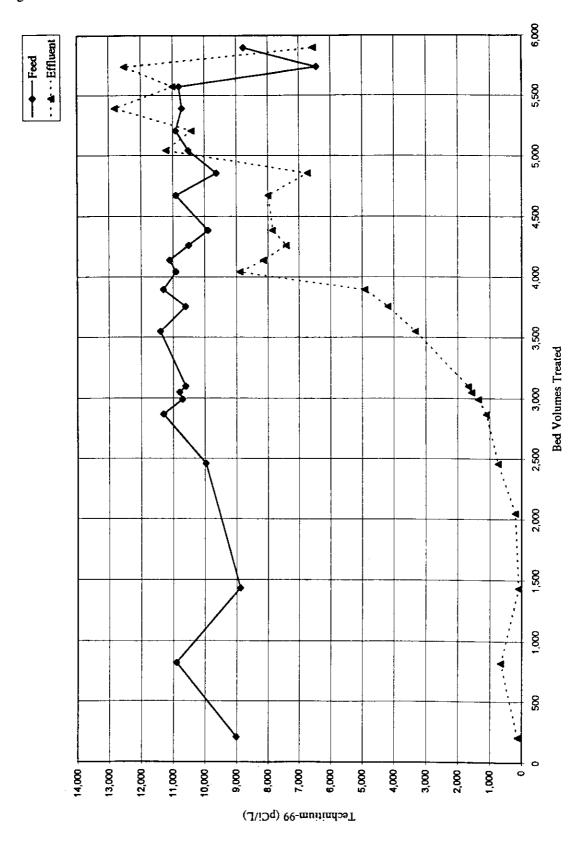


Figure 3-35. Overall Mini-Column Technetium-99 Effluent Versus Bed Volumes Treated, Test 1.

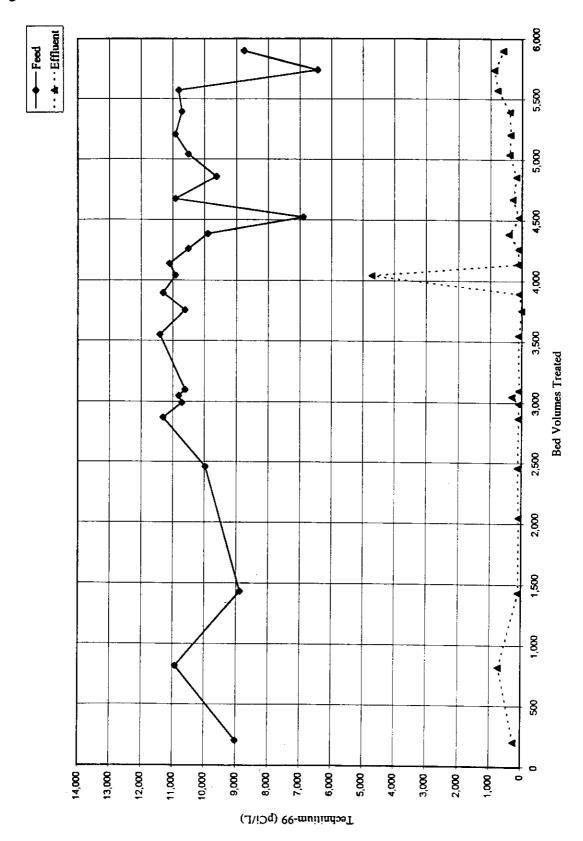


Figure 3-36. Lead Mini-Column Technetium-99 Effluent Versus Bed Volumes Treated, Test 2.

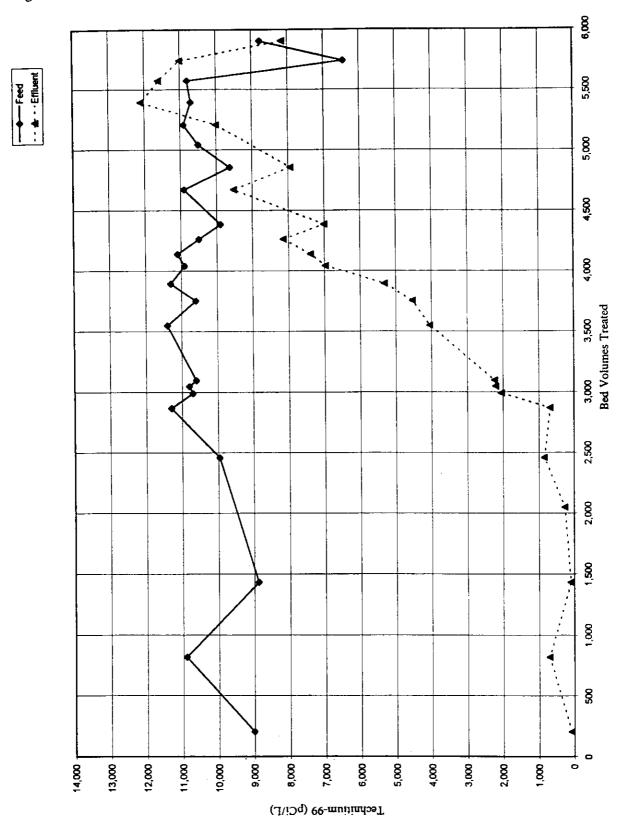


Figure 3-37. Overall Mini-Column Technetium-99 Effluent Versus Bed Volumes Treated, Test 2.

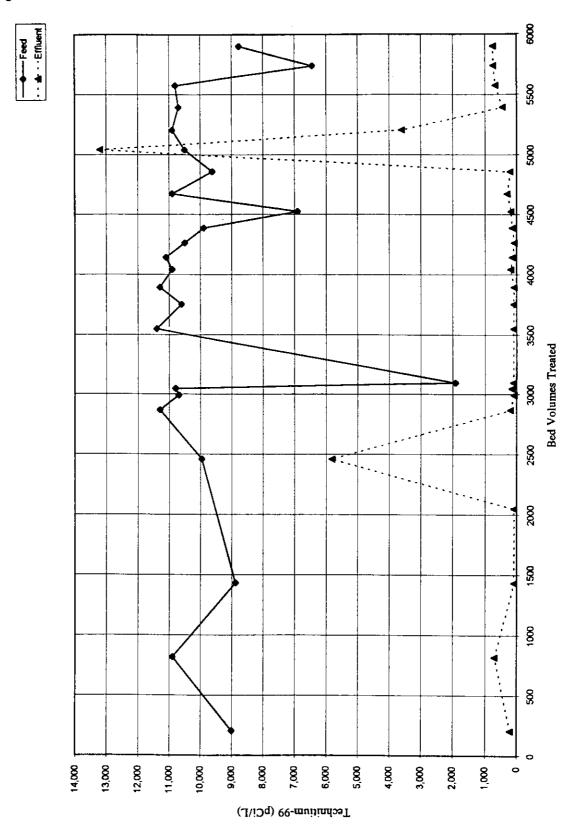


Figure 3-38. Lead Mini-Column Cobalt-60 Effluent Versus Bed Volumes Treated, Test 1.

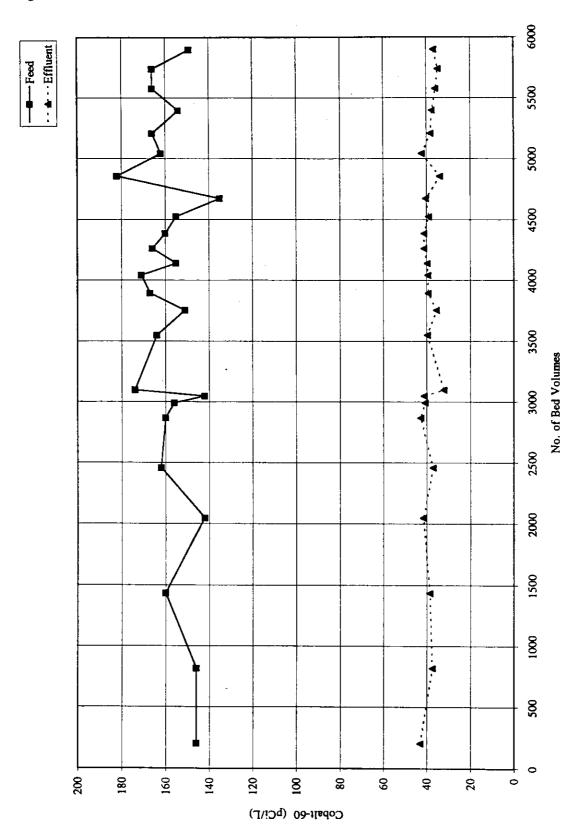
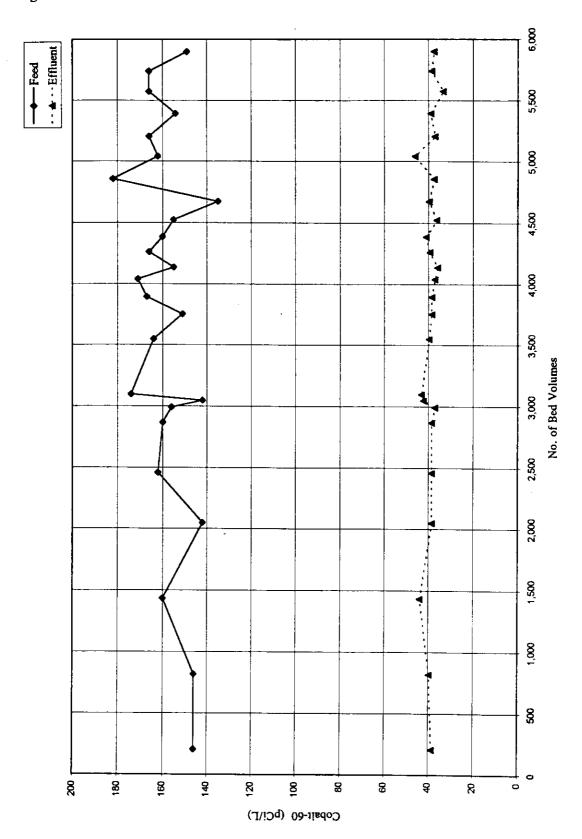


Figure 3-39. Lead Mini-Column Cobalt-60 Effluent Versus Bed Volumes Treated, Test 2.



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Table 3-1. Lead Column Removal Performance, Quantities of Radionuclides.

Contaminant	Average Concentra- tion of Well Influent (pCi/L)	Average Concentration of Lead Column Effluent (pCi/L)	Liters (gal) treated	Total pico- Curies Removed	Specific Activity	Total Grams Removed
		216-F	3-5 Reverse V	Well Plumes		
⁹⁰ Sr - Run 1	3,937	1,785	2,283,725 (603,362)	4.915 x 10 ⁹	1.41 x 10 ¹⁴ pCi/g	3.49 x 10 ⁻⁵
⁹⁰ Sr - Run 2	4,896	980	1,448.406 (382,670)	5.672 x 10 ⁹	e e a a	4.02 x 10 ⁻⁵
⁹⁰ Sr - Total	N/A	N/A	3,732,131 (986,032)	1.059 x 10 ¹⁰	99 04	7.51 x 10 ⁻⁵
¹³⁷ Cs - Run 1	1,354	59.2 (OS=8)	2,283,725 (603,362)	2.957 x 10 ⁹ (3.074 x 10 ⁹)	8.7 x 10 ¹³ pCi/g	3.40 x 10 ⁻⁵ (3.53 x 10 ⁻⁵)
¹³⁷ Cs - Run 2	1,457	59.1 (OS=5.75)	1,448.406 (382,670)	2.025 x 10 ⁹ (2.102 x 10 ⁹)	ин оэ	2.33 x 10 ⁻⁵ (2.42 x 10 ⁻⁵)
137Cs - Total	N/A	N/A	3,732,131 (986,032)	4.982 x 10 ⁹ (5.176 x 10 ⁹)	н п п п	5.73 x 10 ⁻⁵ (5.95 x 10 ⁻⁵)
^{239/240} Pu - Run 1	19.4	5.6 (OS=1.55)	2,283,725 (603,362)	3.153 x 10 ⁷ (4.065 x 10 ⁷)	8.86 x 10 ¹⁰ pCi/g	3.56 x 10 ⁻⁴ (4.59 x 10 ⁻⁴)
^{239/240} Pu (extrapolated for total run)	19.4	5.6 (OS=1.55)	3,732,131 (986,032)	5.150 x 10 ⁷ (6.662 x 10 ⁷)	97 N R	5.81 x 10-4 (7.52 x 10 ⁻⁴)
	1	2	216-BY Cribs	Plume	, <u>, , , , , , , , , , , , , , , , , , </u>	
⁹⁹ Tc	9,007	169.5	1,426.037 (376,760)	1.260 x 10 ¹⁰	1.7 x 10 ¹⁰ pCi/g	0.74 g
⁶⁰ Со	156	39.5 (OS=10.5)	1,426.037 (376,760)	1.661 x 10 ⁸ (2.075 x 10 ⁸)	1.130 x 10 ¹⁵ pCi/g	1.47 x 10 ⁻⁷ (1.84 x 10 ⁻⁷)

NOTES:

- 1. All data calculated from Appendix C tables.
- 2. OS = X is offsite laboratory analytical results. Bracketed calculations for total curies and total grams are based on using offsite analyses when 222-S Laboratory results were reported as less-than detection values.

Table 3-2. Calculated K Values for Strontium-90.

n In	First-Order K Values			
Run ID	First Bed	Second Bed		
Pilot Run 1	0.138	0.135		
1MCT-1	0.268	0.263		
1MCT-2	0.20	0.18		
Pilot Run 2	0.079	0.12		
2MCT-2	0.126	0.19		
2MCT-3	0.136	0.20		
2MCT-1 (loaded)	0.063	0.062		

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Table 3-3. System Removal Performance, Quantities of Radionuclides.

Contaminant	Average Concentration of Well Influent (pCi/L) Average Concentration of System Effluent (pCi/L)		Liters (gal) treated	Total pico- Curies Removed	Specific Activity	Total Grams Removed
		216-E	8-5 Reverse V	Vell Plumes		
⁹⁰ Sr - Run 1	3,937	936	2,283,725 (603,362)	6.853 x 10°	1.41 x 10 ¹⁴ pCi/g	4.86 x 10 ⁻⁵
⁹⁰ Sr - Run 2	4,896	267	1,448,406 (382,670)	6.705 x 10°	ч н и н	4.76 x 10 ⁻⁵
⁹⁰ Sr - Total	N/A	N/A	3,732,131 (986,032)	1.356 x 10 ¹⁰	0 9 11 11	9.62 x 10 ⁻⁵
¹³⁷ Cs - Run 1	1,354	55.6 (OS=8)	2,283,725 (603,362)	2.965 x 10 ⁹ (3.074 X 10 ⁹)	8.7 x 10 ¹³ pCi/g	3.41×10^{-5} (3.53 x 10^{-5})
¹³⁷ Cs - Run 2	1,457	58.5 (OS=5.75)	2,283,725 (382,670)	2.026 x 10 ⁹ (2.102 x 10 ⁹)	n # D n	2.33 x 10 ⁻⁵ (2.42 x 10 ⁻⁵)
¹³⁷ Cs - Total	N/A	N/A	3,732,131 (986,032)	4.991 x 10 ⁹ (5.176 x 10 ⁹)	D 31 EL JI	5.74 x 10 ⁻⁵ (5.95 x 10 ⁻⁵)
^{239/240} Pu - Run I	19.4	5.5 (OS=1.55)	2,283,725 (603,362)	3.174 x 10 ⁷ (4.065 x 10 ⁷)	8.86 x 10 ¹⁰ pCi/g	3.58 x 10 ⁻⁴ (4.59 x 10 ⁻⁴)
239/240Pu (extrapolated for total run)	19.4	5.6 (OS=1.55)	3,732,131 (986,032)	5.188 x 10 ⁷ (6.662 x 10 ⁷)	ध संस	5.86 x 10-4 (7.52 x 10 ⁻⁴)
	 	2	216-BY Cribs	Plume		<u> </u>
% Тс	9,007	164	1,426.037 (376,760)	1.262 x 10 ¹⁰	1.7 x 10 ¹⁰ pCi/g	0.74 g
∞Со	156	38 (OS=10.5)	1,426.037 (376,760)	1.683 x 10 ⁸ (2.075 x 10 ⁸)	1.130 x 10 ¹⁵ pCi/g	1.49 x 10 ⁻⁷ (1.84 x 10 ⁻⁷)

NOTES:

- 1. All data calculated from Appendix C tables.
- 2. OS = X is offsite laboratory analytical results. Bracketed calculations for total curies and total grams are based on using offsite analyses when 222-S Laboratory results were reported as less-than detection values.

Table 3-4. Cost/Resource Requirements. (3 Sheets)

Resources	Dollars				
Design Engineering and Systems Fabric					
Design Engineering					
Exempt	367.9				
Nonexempt	31.7				
Bargaining Unit	0.1				
Materials	341.2				
Other ^b	227.8				
Indirect ^c	527.0				
Subtotal	1,495.7				
Systems Fabrication/Field Installation					
Exempt	3.8				
Craft	17.7				
Subcontractors	458.0 ^d				
Indirects	8.6				
Subtotal	488.10				
Total	1,983.8				
216-B-5 Reverse	Well Operations				
Exempt	235.6 (115.8) ^e				
Nonexempt	24.8 (8.0) ^e				
Craft	16.4 (16.4) ^c				
Materials/Equipment	36.3				
Sample Analysis	158.9 (145.9) ^f				
Other ^b	200.6				
Indirect	232.7				
Total	905.3				
216-BY-Crib	os Operations				
Exempt	178.8 (57.3) ^e				
Nonexempt	60.3 (45.1) ^e				
Craft	4.0 (4.0) ^e				
Materials	44.0				
Analytical	99.5 (84.5) ^f				
Other	98.8				
Indirect	195.6				
Total	681.0				
216-B-5 Reverse We	ll Mini-Column Tests				
Exempt	36.3				
Materials	16.4				
Sample Analysis	199.9 (198.3) ^f				
Other	7.8				
Indirect	43.7				
Total	304.1				

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Table 3-4. Cost/Resource Requirements. (3 Sheets)

Resources	Dollars
216-BY-Cribs Mi	
Exempt	5.9
Materials	1.0
Sample Analysis	10.1 (5.1) ^f
Indirect	5.3
Total	22.3
Project	Support
Exempt	265.7
Nonexempt	24.4
Craft	3.6
Materials	0.3
Other	19.1
Indirect	190.0
Total	503.1
Project Management	(FY 1994 and 1995)
Exempt	224.6
Nonexempt	0.8
Materials	23.8
Other	32.0
Indirect	283.3
Total	564.5
Waste Ma	inagement
Exempt	3.6
Nonexempt	1.1
Other	22.2
Indirect	7.2
Total	34.1
Winterization Activities at 216-B-	5 Reverse Well and 216-BY-Cribs
Exempt	10.2
Craft	57.3
Materials	36.7
Indirect	23.6
Total	127.8
Tank Cleanout at 216-B-5 Re	verse Well and 216-BY-Cribs
Exempt	15.2
Craft	66.1
Other	1.4
Sample Analysis	25.1
Indirect	10.8
Total	118.6

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Table 3-4. Cost/Resource Requirements. (3 Sheets)

Resources	Dollars					
Geosciences						
Labor	37.4					
Materials	9.1					
Sample Analysis	28.6					
Other	38.3					
Indirect	24.7					
Tot	al 138.1					
Test Report (to	date - end of July)					
Exempt	100.6					
Indirect	72.4					
Tot	173.0					
Subtot	al 3571.5					
Grand Tot	al 5555.3					

^aDesign Engineering costs are undifferentiated for the two systems. Design Engineering and Systems Fabrication/Field Installation are FY 1994 activities only.

bOther: Costs reported here vary between tasks, but include Purchase Order contracts, subcontractor Westinghouse Hanford Company, subcontractor analytical (when not reported separately), travel and living, heavy equipment, office supplies, miscellaneous purchased services taxed, shop and laboratory supplies, Hanford Site telephone, U.S. postage, etc.

Indirects include Bechtel Hanford, Inc. (BHI) award fee, BHI direct distributable, BHI general distributable, BHI operating centers, BHI General and Administrative, and organizational overhead.

^dSubcontractor includes labor, materials, equipment, etc., used for site preparation and field installation of both treatment systems.

^eNumbers in parentheses are labor dollars during months of operation.

'Numbers in parenthesis are 222-S Laboratory dollars.

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Table 3-5. N-Springs Groundwater Test Conditions and Breakthrough of Strontium-85 in Clinoptilolite Adsorbent.

Empty Bed Residence Time (min)	Volume	Pore Volume (mL)	rate	Pore Space Residence Time (min)	DV at 0.7%		Breakthrough at End of Run
13.4	6.89	3.68	4.35	7.2	540	860	40% at 4,700 BV
27	10.12	5.38	2.17	14.5	970	1,400	15% at 2,400 BV

NOTE: 0.7% breakthrough = approximately 8 pCi/L and 3.5% breakthrough = approximately 41 pCi/L.

Sources:

Mattigod, S. V., R. J. Serne, T. L. Gervais, D. S. D. Burke, and N. Neerchal, 1995, Remediation of ⁹⁰Sr Contaminated Groundwater from 100 N Area Using Clinoptilolite Ion Exchange Columns: Data from Bench Scale Tests, In Preparation, Pacific Northwest National Laboratory, Richland, Washington.

BV = bed volume

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4.0 GROUNDWATER MONITORING AND ASSESSMENT ACTIVITIES

This chapter describes hydrologic and contaminant impacts associated with extracting and injecting groundwater at the B-5 Reverse Well and the BY-Cribs. Well suitability test data are presented in Section 4.1. Well suitability data are used in conjunction with baseline water quality data to determine well yield, injection capacity, and evaluate well locations as points of extraction/injection. Water-level monitoring efforts to assess hydrologic impacts are presented in Section 4.2. Section 4.3 describes baseline water quality and the contaminant response to pumping at the extraction well. Because of scheduling constraints, post-treatability groundwater samples were not collected. In situ groundwater flow velocity and other assessment activities (e.g., sonic cone penetrometer test) are discussed in Section 4.4.

4.1 WELL SUITABILITY INVESTIGATION

Groundwater monitoring wells near the planned pumping and injection centers and those wells proposed as extraction/injection wells were evaluated to assess fitness-for-use before treatability testing. Tasks performed to assess fitness consisted of borehole camera surveys, well scrubbing and cleaning, and withdrawal and injection tests. The withdrawal/injection tests were performed to determine well yield and injection capacity. Selected wells were redeveloped or deepened (e.g., cement plug drilled out/drilled deeper) to maximize water production during testing. Data collected during well suitability testing and baseline groundwater sampling (Sections 4.3.1 and 4.3.5) were used to select potential extraction and injection wells. Well suitability tests were performed in wells 299-E28-1, 299-E28-2, 299-E28-23, 299-E28-24, and 299-E28-25 at the B-5 Reverse Well site. Wells 299-E28-23 and 299-E28-25 were also sampled during suitability tests to assess changes in contaminant concentrations over time. Well suitability tests were performed in wells 699-49-55A, 699-49-57A, and 699-50-53A at the BY-Cribs site. Table 4-1 summarizes activities performed in each well and gives production data.

Wells 299-E28-23 and 299-E28-7 were selected as the extraction and injection well, respectively, at the B-5 Reverse Well site. Well 299-E28-23 was selected as the extraction well because it is located in the most contaminated section of the plume and the well yield was sufficient to support designed pumping rates. Well 299-E28-7 was selected as the site of injection because it is located within the plume and because minor changes in head were observed with respect to gallons injected. The injection well was later changed to well 299-E28-25 to enhance tracer testing (see discussion in Section 4.4.4). Wells 299-E28-1 and 299-E28-2 were not used as extraction or injection centers because they are outside of the plume. Well 299-E28-24 was not used as an extraction well because of high turbidity and low yield.

Samples collected over time during suitability testing in wells 299-E28-23 and 299-E28-25 showed contaminant concentrations declining 32% to 65% after approximately 2.5 h of pumping. The concentration of 137 Cs decreased by 32% in the these wells over the same period of time. The reduction in the concentrations of $^{239/240}$ Pu and 90 Sr varied in these wells, but was greater than 45%. The groundwater data are presented in Table 4-2.

Wells 699-50-53A and 699-49-55A were selected as extraction and injection wells, respectively, at BY-Cribs. Well 299-50-53A was chosen because it is located in the most contaminated section of the plume. Well 699-49-55A was selected as the site of injection because it is upgradient of the

extraction well, located within the plume, and the closest available well. Well 699-49-57A was not used for extraction or injection because it is too distant from other wells evaluated. None of the wells evaluated at the BY-Cribs produced appreciable amounts of groundwater during pumping.

4.2 WATER-LEVEL MONITORING IN SUPPORT OF TREATABILITY TESTING

This section describes groundwater conditions before and during treatability testing. Data are not yet available to determine groundwater conditions after completion of the test.

4.2.1 Pre-Treatability Test Water-Level Monitoring at the 216-B-5 Reverse Well Site

Groundwater monitoring wells in the vicinity of the B-5 Reverse Well are located in an area where the hydraulic gradient is very small, hydraulic conductivities are high, and flow direction is controlled by the regional gradient from the west and mounding from the B Pond to the east. Based on the available data, the apparent flow direction at the site was to the southeast about 2 weeks before the start of treatability testing (Figure 4-1). The level of confidence placed in data used to create the groundwater map is low as the potential error using standard measuring techniques (e.g., surveying, depth to water measurements, verticality of the borehole) may be large in contrast to the gradient. However, the groundwater flow direction as depicted is not unreasonable based on the regional flow in the vicinity of the 200 East Area. Pre-treatability test water-level measurements collected on August 16, 1994, were used to estimate groundwater flow direction.

Water-level data also indicate that the elevation of the water table is currently declining at a rate of about 1.8 m/yr (0.6 ft/yr). The decline in the elevation of the water table is the result of the reduction in effluent discharge to the sediment column over the last decade, primarily at B Pond.

4.2.2 Water-Level Monitoring During Operation at the 216-B-5 Reverse Well Site

Water levels were monitored at the extraction well (299-E28-23), two injection wells (299-E28-7 and 299-E28-25), and an observation well (299-E28-24) during treatability testing. Well 299-E28-7 was used as the return well until April 7, 1995. Well 299-E28-25 was used as the injection well after that date to support tracer testing.

Water levels in the extraction and observation wells showed no response to pump-and-treat operations. Observed water-level fluctuations correspond mainly to barometric pressure changes. The daily average pumping rate at the extraction well was approximately 102 L/min (27 gal/min) during operations. Maximum sustained yield during operation could not be determined because of the lack of drawdown.

Groundwater was injected into well 299-E28-7 at an average rate of 132 L/min (35 gal/min). This resulted in an average head increase of 0.15 m (0.5 ft) [a maximum of 0.6 m (2 ft was observed]. The average injection capacity is 80.8 L/min/m (70 gal/min/ft), and the average rate of injection in well 299-E28-25 was 178 L/min (47 gal/min). The average head is 0.15 m (0.5 ft) [maximum of

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0.9 m (3 ft)]. The average injection well capacity is 108.5 L/min/m (94 gal/min/ft). Graphs of the complete water-level data set are presented in Appendix D.

During the two tracer tests at the 216-B-5 site, pumping at approximately 106 L/min (28 gal/min) occurred continuously for more than 120 h. Water level monitoring data collected during the tests indicated 0.6 to 1.3 cm (0.25 to 0.5 in.) of drawdown in the extraction well. The uncertainty in hydraulic gradient and aquifer hydraulic properties is much greater in magnitude. Further, top-of-casing surveys include an allowable error greater than 0.01 m, which could corrupt any inference about the cone of depression made from the data. Describing the effects of the extraction well on the aquifer with such little drawdown requires much more precision in aquifer characterization and absolute water level measurement techniques than currently exist.

4.2.3 Post-Treatability Test Water-Level Data at the 216-B-5 Reverse Well Site

Water-level data were not collected to assess groundwater flow direction after treatability testing.

4.2.4 Pre-Treatability Test Water-Level Monitoring at the 216-BY Cribs

Groundwater monitoring wells in the vicinity of the BY-Cribs are located north of the B-5 Reverse Well and are controlled by the same regional flow regime described in Section 4.2.1. The water-table map suggests that groundwater flow direction beneath the site is predominantly to the northwest and is controlled more by mounding from B Pond than by regional gradient. The water table in the vicinity of the BY-Cribs site is shown in Figure 4-2. Pre-treatability test water-level data indicate that the elevation of the water table is declining at a rate of about 0.18 m/yr (0.6 ft/yr). Pre-treatability test water-level measurements collected during June 1994 were used to estimate groundwater flow direction at the BY-Cribs.

4.2.5 Water-Level Monitoring During Operation at the 216-BY Cribs

During treatability testing water levels were monitored at extraction well 699-50-53A, injection well 699-49-55A, and two nearby deeper confined aquifer wells (699-50-53B and 699-49-55B) that penetrated the Elephant Mountain basalt into the Rattlesnake Ridge interbed. The extraction and injection wells were monitored to assess optimum rate of extraction/injection as other unconfined wells in the network were not in the area influenced by the test. Wells monitoring the confined aquifer were evaluated to assess communication with the unconfined aquifer.

Water levels in confined aquifer wells showed no response to treatability test activities. For all wells monitored, water-level fluctuations corresponded mainly to barometric pressure changes. The extraction well sustained an average drawdown of 2.21 m (7.26 ft) at an average pumping rate of 13.2 L/min (3.5 gal/min). The average well capacity at the extraction well is 0.6 L/min/m (0.5 gal/min/ft). The injection well sustained an average injection rate of 95 L/min (25 gal/min) with an average head increase of 2.9 m (9.5 ft) [maximum of 3 m (11 ft)]. The average injection capacity in the injection well is 3.0 L/min/m (2.6 gal/min/ft).

At the BY-Cribs site, drawdown in the extraction well exceeded the thickness of the aquifer. The resulting water level was below the Ringold/Hanford contact. No observation wells were available to

provide additional drawdown data. The cone of depression would have to be based on the drawdown data collected from the extraction well. Interpretation of the data becomes ambiguous because of the uncertainty of the source of the water in the well. The drawdown in the extraction well is not interpretable with regard to areal influence because the cone of depression at the extraction well exceeds the aquifer thickness.

4.2.6 Post-Treatability Test Water-Level Monitoring at 216-BY Cribs

Water-level data were not collected to assess groundwater flow direction after treatability testing.

4.3 CONTAMINANT CONCENTRATION RESPONSE

This section describes baseline water quality and the contaminant response to pumping at the extraction well. Data are not available to determine post-treatability test water quality at the B-5 Reverse Well and the BY-Cribs sites.

4.3.1 Baseline Groundwater Quality in the 216-B-5 Reverse Well Plume

- **4.3.1.1** Cesium-137. Cesium-137 was detected in three monitoring wells before treatability testing. A maximum activity of 2,550 pCi/L was detected in extraction well 299-E28-23 in June 1994. Cesium-137 was also detected in wells 299-E28-7 and 299-E28-25 ranging between 15.3 and 280 pCi/L. Cesium-137 was not detected in well 299-E28-1. The distribution of ¹³⁷Cs before treatability testing is shown in Figure 4-3. Baseline groundwater data are presented in Table 4-2.
- **4.3.1.2 Plutonium-239/240.** Plutonium-239/240 was detected in two wells at the B-5 Reverse Well site before treatability testing in excess of the MCL (1.2 pCi/L). A maximum activity of 25.7 pCi/L was detected in well 299-E28-23 in March 1994. The level of contamination in well 299-E28-25 did not exceed 17.5 pCi/L. Plutonium-239/240 was not detected in wells 299-E28-1 and 299-E28-7. The distribution of ^{239/240}Pu before treatability testing is shown in Figure 4-3. Baseline groundwater data are presented in Table 4-2.
- **4.3.1.3** Strontium-90. Elevated levels of ⁹⁰Sr in excess of the MCL (8 pCi/L) were detected in wells 299-E28-7, 299-E28-23, and 299-E28-25. The maximum activity exceeded 11,400 pCi/L in well 299-E28-23 during the June 1994 sampling event. Concentrations in wells 299-E28-7 and 299-E28-25 range between 160 and 4,730 pCi/L. Strontium-90 activity did not exceed 0.5 pCi/L in well 299-E28-1. The distribution of strontium-90 before treatability testing is shown in Figure 4-3. Baseline groundwater data are presented in Table 4-2.

4.3.2 Contaminant Response to Pumping at Extraction Well 299-E28-23

Groundwater samples were collected daily during Run 1 at the B-5 Reverse Well site to assess the contaminant response to pumping. Although sampling and testing started on August 29, 1994, effective operations of the system did not begin until January 17, 1995. Run 1 ended on March 17, 1995, concluding a 2-month cycle of pumping groundwater an average of 8-h/day, 5 days/week (i.e., weekly operation). Starting on February 6, 1995, the extraction well was pumped

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continuously for 5 days and then shut off. The average pumping rate during all stages of testing in Run 1 was about 102 L/min (27 gal/min) and resulted in total volume of 2,283,967 L (603,362 gal) of groundwater being extracted. During Run 1, groundwater was injected into well 299-E28-7, located about 17 m (57 ft) southeast of the extraction well.

Run 2 started on April 10, 1995 (about 3 weeks after Run 1 was completed). Groundwater samples were collected every other day. Well 299-E28-25 located about 7 m (24 ft) northwest of the extraction well was used as the injection well. Run 2 continued for 6 weeks at an average pumping rate of 100 L/min (26.4 gal/min). The average pumping cycle was 8 h/day, 5 days/week (i.e., weekly operation). The rate of extraction was increased to 105 L/min (27.8 gal/min) on May 21, 1995, and the well was continuously pumped for 5.5 days. Samples were collected every day during continuous pumping. During weekly operation, 700,299 L (185,000 gal) was pumped compared to 660,583 L (174,508 gal) during continuous pumping in Run 2.

Accounting for processing during startup activities and other operating periods, a total of 37,325,255 L (986,032 gal) was processed through the B-5 Reverse Well treatment system (see Section 3.1.1.1). Using porosities of 0.22 and 0.30 estimated from the tracer tests (see Section 4.4.4), the calculated ratios of capture over the duration of the B-5 Reverse Well treatability test ranged from 15 to 20 m (50 to 65 ft).

During Runs 1 and 2, the concentration of ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr ultimately decline by the end of each run. Figures 4-4, 4-5, and 4-6 show the variability and overall reduction in the concentration of ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr, respectively, in the groundwater. The average concentration during each stage of pumping (e.g., weekly versus continuous) in Runs 1 and 2 are summarized and ranked in Table 4-3 to show the contaminant response to groundwater extraction. Discussions of the contaminant response to pumping at the extraction well are presented in the following sections.

4.3.2.1 Cesium-137. The concentration of ¹³⁷Cs was 1,470 pCi/L in the initial groundwater sample collected during Run 1. This concentration can be compared to an average baseline concentration of ¹³⁷Cs (2,265 pCi/L) in the groundwater to assess the initial contaminant response to pumping, which indicates that ¹³⁷Cs contamination was reduced approximately 35%. A similar response (32% reduction) was also observed during well suitability testing.

The concentration of ¹³⁷Cs ranged between 1,170 and 1,870 pCi/L during weekly operations in Run 1 [pumping 107 L/min (26.7 gal/min), generally 8 h/day, 5 days/week], and a gradual downward trend in the level of contamination was observed. An average concentration of 1,450 pCi/L was detected in the groundwater during the first half of Run 1 compared to an average of 1,252 pCi/L during the second half of Run 1. This indicates that the contaminant response to pumping was greater at the beginning of Run 1. At the end of weekly operations, the concentration of ¹³⁷Cs was 1,210 pCi/L. This concentration is 47% below baseline and 18% below the initial concentration of 1,470 pCi/L in Run 1. An average concentration of 1,360 pCi/L was detected in the groundwater during weekly operations. The data also suggest that the level of ¹³⁷Cs contamination occasionally increased 10% over weekends when no pumping occurred.

Slightly more than a 10% decline in the level of 137 Cs contamination was observed during continuous pumping (28.3 gal/min for 5 days) in Run 1. Contamination ranged between 1,350 and 1,570 pCi/L during continuous pumping, and the average concentration of 137 Cs in the groundwater was 1,425 pCi/L.

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Run 2 began after a 3-week period without extracting groundwater. Groundwater samples collected during Run 2 indicated that ¹³⁷Cs contamination increased by 12% to 1,380 pCi/L at the start of the test, and an upward trend in the concentration of ¹³⁷Cs was observed. A general pattern of concentrations decreasing and increasing during periods of extraction and no pumping, respectively, were also observed. The 3-week recovery period accounts for the initial rise in ¹³⁷Cs at the start of Run 2. Changing the site of groundwater injection to well 299-E28-25 is likely the major factor governing the upward trend observed during pumping. The pumping rate probably had a negligible effect on the trend as the pumping rate approximates the average extraction rate in Run 1. The average concentration of ¹³⁷Cs in the groundwater during weekly operations was 1,590 pCi/L in Run 2.

During Run 2, groundwater was extracted continuously at a rate of 105 L/min (27.8 gal/min) for 5 days. A downward trend in the level of contamination was observed during this period. The initial and final concentration of ¹³⁷Cs was 1,820 and 1,310 pCi/L, respectively, and represents a 28% decline in the concentration of ¹³⁷Cs. An average concentration of 1,490 pCi/L was detected in the groundwater during continuous pumping in Run 2.

4.3.2.2 Plutonium-239/240. The concentration of ^{239/240}Pu was 12 pCi/L in the initial groundwater sample collected during Run 1. This concentration is 48% below the baseline level of 23 pCi/L established before the test.

The concentration of ^{239/240}Pu ranged between 12 and 44 pCi/L during weekly operations in Run 1. Contaminant levels near the beginning of the test increased to a maximum of 44 pCi/L, indicating that higher concentrations of ^{239/240}Pu were made available by extracting groundwater. The higher concentrations observed near the beginning of the test resulted in the greater contaminant mass (average 24 pCi/L) being removed in the first part of Run 1 compared to an average of 17 pCi/L during the later half of the test. The final concentration of ^{239/240}Pu in Run 1 was 13.7 pCi/L and within 1.7 pCi/L of the initial concentration (12 pCi/L) at the start of the test. If the initial concentration of 12 pCi/L is not considered at the start of Run 1, a downward decline in the level of contamination is observed over the length of the test. The average concentration in the groundwater during weekly pumping in Run 1 is approximately 19 pCi/L.

The concentration of ^{239/240}Pu ranged between 15.6 and 17.5 pCi/L during continuous pumping [107 L/min (28.3 gal/min)] in Run 1. A downward trend in the level of ^{239/240}Pu concentration was observed, and the average concentration in the groundwater was about 16.5 pCi/L. The average groundwater concentration during continuous pumping compares similarly to the average groundwater concentration (17 pCi/L) during the later half of weekly operations in Run 1.

The concentration of ^{239/240}Pu in the groundwater during Run 2 increased by 31% to 20 pCi/L 3 weeks after Run 1 was completed. As described regarding ¹³⁷Cs concentrations in Run 2, the 3-week recovery period and change in injection site form the basis for the higher contaminant levels. The concentration of ^{239/240}Pu ranged between 15.7 and 24.1 pCi/L during routine pumping [107 L/min (28.3 gal/min)] in Run 2. The average concentration during weekly pumping in Run 2 was about 20 pCi/L.

During the last stage of Run 2, groundwater was extracted continuously at a rate of 105 L/min (27.8 gal/min) for 5 days. The concentration of ^{239/240}Pu ranged between 19.9 and 21.1 pCi/L during continuous pumping in Run 2, and a slight downward trend in the level of contamination was

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observed. The decline in contaminant concentration was about 5%. The average concentration during pumping was 20 pCi/L.

4.3.2.3 Strontium-90. The concentration of ⁹⁰Sr was 4,560 pCi/L in the initial groundwater sample collected during Run 1. This concentration is 52% below the baseline level of 9,850 pCi/L established before the test. A similar response (greater than a 45% reduction in the concentration of ⁹⁰Sr) was also observed during well suitability testing.

The concentration of 90 Sr ranged between 3,230 and 4,570 pCi/L, and a gradual, decline in the level of contamination was observed in Run 1 during weekly pumping. An average concentration of 3,856 pCi/L was detected in the groundwater during the first phase of Run 1 compared to an average of 3,490 pCi/L during the later half of Run 1. This indicates that the contaminant response to pumping was greater in the beginning of Run 1. The average concentration of 90 Sr in the groundwater during weekly pumping was approximately 3,663 pCi/L in Run 1. In addition, at the end of weekly operations the concentration of 90 Sr was 3,520 pCi/L, which is 65% below baseline and 23% below the initial concentration of 4,560 pCi/L in Run 1.

The concentration of ⁹⁰Sr ranged between 3,400 and 4,500 pCi/L during continuous pumping, and a downward decline in the level of contamination was observed. The average concentration during continuous pumping was 3,894 pCi/L. This average concentration approximates average contaminant levels observed in the groundwater during the first half of weekly pumping in Run 1.

The concentration of ⁹⁰Sr in the groundwater during Run 2 increased by 23% to 4,610 pCi/L 3 weeks after Run 1 was completed. As described regarding ¹³⁷Cs and ^{239/240}Pu concentrations in Run 2, the 3-week recovery period and change in injection site form the basis for the higher contaminant levels. A general increase in the concentration of ⁹⁰Sr was observed in Run 2. However, a general pattern of concentrations declining during extraction and increasing during periods of no pumping were also observed. The average rate of removal during Run 2 was 4,970 pCi/L.

Continuous pumping at a rate of 95 L/min (25 gal/min) at the end of Run 2 resulted in a decline in the concentration of 90 Sr from 6,410 to 3,600 pCi/L. The average concentration in the groundwater was 4,488 pCi/L.

4.3.3 Review of Data

As indicated in Sections 1.3.2 and 4.2 of the Pilot-Scale Treatability Test Plan for the 200-BP-5 Operable Unit (DOE-RL 1995), radionuclide concentrations in the groundwater were expected to rapidly decrease following the start of extraction. The conceptual model assumed that the contaminants dissolved in the groundwater would be depleted after several days to weeks of pumping. This was suggested by the limited migration of the plumes in the last 50 years and the generally high K_d values of the contaminants. Specifically, once the contaminated groundwater in the aquifer porespaces had been extracted, contamination concentrations would decline to very low levels. The rate of re-equilibration between sediments contaminated with the radionuclides and "fresh" groundwater would proceed very slowly and would be further limited by continued groundwater pumping. Thus, at some stage of operations, contaminant concentrations would drop and the corresponding process plant extraction efficiency would decline to a very low level. At this point, overall test efficiency would benefit from a halt to allow the aquifer contaminant concentrations to reequilibrate. The concern over this possibility was such that the test plan was modified to require an

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agreement between DOE, EPA and Ecology to approve shutdown and restart of operations. Thus, the consistent concentration of contaminated groundwater was unexpected and prompted an examination of extraction data.

Baseline groundwater data and groundwater data collected in Run 1 were compared to assess the initial contaminant response to pumping. The data indicate that the contaminant response to pumping was substantial during the initial stage of pumping. The initial decline in contaminant concentrations for 137 Cs, $^{239/240}$ Pu, and 90 Sr were 35%, 48%, and 52% below baseline, respectively. Comparing these initial changes in the level of contamination to published K_d values would suggest that $^{239/240}$ Pu ($K_d = 2,000 \text{ mL/g}$, very highly sorbed) and 137 Cs ($K_d = 500 \text{ mL/g}$ to mL/g, highly sorbed) concentrations should respond at a slower rate than anticipated to the effects of further pumping. Only the concentration of 90 Sr ($K_d = 20 \text{ mL/g}$, moderately sorbed) was expected to change significantly during pumping. However, data indicate that once the concentrations declined from baseline values, there was a fairly constant range of concentrations for any run.

At least two possible explanations for the differences in the contaminant response to pumping versus the assumed response, as based on reported K_d values can be offered:

- The contaminants' chemical properties (i.e., sorption rate) may be different than the published K_d values indicate. The waste stream was injected directly into the groundwater and may still be influenced by the waste stream chemistry. Therefore, the contaminants could be less strongly bonded to the sediment and would be more mobile.
- There may be greater quantities of the contaminants in the aquifer system than calculated from historical process knowledge. Process knowledge quantities were calculated from sample analyses taken from individual batches sent to Tank 241-B-361. Sampling or laboratory measurement inaccuracies may have under-reported the quantities of waste actually going to the ground. Therefore, a greater quantity of the contaminants may be present in the groundwater.

Additional direct evidence is not available to support either alternative. New K_d values cannot be calculated from available test data, and new values cannot be assumed. Circumstantial evidence for larger quantities of waste reaching the ground is presented in Section 5.3.1 and was taken from Smith (1980). Evidence for different K_d values may be inferred in the discussion of the B-5 site's phosphate-based geochemistry in Section 5.3.1.

Trends outlined in Section 4.3 suggest that the best operational setting for optimum contaminant removal is the weekly pumping cycle in the configuration of the extraction and injection wells in Run 2. The data also indicate that a recovery period between pumping cycles allows equilibration between the groundwater and contaminants in the aquifer system. The recovery period results in greater contaminant concentrations being available for extraction, which, in turn, enhances removal of the radionuclides mass from the plume. Enhanced contaminant extraction through the use of reactants to remobilize the plume.

The addition into the aquifer of chemical agents to increase the solubility of the contaminants would result in increased contaminant extraction rates and increased process efficiency.

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4.3.4 Post-Treatability Test Water Quality

Post-treatability test groundwater data were not collected for this investigation.

4.3.5 Baseline Groundwater Quality at the 216-BY Cribs Plume

- **4.3.5.1** Cobalt-60. Cobalt-60 was detected at a maximum activity of 74.3 pCi/L in well 699-52-54. The contaminant was also detected in wells 299-E33-7, 699-49-55A, 699-50-53A, and 699-55-57, ranging between 6.0 pCi/L and 50.4, as well as in three cone penetrometer samples from pushes 49-53, 49-54, and 50-54. Cobalt-60 ranged between 9.7 pCi/L and 20.1 pCi/L in cone penetrometer samples. All samples analyzed for ⁶⁰Co were below the MCL of 100 pCi/L. The distribution of ⁶⁰Co before treatability testing is shown in Figure 4-7. Baseline groundwater data are presented in Table 4-4.
- **4.3.5.2 Technetium-99.** Technetium-99 was detected in all groundwater samples. The concentration of ⁹⁹Tc ranges between 2.47 and 4,310 pCi/L, with the maximum activity occurring in well 699-52-54 during the April 1994 sampling event. The MCL of 900 pCi/L was exceeded in wells 299-E33-13, 699-49-57A, 699-50-53A, 699-52-54, and 699-55-57, as well as sonic push 50-54. Technetium-99 in all other groundwater samples collected ranges between 2.47 and 775 pCi/L. The distribution of ⁹⁹Tc before treatability testing is shown in Figure 4-7. Baseline groundwater data are presented in Table 4-4.
- 4.3.5.3 Contaminant Response to Pumping at Extraction Well 699-50-53A. Groundwater samples were collected during operations at the BY-Cribs site to assess the contaminant response to pumping. Although sampling and testing started on August 29, 1995, effective operations of the system did not begin until January 18, 1995. The treatability test continued for 4 months (until May 22, 1995). Groundwater samples were collected from a sample port located before the treatment system prefilters. Samples were analyzed for 60Co and 99Tc at the 222-S Laboratory. Confirmatory samples were also occasionally collected and shipped to an offsite laboratory for analyses.

Groundwater was extracted from the aquifer at an average rate of approximately 13.2 L/min (3.5 gal/min). The maximum average sustainable pumping rate was approximately 12.9 L/min (3.4 gal/min) and was limited by the saturated thickness [~0.3 m (1 ft)] of the unconfined aquifer. A total volume of 1,426,187 L (376,760 gal) was extracted during testing. Using estimated porosity range of 0.2 to 0.3, the calculated radios of capture over the duration of the treatability test ranged from 70 to 86 m (231 ft to 283 ft).

4.3.5.3.1 Cobalt-60. The concentration of ⁶⁰Co was 63 pCi/L in the initial groundwater sample collected at the BY-Cribs site. This concentration can be compared to the baseline concentration of ⁶⁰Co (28 pCi/L) to assess the initial contaminant response to pumping. The initial contaminant response to pumping indicates that ⁶⁰Co contamination increased by 125%. A gradual increase in the level of contamination was observed in the rest of the treatability test. The final concentration at the end of the test was 177 pCi. This increase in the level of contamination is over 600% above baseline and 280% above the initial concentration at the start of the test. The data indicate that ⁶⁰Co is very loosely bonded within the aquifer system and is readily mobilized during pumping. The average concentration of ⁶⁰Co in the groundwater during pumping was 160 pCi/L. Figure 4-8 shows the variability and overall increase in the concentration of ⁶⁰Co in the groundwater at the BY-Cribs site.

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The published K_d of 60 Co (2,000 mL/g, highly immobile) suggests that the contaminant should not respond significantly to pumping. As discussed in Section 1.3.2, the increased mobility of 60 Co is attributed to complexing with ferrocyanide. A possible explanation for the upward trend in the level of contamination may be inherently related to the length of time groundwater was extracted from the aquifer, as the higher levels of contamination were not observed during routine/baseline sampling. The higher levels of contamination were likely due to the larger area influenced by pumping over the average extraction period (e.g., 8 h/day, 5 days/week).

4.3.5.3.2 Technetium-99. The concentration of ⁹⁹Tc was 3,270 pCi/L in the initial groundwater sample collected at the BY-Cribs site. This concentration is two times greater than the baseline level of 1,585 pCi/L established before the test. A gradual increase in the level of contamination was observed throughout the rest of the test. The final concentration at the end of the test was 6,960 pCi/L. The increase in the level of contamination is over 400% above baseline and 200% above the initial concentration at the start of the test. The average concentration of ⁹⁹Tc in the groundwater during pumping was 8,810 pCi/L, and values above 10,000 pCi/L were frequently observed. Figure 4-9 shows the variability and overall increase in the concentration of ⁹⁹Tc in the groundwater at the BY-Cribs site.

The sample data suggest that 99 Tc is very loosely bonded or entrained within the aquifer system and, therefore, is readily available for extraction. The high mobility of 99 Tc observed is consistent with the published K_d value of 0 mL/g. A possible explanation for the upward trend in the level of contamination is related to the length of time groundwater was extracted from the aquifer as described for 60 Co.

4.3.6 Post-Treatability Test Water Quality

Post-treatability test groundwater data were not collected for this investigation.

4.4 DESCRIPTION OF OTHER HYDROGEOLOGIC ASSESSMENT ACTIVITIES

The activities described in this section were performed to respond to test objectives in (DOE-RL 1995c) specifying greater aquifer characterization. A more detailed list of activities was agreed to in the 200 NPL Agreement/Change Control Form (BHI-00203).

4.4.1 Sonic Push Groundwater Sampling for 216-BY-Cribs Plume Definition

On July 10, 1994, a 200 NPL Agreement/Change Control Form (BHI-00203) was submitted to approve additional aquifer characterization activities in the vicinity of the BY-Cribs plume in the 200-BP-5 Operable Unit. The work was intended to locate areas of greater aquifer thickness and high contaminant concentrations in the area south of planned extraction well 699-50-53A and the northern fenceline of the 200 East Area. To accomplish these objectives, a Water Development Company sonic drill rig was used to push standard 1-m-long, 4.45-cm- (1-3/4-in.) outer diameter, 2.5-cm (1-in.) inner diameter cone penetrometer rods to collect groundwater samples and determine the bottom of the aquifer. Figure 4-10 presents the eight locations proposed for testing.

Table 4-5 summarizes the field work that was performed. In 15 operating days, approximately 546 m (1,790 ft) of hole was pushed in 16 attempts at six of the eight sites. More than 61 m (200 ft) of cone penetrometer rod was abandoned in the ground above the water table because of stress failures of the rod couplings. The sonic push program was terminated because of schedule constraints and the inability of the rods to handle the drilling stresses necessary to penetrate the lower Hanford gravel unit that overlies and comprises the aquifer.

Groundwater samples were successfully collected at three locations shown in Figure 4-11. The data are presented in Section 4.3.5. The figure also shows the three sites where the sonic push rig was unable to penetrate to groundwater. In all cases of refusal, penetration terminated in the lower Hanford gravel unit with all attempts ending within 9 m (30 ft) of the expected water-table elevation.

Holes that reached the water table terminated in a hard layer that refused further penetration. It is assumed that the depth of refusal below the water table corresponds to the bottom of the aquifer (top of the basalt). Table 4-6 presents the depths below ground surface of the water table and the refusal unit. The difference between the two numbers is listed as the assumed thickness of the unconfined aquifer.

4.4.2 Production Testing During Treatability Testing

Preliminary analysis of data collected from treatability test activities indicates that the aquifer and contaminant distribution of the BY-Cribs plume are not adequately defined to support the decision-making process. In lieu of a drilling program, a series of activities was performed as part of the 200 NPL Agreement/Change Control Form (BHI-00203) to support a hydrogeologic assessment of the 200-BP-5 plumes.

4.4.2.1 Production Testing at Well 699-55-57. As part of the 200 NPL Agreement/Change Control Form (BHI-00203), a well fitness-for-use assessment was called out for well 699-55-57. Groundwater samples collected from this well had, over the past several years indicated that the aquifer in this area contained ⁹⁹Tc concentrations at or above the MCL. This assessment activity was intended to determine if the well could be used as a supplementary or alternate extraction well for the BY-Cribs plume treatability test.

Well 699-55-57 was drilled in 1975 to a depth of 55 m (180 ft) bgs. Fifteen-centimeter (six-inch) casing was set at the top of basalt, 52 m (170 ft) bgs, and supposedly perforated by a cutting torch. The bottom 3 m (10 ft) of the well was uncased in the underlying fractured basalt. Earlier borehole camera surveys failed to recognize any perforations in the casing, and an attempt to mechanically perforate the 1.3-cm (0.5-in.) heavy-walled casing failed in 1982. The well was producing approximately 8 L/min (2 gal/min) when it was cleaned and redeveloped as part of the 200-BP-1 Operable Unit site characterization activity in 1990.

In late 1994, the well was cleaned and a second borehole camera survey was conducted. This survey also could not identify any evidence of perforations in the casing. After a measurement of the discharge of the well during a sampling event confirmed that the well was still capable of producing approximately 8 L/min (2 gal/min), a decision was made to perforate the casing using explosive charges in an attempt to stimulate increased well production. The well was jet-shot perforated in the interval 3 m (10 ft) above the top of basalt. A 10-cm.- (4-in.) diameter #20 slot screen was later set

from the top of the perforations to the bottom of the well to control fine formation material that was running into the well through the perforations.

An 8-h pumping test was performed in the well on March 1, 1995, and groundwater samples were collected at 2-h intervals to identify any changes in contaminant concentrations during the test. The results of the sample analyses are found in Table 4-7. An upward trend in the level of contamination was observed as described in Sections 4.3.5.3.1 and 4.3.5.3.2 for ⁶⁰Co and ⁹⁹Tc.

Because of generator/pump fluctuations during the test, a steady pumping rate could not be established. It was observed that a pump rate of about 9.5 L/min (2.5 gal/min) produced a rapid drawdown of the water level and exceeded the productivity of the well.

Because this production rate was not satisfactory for a candidate extraction well, further investigations were terminated.

4.4.2.2 Sampling at Well 699-55-60. The 200 NPL Agreement/Change Control Form (BHI-00203) also called for the expansion of the BY-Cribs monitoring network to aid in evaluating the extent of the plume. One of the activities outlined in that notice was the sampling of wells 699-55-60A and 699-55-60B. Well 699-55-60A was drilled in 1945 as a water supply well to the 200 north facilities and was the nearest well west of the BY-Cribs monitoring network. Well 699-55-60B was also drilled in 1945 as a water supply well for the 200 north facilities. Upon inspection, well 699-55-60B was found to be unusable because of debris in well.

Well 699-55-60A was sampled on March 2, 1995 using a temporary submersible pump. Before sampling, the well was purged at a rate of 8.3 L/min (2.2 gal/min) for 73 minutes. The pH, dissolved oxygen, and temperature were monitored to ensure collection of a representative sample. The results of the sampling found that all three major contaminants of concern were well below the MCL: 26.2 pCi/L of ⁹⁹Tc were detected (MCL = 900 pCi/L); 0.975 pCi/L of ¹³⁷Cs were detected (MCL = 120 pCi/L); and ⁶⁰Co was below detection limit of 7.11 pCi/L. Thus, the data do not suggest any current influence from the BY-Cribs plume in this part of the aquifer.

4.4.3 Groundwater In Situ Flow Survey

To further support the hydrogeologic assessment of the BY-Cribs plume, as directed by the 200 NPL Agreement/Change Control Form (BHI-00203), in-well flow velocity measurements were performed in wells monitoring the plume. The wells, listed in Table 4-8, were all located between the north end of the 200 East Area and the Gable Mountain Gap. Flow velocities were determined by the thermal perturbation method at various levels within each well (see Table 4-8). Figure 4-12 is a well location map.

The groundwater flow direction and magnitude for each well are shown in Figure 4-12. In general, across the study area, the following may be inferred from the flow velocity measurements:

- Groundwater flow direction in the northwest portion of the 200 East Area is to the south, with magnitudes ranging from 0.13 m/day to 0.49 m/day (0.43 ft/day to 1.6 ft/day).
- Groundwater flow direction northeast of the BY-Cribs is to the northwest around a basalt high with a magnitude of 0.6 m/day (1.9 ft/day).

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• Groundwater flow direction north of the 200 East Area and south of West Lake is northerly with a magnitude ranging from less than 0.012 m/day (0.04 ft/day) to 0.34 m/day (1.1 ft/day).

Groundwater velocity measurements less than 0.012 m/day (0.04 ft/day) are considered less than the practical resolution of the flowmeter and necessarily invalidate the flow direction and rate.

The complete analysis and results of the flowmeter survey are discussed in Appendix E.

4.4.4 216-B-5 Reverse Well Tracer Tests

Tracer tests were performed in the B-5 Reverse Well plume to characterize contaminant transport properties of the aquifer and determine the extent of recirculation within the plume resulting from the proximity of the extraction well (299-E28-23) to injection wells. Wells 299-E28-23 and 299-E28-7 were used in the first test as the discharge and recharge well, respectively, and wells 299-E28-23 and 299-E28-25 were used in the second test as the discharge and recharge well, respectively. The state plane coordinates of the wells are east 683,472 m (2,242,361 ft) and north 136,746 m (448,643 ft) for well 299-E28-23, east 683,484 m (2,242,402 ft) and north 136,734 m (448,603 ft) for well 299-E28-7, and east 683,466 m (2,242,344 ft) and north 136,751 m (448,660 ft) for well 299-E28-25. Based on these coordinates, wells 299-E28-23 and 299-E28-7 are located 17.46 m (57.28 ft) apart, and wells 299-E28-23 and 299-E28-25 are located 7.32 m (24.02 ft) apart. Both tracer tests involved 24-h/day extraction and injection at roughly equal flow rates. Tracer sampling of the extraction well started at the onset of slug injection and was performed on an hourly basis throughout the tests. Samples were collected from the injection wellhead at the start of tracer injection and periodically afterwards until the tracer supply was depleted. Three tracer slug samples were collected from test 1, and four samples were collected from test 2. The following sections outline tracer preparation techniques, test start/finish times, flow rates, and volume totals for each tracer test.

- **4.4.4.1 Tracer Test 1.** Before starting tracer test 1, approximately 5 kg of lithium bromide was mixed with water in a drum and pumped into the base of the effluent tank. Tracer mixing within the effluent tank was accomplished by the turbulence generated from the process flow and pumping. Tracer test 1 started on February 6, 1995, at 08:24 with the injection of approximately 21,198 L (5,600 gal) of lithium bromide at an average flow rate of 199.3 L/min (52.5 gal/min). Tracer samples were collected at 08:39, 09:35, and 10:05, and tracer injection finished around 10:25. After charging the effluent tank, effluent injection began at 14:15, and continued until 04:00 hours, February 11, 1995, at an average flow rate of 98 L/min (26 gal/min). Effluent extraction began on February 6, 1995, at 08:02, and continued until 04:00 hours, February 11, 1995, with one interruption: generator servicing occurred on February 7, 1995, from 07:20 to 07:40. The average extraction rate during the test was 107.1 L/min (28.3 gal/min), and the total volume extracted was 507,266 L (134,006 gal). Effluent sampling began on February 6, 1995 at 8:56, and continued hourly until the test ended. Table 4-9 provides a synopsis of the tracer test 1 events.
- **4.4.4.2** Tracer Test 2. Daily background bromide sampling at the extraction well began on May 1, 1995, and continued until the tracer was injected on May 21, 1995, excluding May 6 and 7, May 12 through 14, and May 18 through 20, 1995. On May 19, 1995, 10 kg of potassium bromide was mixed with water and pumped into the effluent tank in the same manner as in tracer test 1. The tracer volume in the tank measured 23,352 L (6,169 gal). Tracer test 2 began on May 21, 1995 at

08:36, with the bromide tracer injected at an average of 1,900.3 L/min (50.2 gal/min). Tracer samples were collected at 08:40, 09:10, 09:40, and 10:10, and tracer injection finished around 10:30. Effluent injection began subsequently and continued until 11:00 hours on May 26,, 1995, at an average flow rate of 83 L/min (22 gal/min). Effluent extraction began on May 21, 1995, at 08:40, and continued until 11:00 hours, May 26, 1995, with one interruption: generator servicing occurred on May 23, 1995, from 07:27 to 07:47. The average extraction rate during the test was 105.6 L/min (27.9 gal/min), and the total volume extracted was 660,583 L (174,508 gal). Effluent sampling began on May 21, 1995, at 8:46, and continued hourly until the test ended. Table 4-10 provides a synopsis of the tracer test 2 events.

4.4.4.3 Tracer Test Analysis and Results. Data collected from the tracer test were analyzed according to the method described in Grove (1971) for a discharging-recharging well pair. The concentration of the discharged effluent is calculated by solving the transport equation:

$$\frac{\delta c}{\delta t} = -\frac{q}{\phi} \frac{\delta c}{\delta x} + D \frac{\delta^2 c}{\delta x^2}$$
 (1)

where:

c = concentration (ppm)

t = time(T)

q = the average darcy flux (L/T)

 ϕ = the porosity

D = the dispersion coefficient (L^2/T), for a series of discrete flow tubes connecting the well pair [see equations 36 and 38 in Grove (1971) for the solutions to equation 1].

The individual concentration values are summed incrementally through time into a composite concentration. The porosity and dispersion coefficient are varied until a suitable match is obtained between the calculated concentration curve and the field data. The concentration curves were generated using the FORTRAN program presented in the appendix of Grove (1971), and the curve matching was performed visually.

Certain assumptions govern the analysis. The aquifer is assumed to be homogeneous, and all flow occurs horizontally and radially outward from the recharge well and horizontally and radially toward the discharge well. The groundwater velocity is assumed to be steady state and dependent only on the pumping rate, and if diffusional effects are neglected, the dispersion coefficient is then proportional to the dispersivity (L). Although a tracer test is transient by nature, assuming a steady-state flow field is reasonable here because most of the changes occurred early in the test, especially compared to the overall test duration. These assumptions allow equation 1 to be solved analytically and avoid the necessity of a numerical solution.

For both tests, the bromide solution was injected into the aquifer at approximately 189 L/min (50 gal/min) for 112 minutes, and then the recharge rate was reduced to match the discharge rate. Considering the overall duration of the tests, pumping at 189 L/min (50 gal/min) for that time [instead of around 107 or 105 L/min (28.3 or 27.8 gal/min)] probably did not result in significant changes in the velocity field. However, it did result in the injection of almost twice the quantity of bromide into the aquifer than is accounted for in the equations. This would be expected to cause the theoretical curves to deviate from the data in at least two ways: (1) the bromide would arrive at the discharge well sooner than calculated and (2) the actual shape of the concentration pulse would be

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broader than the calculated curve. The greater volume of bromide solution and elevated injection rate would initially push the bromide solution farther out from the recharge well. In addition to the greater solution volume displacing a correspondingly greater volume in the aquifer, the elevated injection rate would also increase the hydraulic gradient away from the recharging well. Although this effect would be short-lived, especially compared to the overall duration of the test, it would influence the results to some degree. The actual shape of the concentration pulse would be broader than the calculated curve simply because the greater quantity of bromide solution would require a greater amount of time to be pumped out of the aquifer. These elements introduce some uncertainty into the analysis, but the mathematics involved are robust enough to keep the interpretation meaningful.

4.4.4.4 Tracer Test 1, February 6 through 11, 1995. Approximately 5 kg of lithium bromide was dissolved in a 5,600-gal tank, and the solution was injected into well 299-E28-23. Bromide concentrations measured at the recharge well averaged 195 ppm, which agreed with the calculated concentration of around 217 ppm. Baseline bromide concentration values measured at the discharging well ranged between 0.139 and 0.241 ppm. Because the background range represented greater than 20% of the maximum measured concentration at the discharge well (0.673 ppm), a background concentration of 0.149 ppm was subtracted from the concentration measured at the discharge well. The value of 0.149 was based on the trend of the background data and the rising limb of the concentration curve associated with the injected tracer pulse.

Figure 4-13 shows the measured data with the fitted analytical curve, with the dispersivity equal to 7 m (22 ft) and the porosity equal to 0.30. The data exhibit some scattering, which may be the result of preferential transport pathways and heterogeneities in the soil properties, changes and variations in the recharging and discharging rates, and variations in the sample measurements of the bromide concentration. The measured concentration exhibits an apparent downward trend from the 80- to 110-h mark of the test, but later measurements (216 to 238 h) of the bromide concentration ranged between 0.343 and 0.399 ppm above background. The downward trend diverges from the analytical solution, but the later measurements are in much closer agreement. Considering the high transmissivity and inherent heterogeneities of the aquifer, as well as the magnitude of the concentration measurements, especially compared to background measurements, the scatter exhibited by the data should be expected, and a wider range should be accorded to the calculated parameters.

Figures 4-14 and 4-15 show the effects of varying the porosity and dispersivity values, respectively. In general, the two parameters are essentially independent of one another: changes in one parameter do not affect the other. Changes in the porosity mostly affect the amplitude of the maximum concentration pulse but also influence the time required for the maximum concentration to reach the discharge well. Changes in the dispersivity mostly affect the time that the concentration peaks at the discharge well but also influence the amplitude of the concentration pulse. The porosity term (as used in this analysis) describes the amount of water in each flow tube that dilutes the tracer concentration. As the porosity value is increased, the dilution increases, hence the concentration decreases. The dispersivity describes the extent to which the tracer will spread longitudinally about the advective front in each flow tube. This affects how quickly the tracer reaches the discharging well.

The amplitude of the maximum concentration pulse is strongly correlated to the porosity; the correlation is almost inversely linear. Figure 4-14 shows that doubling the porosity reduces the peak concentration by about one-half, whereas halving the porosity increases the peak concentration by about a factor of two. Figure 4-15 shows the effects of varying the dispersivity. As the dispersivity increases, the breakthrough time of the concentration peak shifts to the left. In general, the curves

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also become more peaked as the dispersivity increases, as seen by comparing the curves with dispersivities of 5, 7, and 9 m (16, 22, and 28 ft).

4.4.4.5 Tracer Test 2, May 21 through 25, 1995. Ten kilograms of potassium bromide were dissolved in the 3-m- (10-ft) diameter tank holding 320 cm (126 in.) of water $\{23,352 \text{ L} \text{ [B x (5 \text{ ft})}^2 \text{ x } 126/12 \text{ ft x } 7.48 \text{ gal/ft}^3 = 6,169 \text{ gal of water]}\}$, and the solution was injected into well 299-E28-25. All bromide concentrations measured at the recharge well exceeded 3,500 ppm, which is an order of magnitude higher than what the concentration was calculated to be (\sim 288 ppm). The reason for this discrepancy has not yet been determined. For these calculations, it was assumed that through the process of mixing in the tank, turbulence through the injection, and further mixing in the aquifer, the bromide should have dispersed fairly uniformly throughout the solution to the 288 ppm concentration.

Figure 4-16 shows the data resulting from the fitted analytical curve, with the dispersivity equal to 3.7 m (12 ft) and the porosity equal to 0.22. Bromide concentration measurements taken before the test indicated that the background bromide concentration at well 299-E28-23 was 0.162 ppm. As expected, the initial bromide breakthrough arrived at the discharge well before the curve predicted, and the peak of the measured concentrations was broader than the theoretical curve. Also, the later time-measured concentration data remained higher than the predicted analytical curve, indicating that either recirculation of the bromide from the extraction well to the injection well occurred, or the bromide injected over the 105 L/min (27.8 gal/min) (and thus not accounted for in the analysis equations) resulted in the elevated values. Considering that the bromide concentration attenuated by a factor of about 50 between the injection and extraction wells, even recirculation of the bromide would not be sufficient to maintain the elevated concentrations measured at the extraction well. Thus, the second explanation appears more plausible. Figures 4-17 and 4-18 show the sensitivity of the analysis to the effects of varying the porosity and dispersivity values, respectively.

The results of the two tests indicate that the effective porosity ranges between 0.22 and 0.30, and the dispersivity ranges between 3.7 and 6.3 m (12 and 22 ft). Based on the assumptions of the analysis, the values determined from the second test are probably more representative of the aquifer properties. The wells used in the second test were more than twice as close as the wells used in the first test, and thus much more likely to create the flow field described by the mathematics. With a transmissivity estimated to be 20,000 m²/day (216,000 ft²/day), pumping at 106 L/min (28 gal/min) would not result in more than 1.3 cm (0.5 in.) of drawdown outside the well casing (according to the Theis equation). Thus, the influence of the pumping on the aquifer would not extend out very far radially from the well. With such a small hydraulic change in the aquifer, differentiating between the transport caused by the pumping and the transport because of ambient conditions becomes difficult. As the distance between the wells increases, the injected bromide would be less likely to follow a direct path to the extraction well, and the extraction well would pump more ambient water than tracer-charged water from the injection well. Tracer arriving at the extraction well would be more diluted than the mathematics would dictate, thus skewing the analytical curve toward higher porosity and dispersivity values. The wells used in the second test were probably close enough to establish the flow field described by the analytical equation, and the values derived from that analysis provide good estimates of the aquifer properties. The results from the first tracer test provide bounds on the maximum values of the porosity and dispersivity. Although greater in magnitude than the results from the second test, the values from the first test are within reason for such a highly transmissive aquifer.

As a check of the values determined from the tracer analyses, the gradient calculated from the tracer analysis was compared to the drawdown predicted by the Theis equation. The tracer test gradient was

based on the straight-line distance between the injection and recharge wells and the travel time along this streamline. The actual gradient would have to be greater than this value because all other streamlines are longer than the straight-line distance. Using Darcy's law,

$$v = \frac{q}{\Phi} = \frac{K \Delta H}{\Phi \Delta x} \tag{2}$$

where:

v = groundwater linear velocity (m/day)

q = darcy flux (m/day)

 ϕ = porosity

K = hydraulic conductivity (m/day) $\Delta H = hydraulic head difference (m)$

 $\Delta x = distance (m).$

Equation 2 can be rearranged to solve for the hydraulic head difference,

$$\Delta H = \frac{\nu \phi}{K} \Delta x \tag{3}$$

The velocity can be estimated from the travel time along the straight line connecting the two wells (provided in the output files: 206.6 h for test 1 and 28.3 h for test 2) and the distance between the two wells [17.46 m (57.28 ft) for test 1 and 7.32 m (24.02 ft) for test 2]:

$$v_1 = (\frac{57.28ft}{206.6hr})(\frac{24hr}{1day}) = 6.66\frac{ft}{day} \quad or \quad 2.03\frac{m}{days}$$
 (4)

and

$$v_2 = (\frac{24.02ft}{26.8hr})(\frac{24hr}{1day}) = 21.5\frac{ft}{day} \quad or \quad 6.56\frac{m}{day}$$
 (5)

For K = 1,500 m/day (4,800 ft/day) (e.g., see Connelly et al. 1992) and $\phi = 0.25$ (based on the tracer test results),

$$\Delta H_1 = (6.66 \frac{ft}{day}) \frac{0.25}{(4800 \frac{ft}{day})} (57.28 ft) = 2.0 \times 10^{-2} ft \quad or \quad 5.9 \times 10^{-3} m$$
 (6)

and

$$\Delta H_2 = (21.5 \frac{ft}{day}) \frac{0.25}{(4800 \frac{ft}{day})} (24.02ft) = 2.6 \times 10^{-2} ft \quad or \quad 8.0 \times 10^{-3} m$$
 (7)

Because the system is symmetrical, the drawdown at the extraction well is one-half of the hydraulic head difference, or 3.0×10^{-3} m (1.0×10^{-2} ft) and 4.0×10^{-3} m (1.3×10^{-2} ft), respectively.

The Theis equation can be used to calculate the drawdown as a result of pumping at a well,

$$s = \frac{Q}{4\pi T} W(u), \qquad u = \frac{r^2 Sy}{4Tt}$$
 (8)

where:

s = drawdown (m)

 $Q = pumping rate (m^3/day)$

T = aquifer transmissivity (m²/day)

W(u) = well function [see Freeze and Cherry (1979), p. 318 for tabulated values of this function]

r = well screen radius (m) Sy = aquifer specific yield

t = time (days).

For this analysis, the porosity was substituted for the specific yield and $T = 1,500 \text{ m/day} \times 13.7 \text{ m}$ (4,800 ft/day x 45 ft) = 20,000 m³/day (216,000 ft²/day). For a well screen radius of 0.051 m (0.167 ft) (4-in.-diameter well) and a time of 3 days (sufficient to reach reasonably steady-state conditions),

$$u = \frac{(0.051m)^2(0.25)}{4(20,000\frac{m^2}{day})(3days)} = 2.7 \times 10^{-9}$$
(9)

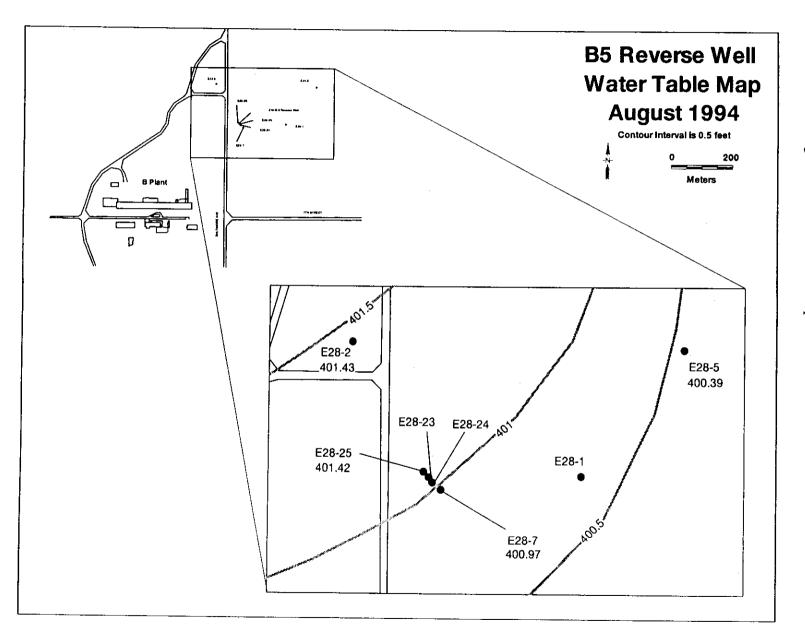
and W(2.7 x 10^{-9}) = 19.33. For a pumping rate of 154.2 m³/day (5,448 ft³/day) (28.3 gal/min), the drawdown is

$$s_1 = \frac{(154.2 \frac{m^3}{day})}{(4)(\pi)(20,000 \frac{m^2}{day})} (19.33) = 1.2 \times 10^{-2} m$$
 (10)

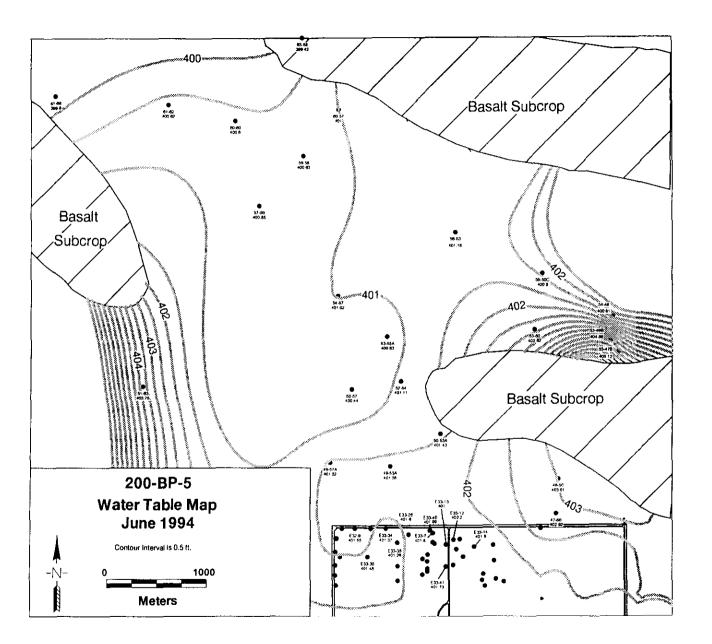
For a pumping rate of 151.5 m³/day (5,352 ft³/day) (27.8 gal/min), the drawdown is

$$s_2 = \frac{(151.5 \frac{m^3}{day})}{(4)(\pi)(20,000 \frac{m^2}{day})} (19.33) = 1.2 \times 10^{-2} m$$
 (11)

The drawdown values calculated by the two methods are the same order of magnitude $(3.0 \times 10^3 \text{ and } 4.0 \times 10^3 \text{ m})$ compared to $1.2 \times 10^2 \text{ and } 1.2 \times 10^2 \text{ m}$, respectively), and provide additional confidence in the values calculated by the tracer test analysis. As noted before, the drawdown determined from the tracer test analysis represents a minimum value; the actual drawdown would be expected to be greater, bringing the values into even closer agreement.



4F-1



4F-2

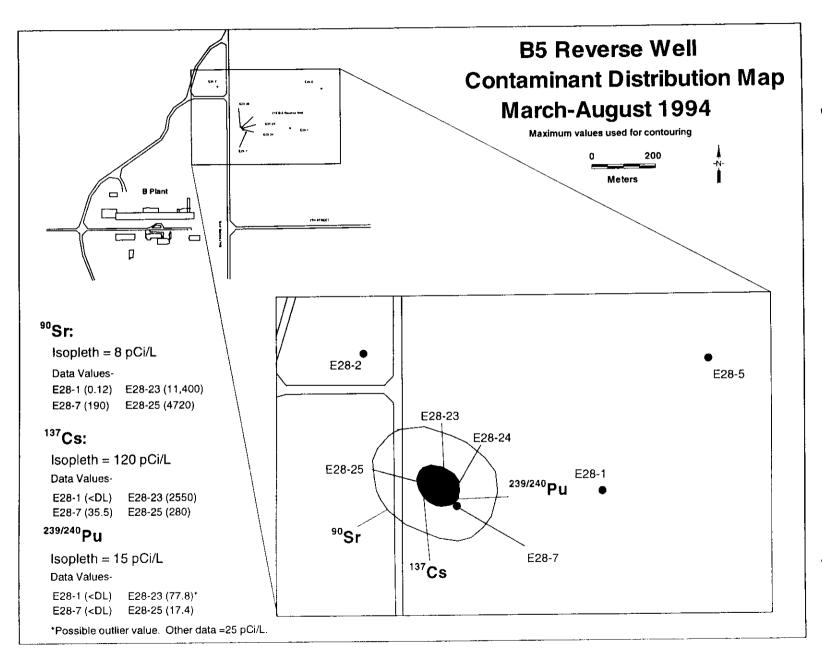
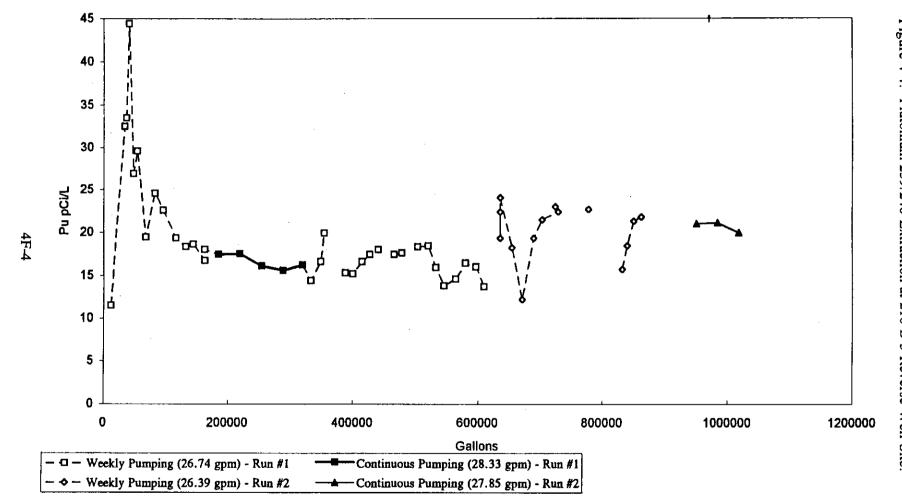
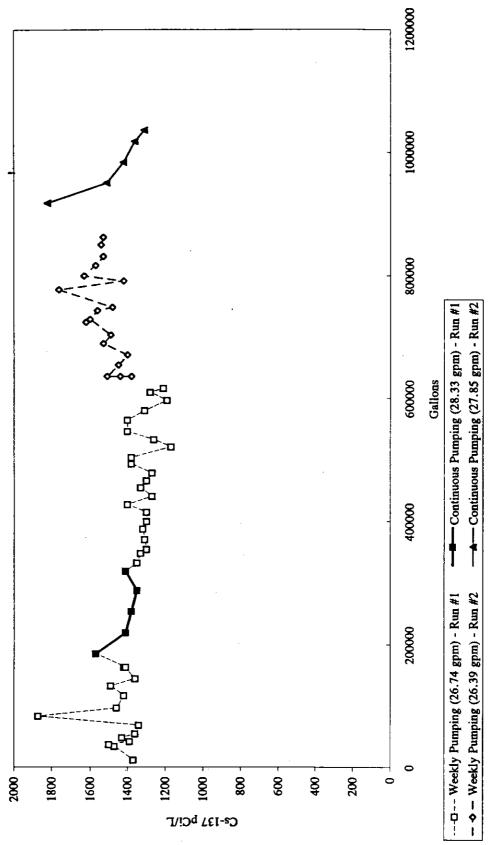


Figure 4-3. 216-B-5 Reverse Well Baseline Contamination Distribution Map



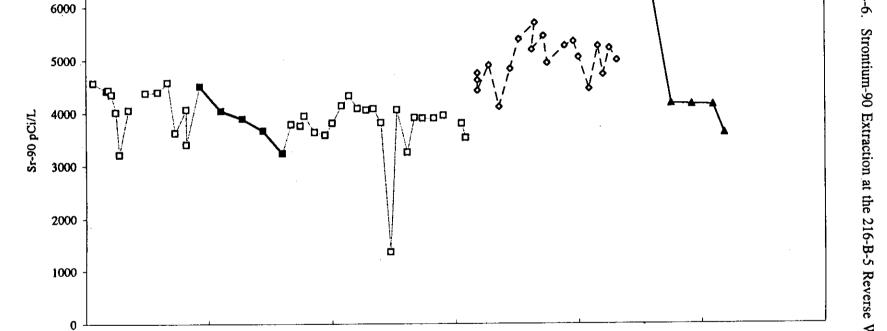
Plutonium-239/240 Extraction at 216-B-5 Reverse Well Site

Figure 4-5. Cesium-137 Extraction at the 216-B-5 Reverse Well Site.



4F-6

7000



600000

Gallons

Continuous Pumping (28.33 gpm) - Run #1

— ▲ Continuous Pumping (27.85 gpm) - Run #2

400000

200000

--- Weekly Pumping (26.74 gpm) - Run #1

◆ - Weekly Pumping (26.39 gpm) - Run #2

Figure 4-6. Strontium-90 Extraction at the 216-B-5 Reverse Well Site. 1200000

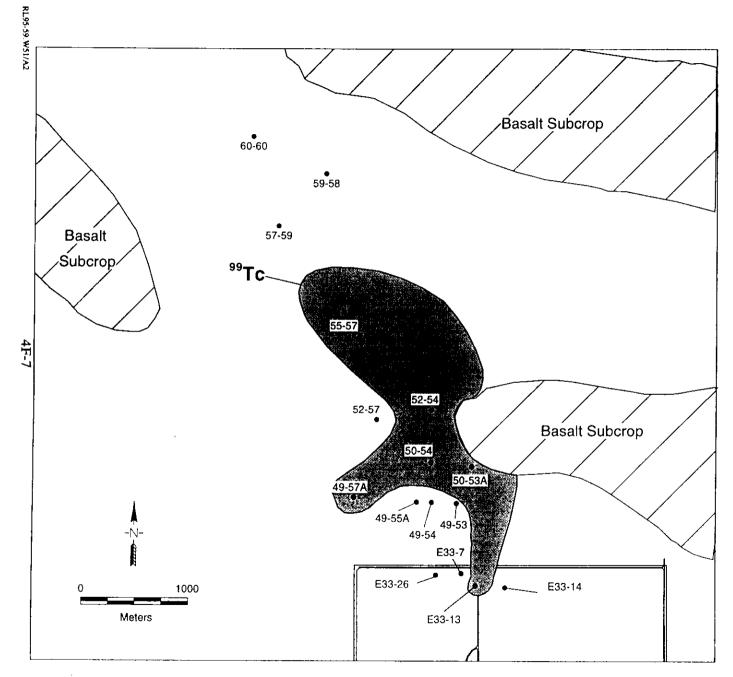
1000000

800000

Figure 4-7.

216-BY

Baseline Contamination Distribution Map



BY Cribs Contaminant Distribution Map April 1994

⁹⁹Tc:

⁶⁰Co:

Isopleth = 100 pCi/L* Data Values-E33-7 (22) 50-53A (27) E33-13 (9) 50-54 (19) E33-14(4) 52-54 (74) E33-26(6) 52-57 (5) 49-53(10) 55-57 (50) 49-54(20) 57-59 (<DL) 49-55A (8) 59-58 (1) 49-57A (17) 60-60 (<DL)

*No ⁶⁰Co value exceeds 100 pCi/L.

Figure 4-8. Cobalt-60 Extraction at the 216-BY Cribs Site.

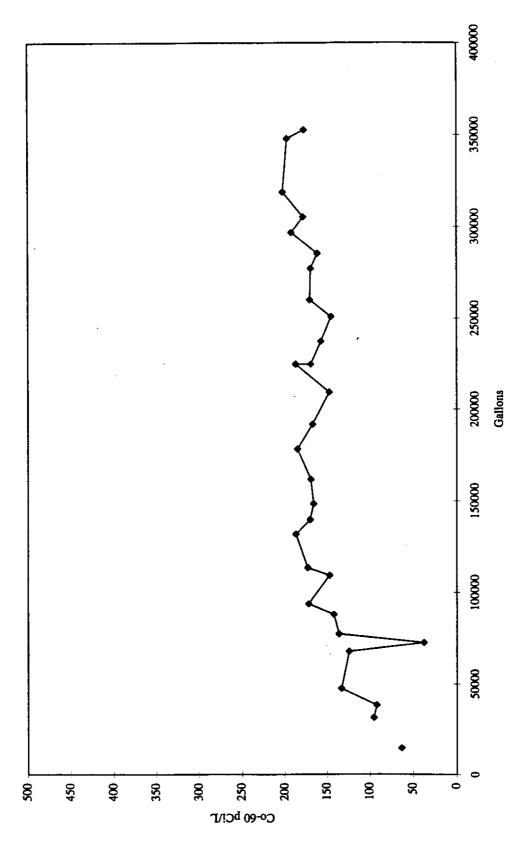
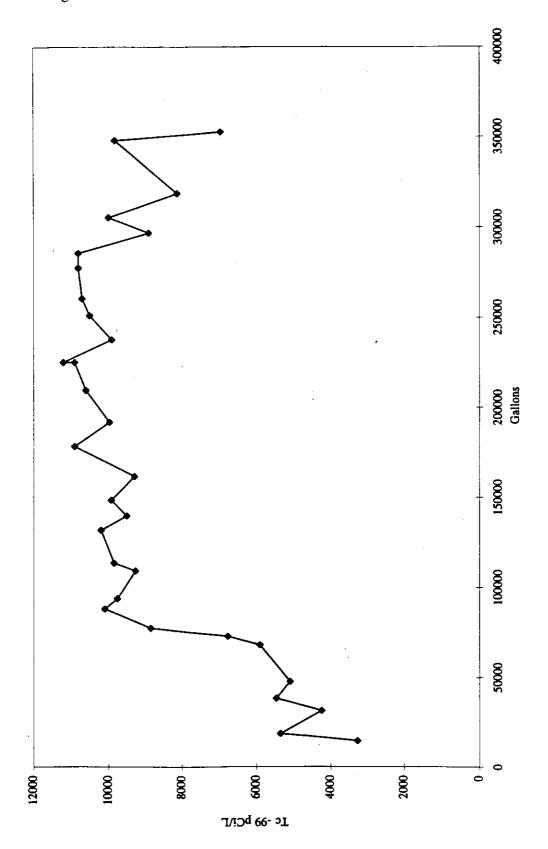


Figure 4-9. Technetium-99 Extraction at the 216-BY Cribs Plume Site.



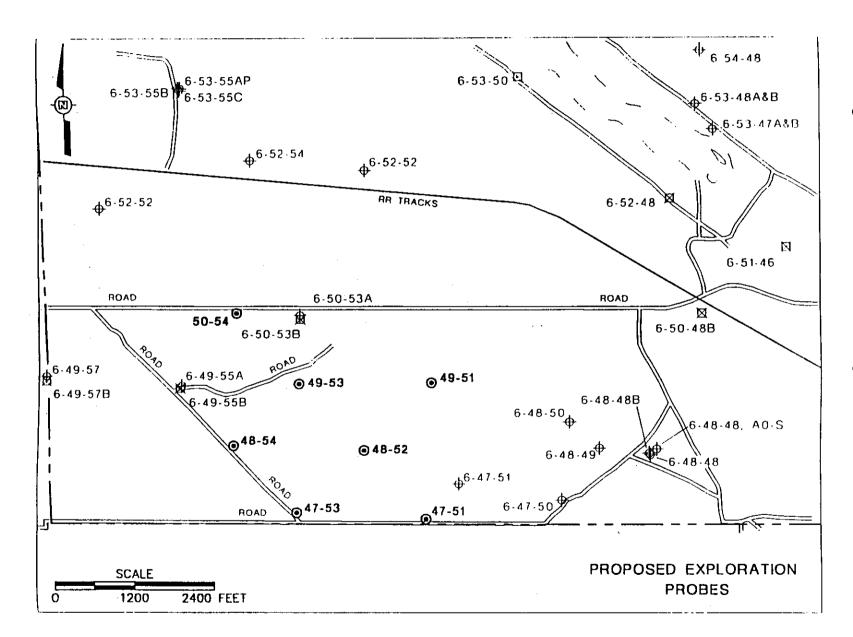


Figure 4-10. Sonic Push Well Location Map at the 216-BY Cribs Site

Figure 4-11. Groundwater Sampling Map for Sonic Push Testing at 216-BY Cribs.

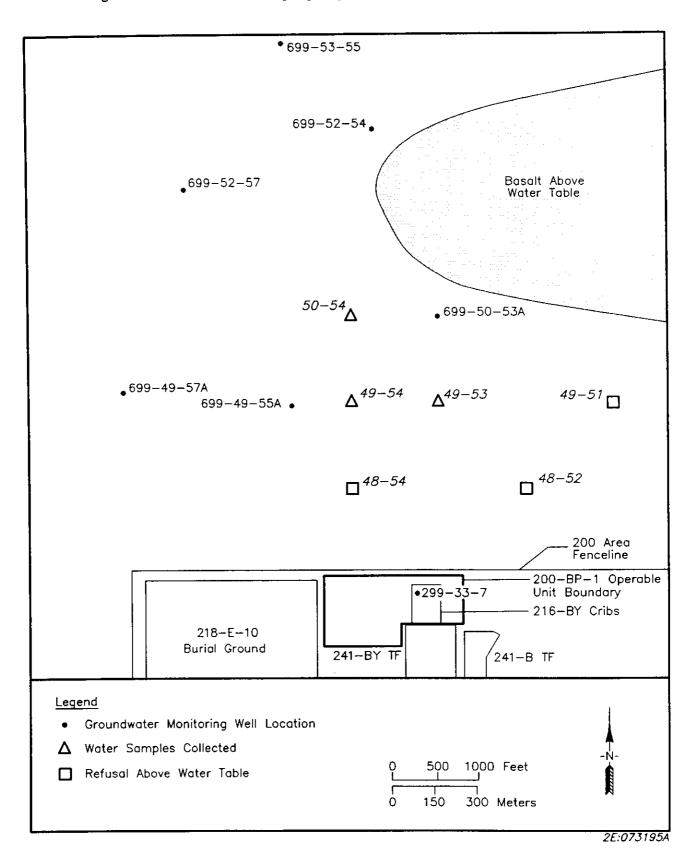


Figure 4-12. In Situ Velocity Flowmeter Test Wells in the 216-BY Cribs Plume.

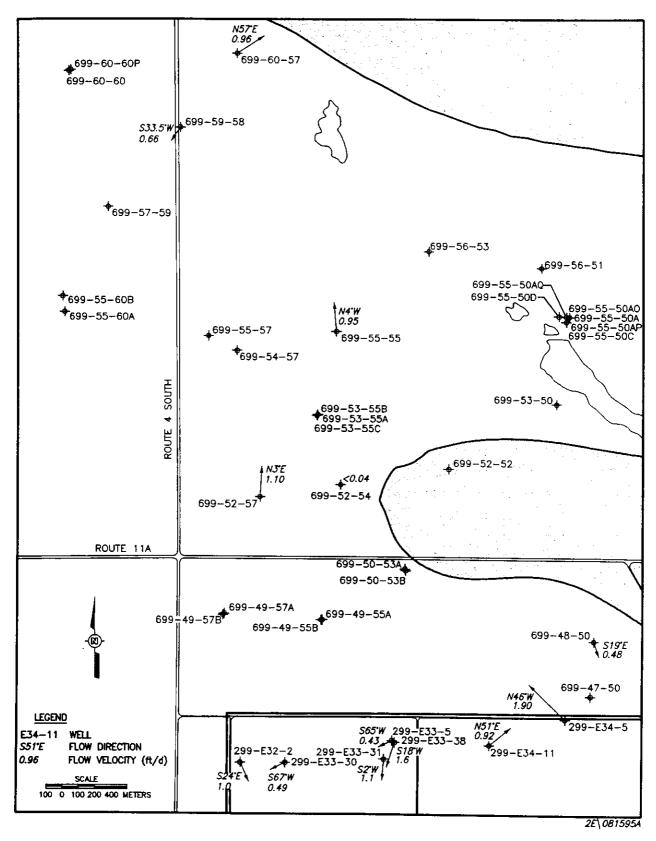
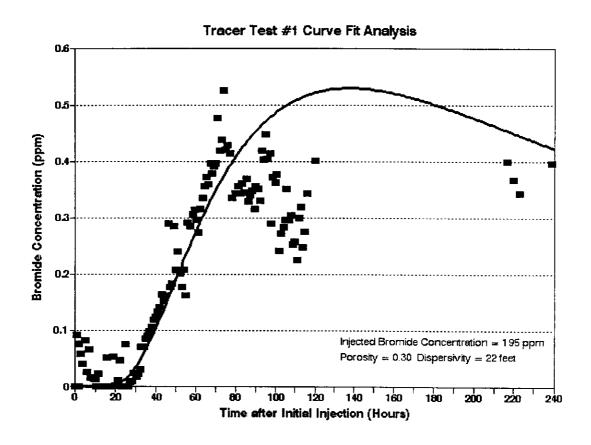


Figure 4-13. Comparison of the Analytical Solution Output to the Measured Bromide Concentration at Well 299-E28-23.



4F-13

Figure 4-14. Sensitivity Analysis of the Analytical Solution Output to the Porosity Term Varied by 17%.

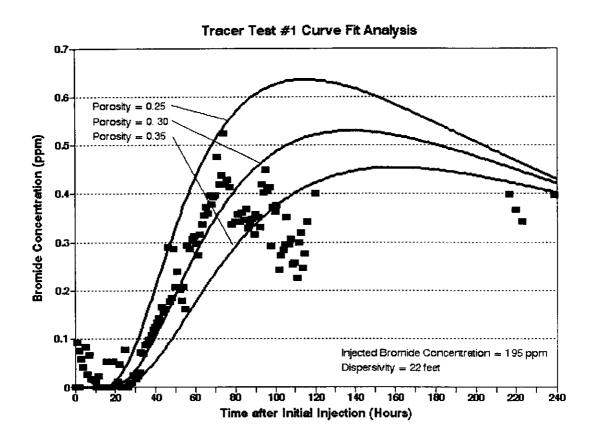
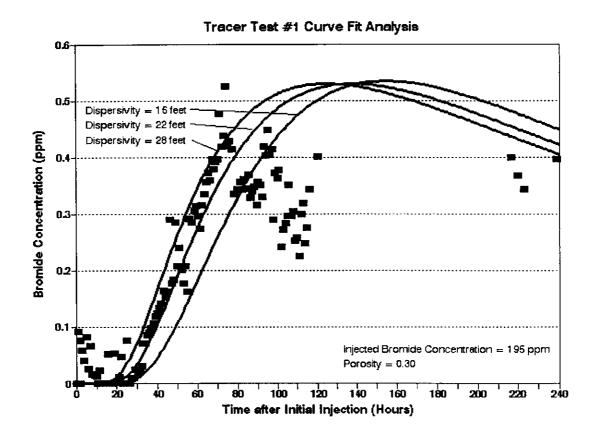


Figure 4-15. Sensitivity Analysis of the Analytical Solution Output to the Dispersivity Term Varied by 27%.



4F-15

Figure 4-16. Comparison of the Analytical Solution Output to the Measured Bromide Concentration at Well 299-E28-23.

Tracer Test #2 Curve Fit Analysis

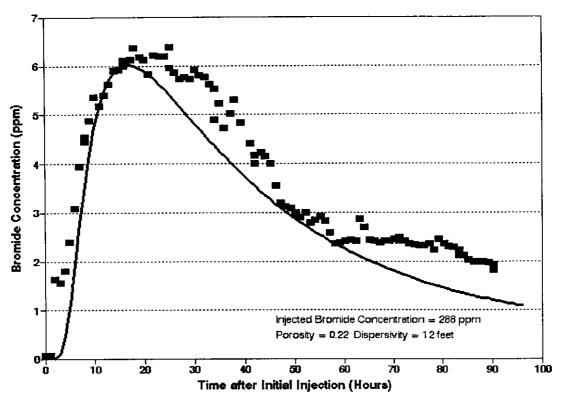


Figure 4-17. Sensitivity Analysis of the Analytical Solution Output to the Porosity Term Varied by 27%.

Tracer Test #2 Curve Fit Analysis

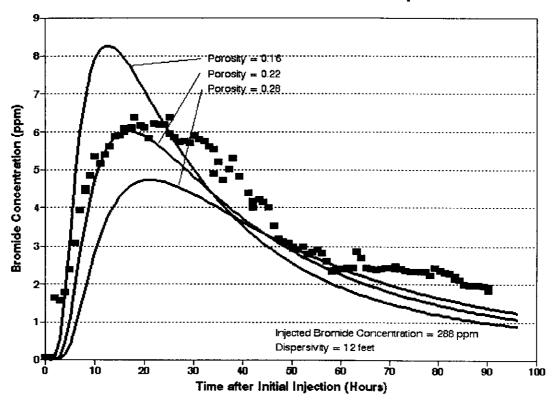


Figure 4-18. Sensitivity Analysis of the Analytical Solution Output to the Dispersivity Term Varied by 25%.

Tracer Test #2 Curve Fit Analysis

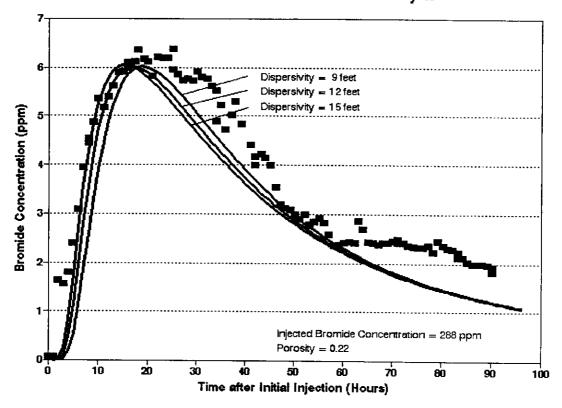


Table 4-1. Summary of Well Suitability Activities and Production Data.

Well	Camera Survey	Well Scrubbed/ Cleaned	Well Redeveloped	Well Deepened	Head Loss/Gain	Pump/ Injection Rate (L/min)
299-E28-1	Yes	Yes	No	No	0.2/5.5	75/76
299-E28-2	Yes	Yes	No	No	NMD/-	76
299-E28-7	No	No	No	No	NMD/1.5	16.3/208
299-E28-23	No	No	No	No	.01/-	70
299-E28-24	No	No	Yes	No	8.3/-	5.7
299-E28-25	No	No	No	No	0.2/-	76.5
699-49-55A	Yes '	No	No	No	9/6.4	8.7/76
699-49-57A	Yes	No	No	No	NMD	23
699-50-53A	No	No	Yes	Yes²	16.5	19

^aWell 699-50-53A was drilled over 6 m (20 ft) into the upper basalt. The extraction pump was placed within the basalt to increase well yield due to the limited saturated thickness.

NMD = no measurable drawdown.

Table 4-2. 216-B-5 Reverse Well Baseline Water Quality Data. (Section 4.3.1.2) (2 Sheets)

Well	Date Sampled	Contaminant	Activity	Units
200 E29 1	03-23-94	¹³⁷ Cs	-0.8	pCi/L
299-E28-1	03-23-94	¹³⁷ Cs	-5.63	pCi/L
299-E28-2ª				
299-E28-5ª				
299-E28-7	04-18-94	¹³⁷ Cs	15.3	pCi/L
	08-30-94	¹³⁷ Cs	35.5	pCi/L
	03-29-94	¹³⁷ Cs	2,215	pCi/L
299-E28-23	06-30-94	¹³⁷ Cs	2,550 ^b	pCi/L
	07-18-94	¹³⁷ Cs	2,030	pCi/L
299-E28-24 ^a				
	03-29-94	¹³⁷ Cs	180	pCi/L
299-E28-25	07-20-94	¹³⁷ Cs	234/159°	pCi/L
	08-29-94	¹³⁷ Cs	280	pCi/L
299-E28-1	03-23-94	^{239/240} Pu	-0	pCi/L
299-E28-2ª			*	T- 47
299-E28-5ª				
299-E28-7	04-18-94	^{239/240} Pu	-0	pCi/L
· ·	03-29-94	^{239/240} Pu	25.7	pCi/L
299-E28-23	06-30-94	^{239/240} Pu	20.1 ^b	pCi/L
	07-18-94	^{239/240} Pu	77.8°/25 ^d	pCi/L
299-E28-24 ^a				
200 E29 25	03-29-94	^{239/240} Pu	17.35	pCi/L
299-E28-25	08-29-94	^{239/240} Pu	0	pCi/L
299-E28-1	03-23-94	⁹⁰ Sr	0.12	pCi/L
299-E28-2 ^a				
299-E28-5 ^a				
299-E28-7	04-18-94	⁹⁰ Sr	160	pCi/L
	08-30-94	⁹⁰ Sr	190	pCi/L
	03-29-94	⁹⁰ Sr	9,460	pCi/L
299-E28-23	06-30-94	⁹⁰ Sr	11,400 ^b	pCi/L
	07-18-94	%Sr	8,690/4,710°	pCi/L
299-E28-24ª				

96 3494 3094 DOE/RL-95-59 Rev. 0

Table 4-2. 216-B-5 Reverse Well Baseline Water Quality Data. (Section 4.3.1.2) (2 Sheets)

Well	Date Sampled	Contaminant	Activity	Units
200 F29 25	03-29-94	⁹⁰ Sr	4,720	pCi/L
299-E28-25	07-20-94	⁹⁰ Sr	4,250	pCi/L

NOTES:

^aWell not sampled.

bValue used for initial resin testing.

^cPossible outlier.

^dThe two values represent changes in concentration over time during well suitability testing.

Table 4-3. Summary of the Average Concentration of Contaminants in Groundwater During Pumping.

Ranking	Run # and Pumping Cycle	Contaminant	Average Concentration (pCi/L)
1	Run 2. Pumping 99.94 L/min, 8 h/day.	¹³⁷ Cs	1,590
2	Run 2. Continuous pumping at 105.2 L/min.	¹³⁷ Cs	1,490
3	Run 1. Continuous pumping at 107.1 L/min.	¹³⁷ Cs	1,425
4	Run 1. Pumping 101.1 L/min, 8 h/day.	¹³⁷ Cs	1,360
1 ^b	Run 2. Pumping 99.9 L/min, 8 h/day.	^{239/240} Pu	20
1 ^b	Run 2. Continuous pumping at 105.2 L/min.	^{239/240} Pu	20
2	Run 1. Pumping 101.1 L/min, 8 h/day.	^{239/240} Pu	19
3	Run 1. Continuous at pumping 107.1 L/min.	^{239/240} Pu	17
1	Run 2. Pumping 99.9 L/min, 8 h/day.	⁹⁰ Sr	4,970
2	Run 2. Continuous pumping at 105.2 L/min.	⁹⁰ Sr	4,488
3	Run 1. Continuous pumping at 107.1 L/min.	**Sr	3,894
4	Run 1. Pumping 101.1 L/min, 8 h/day.	⁹⁰ Sr	3,663

^aA "1" denotes the highest average concentration of contaminant in the groundwater during treatability testing; a "4" denotes the lowest.

^bThe average concentration of ^{239/240}Pu in the groundwater is the same.

Table 4-4. 216-BY Cribs Baseline Groundwater Data.

299-E33-7	Well	Date Sampled	Contaminant	Activity	Units
08-29-94 %Co 9.02 pCi/L	200 F32 7	04-26-94	[∞] Co	22.1	pCi/L
299-E33-14 04-28-94 [∞] Co 3.61 pCi/L 299-E33-26 04-28-94 [∞] Co 5.95 pCi/L 699-49-55A 04-11-94 [∞] Co 7.83 pCi/L 699-49-57A 04-11-94 [∞] Co 17 pCi/L 699-50-53A 04-11-94 [∞] Co 27.2 pCi/L 699-52-54 04-21-94 [∞] Co 28.3° pCi/L 699-52-57 04-20-94 [∞] Co 74.3 pCi/L 699-52-57 04-20-94 [∞] Co 5.47 pCi/L 699-53-59 04-15-94 [∞] Co 5.47 pCi/L 699-57-59 04-15-94 [∞] Co 4.53 pCi/L 699-59-8 04-21-94 [∞] Co 1.2 pCi/L 699-60-60 04-21-94 [∞] Co 9.7 pCi/L 49-53* 07-30-94 [∞] Co 20.1 pCi/L 49-54* 07-26-94 [∞] Co 20.1 pCi/L 299-E33-7 07-26-94 [∞] Co	299-E33-7	08-29-94	[∞] Co	9.02	pCi/L
299-E33-26 04-28-94 ⁶⁰ Co 5.95 pCi/L 699-49-55A 04-11-94 ⁶⁰ Co 7.83 pCi/L 699-49-57A 04-11-94 ⁶⁰ Co 17 pCi/L 699-50-53A 04-11-94 ⁶⁰ Co 27.2 pCi/L 699-52-54 04-21-94 ⁶⁰ Co 28.3° pCi/L 699-52-57 04-20-94 ⁶⁰ Co 74.3 pCi/L 699-55-57 04-19-94 ⁶⁰ Co 5.47 pCi/L 699-57-59 04-15-94 ⁶⁰ Co 4.53 pCi/L 699-59-58 04-21-94 ⁶⁰ Co 4.53 pCi/L 699-60-60 04-21-94 ⁶⁰ Co 1.2 pCi/L 49-53 ^b 07-30-94 ⁶⁰ Co 9.7 pCi/L 49-54 ^b 07-26-94 ⁶⁰ Co 20.1 pCi/L 299-E33-7 04-26-94 ⁶⁰ Co 19.2 pCi/L 299-E33-13 01-14-94 ⁶⁰ Tc 685 pCi/L 299-E33-13 01-14-94	299-E33-13	04-14-95	[∞] Co	9.21	pCi/L
699-49-55A 04-11-94	299-E33-14	04-28-94	[∞] Co	3.61	pCi/L
699-49-57A 04-11-94 6°Co 17 pCi/L 699-50-53A 04-11-94 6°Co 27.2 pCi/L 699-52-54 04-21-94 6°Co 28.3° pCi/L 699-52-57 04-21-94 6°Co 74.3 pCi/L 699-52-57 04-19-94 6°Co 5.47 pCi/L 699-55-57 04-19-94 6°Co 50.4 pCi/L 699-57-59 04-15-94 6°Co 4.53 pCi/L 699-59-58 04-21-94 6°Co 1.2 pCi/L 699-60-60 04-21-94 6°Co -0.46 pCi/L 49-53° 07-30-94 6°Co 20.1 pCi/L 49-54° 07-26-94 6°Co 20.1 pCi/L 299-E33-7 04-26-94 6°Co 19.2 pCi/L 299-E33-13 01-14-94 6°Tc 685 pCi/L 299-E33-14 04-28-94 6°Tc 34.7 pCi/L 299-E33-26 04-28-94 6°Tc 775 </td <td>299-E33-26</td> <td>04-28-94</td> <td>[∞]Co</td> <td>5.95</td> <td>pCi/L</td>	299-E33-26	04-28-94	[∞] Co	5.95	pCi/L
699-50-53A	699-49-55A	04-11-94	⁶⁰ Co	7.83	pCi/L
699-50-53A 06-30-94 ***Co 28.3** pCi/L 699-52-54 04-21-94 ***Co 74.3 pCi/L 699-52-57 04-20-94 ***Co 5.47 pCi/L 699-55-57 04-19-94 ***Co 50.4 pCi/L 699-57-59 04-15-94 ***Co -4.53 pCi/L 699-59-58 04-21-94 ***Co -0.46 pCi/L 699-60-60 04-21-94 ***Co 9.7 pCi/L 49-53* 07-30-94 ***Co 9.7 pCi/L 49-54* 07-26-94 ***Co 20.1 pCi/L 50-54* 07-28-94 ***Co 19.2 pCi/L 299-E33-7 04-26-94 ***Tc 619 pCi/L 299-E33-13 01-14-94 ***Tc 1,380 pCi/L 299-E33-14 04-28-94 ***Tc 34.7 pCi/L 299-E33-26 04-28-94 **Tc 775 pCi/L 699-49-55A 04-11-95 **Tc	699-49-57A	04-11-94		17	pCi/L
699-52-54 04-21-94 6°Co 74.3 pCi/L 699-52-57 04-20-94 6°Co 74.3 pCi/L 699-55-57 04-20-94 6°Co 5.47 pCi/L 699-55-57 04-19-94 6°Co 50.4 pCi/L 699-55-57 04-19-94 6°Co 50.4 pCi/L 699-59-58 04-21-94 6°Co 1.2 pCi/L 699-60-60 04-21-94 6°Co -0.46 pCi/L 49-53 07-30-94 6°Co 9.7 pCi/L 49-54 07-26-94 6°Co 20.1 pCi/L 299-E33-7 04-26-94 6°Co 19.2 pCi/L 299-E33-13 01-14-94 6°Co 19.2 pCi/L 299-E33-14 04-28-94 6°Co 19.2 pCi/L 299-E33-15 01-14-94 6°Co 19.2 pCi/L 299-E33-16 04-28-94 6°Co 19.2 pCi/L 299-E33-17 04-26-94 6°Co 19.2 pCi/L 299-E33-18 01-14-94 6°Cc 19.2 pCi/L 299-E33-19 01-14-94 6°Cc 19.2 pCi/L 299-E33-10 04-28-94 6°Cc 19.2 pCi/L 299-E33-10 01-14-94 6°Cc 19.2 pCi/L 299-E33-10 04-28-94 6°Cc 19.2 pCi/L 299-E33-7 04-28-9	600 50 52 A	04-11-94	[∞] Co	27.2	pCi/L
699-52-57 04-20-94	099-30-33A	06-30-94	[∞] Co	28.3ª	pCi/L
699-55-57 04-19-94 ⁶⁰ Co 50.4 pCi/L 699-57-59 04-15-94 ⁶⁰ Co -4.53 pCi/L 699-59-58 04-21-94 ⁶⁰ Co 1.2 pCi/L 699-60-60 04-21-94 ⁶⁰ Co -0.46 pCi/L 49-53 ^b 07-30-94 ⁶⁰ Co 9.7 pCi/L 49-54 ^b 07-26-94 ⁶⁰ Co 20.1 pCi/L 50-54 ^b 07-28-94 ⁶⁰ Co 19.2 pCi/L 299-E33-7 04-26-94 ⁶⁰ Tc 619 pCi/L 299-E33-13 01-14-94 ⁶⁰ Tc 685 pCi/L 299-E33-14 04-28-94 ⁶⁰ Tc 34.7 pCi/L 299-E33-26 04-28-94 ⁶⁰ Tc 775 pCi/L 699-49-55A 04-11-95 ⁶⁰ Tc 381 pCi/L 699-49-57A 04-11-94 ⁶⁰ Tc 1,525 pCi/L 699-50-53A 04-11-94 ⁶⁰ Tc 1,480 ⁶ pCi/L 699-52-54 04-21-94	699-52-54	04-21-94)	74.3	pCi/L
699-57-59 04-15-94 ©Co -4.53 pCi/L 699-59-58 04-21-94 ©Co 1.2 pCi/L 699-60-60 04-21-94 ©Co -0.46 pCi/L 49-53b 07-30-94 ©Co 9.7 pCi/L 49-54b 07-26-94 ©Co 20.1 pCi/L 50-54b 07-28-94 ©Co 19.2 pCi/L 299-E33-7 04-26-94 °Tc 619 pCi/L 299-E33-13 01-14-94 °Tc 685 pCi/L 299-E33-14 04-28-94 °Tc 1,380 pCi/L 299-E33-26 04-28-94 °Tc 775 pCi/L 299-E33-26 04-28-94 °Tc 775 pCi/L 699-49-55A 04-11-95 °Tc 381 pCi/L 699-49-57A 04-11-94 °Tc 1,525 pCi/L 699-50-53A 04-11-94 °Tc 1,480° pCi/L 699-52-54 04-21-94 °Tc 2.47 <t< td=""><td>699-52-57</td><td>04-20-94</td><td>⁶⁰Co</td><td>5.47</td><td>pCi/L</td></t<>	699-52-57	04-20-94	⁶⁰ Co	5.47	pCi/L
699-59-58 04-21-94 ©Co 1.2 pCi/L 699-60-60 04-21-94 ©Co -0.46 pCi/L 49-53b 07-30-94 ©Co 9.7 pCi/L 49-54b 07-26-94 ©Co 20.1 pCi/L 50-54b 07-28-94 ©Co 19.2 pCi/L 299-E33-7 04-26-94 ©Tc 619 pCi/L 299-E33-7 04-26-94 ©Tc 685 pCi/L 299-E33-13 01-14-94 ©Tc 1,380 pCi/L 299-E33-14 04-28-94 ©Tc 34.7 pCi/L 299-E33-26 04-28-94 ©Tc 775 pCi/L 699-49-55A 04-11-95 ©Tc 381 pCi/L 699-49-57A 04-11-94 ©Tc 1,525 pCi/L 699-50-53A 04-11-94 ©Tc 1,690 pCi/L 699-52-54 04-21-94 ©Tc 4,310 pCi/L 699-52-57 04-20-94 ©Tc 2,47 <td< td=""><td>699-55-57</td><td>04-19-94</td><td>⁶⁰Co</td><td>50.4</td><td>pCi/L</td></td<>	699-55-57	04-19-94	⁶⁰ Co	50.4	pCi/L
699-60-60 04-21-94 ***Co -0.46 pCi/L 49-53** 07-30-94 ***Co 9.7 pCi/L 49-54** 07-26-94 ***Co 9.7 pCi/L 50-54** 07-28-94 ***Co 19.2 pCi/L 299-E33-7 04-26-94 ***Tc 619 pCi/L 299-E33-7 08-29-94 ***Tc 685 pCi/L 299-E33-13 01-14-94 ***Tc 1,380 pCi/L 299-E33-14 04-28-94 ***Tc 775 pCi/L 299-E33-26 04-28-94 ***Tc 775 pCi/L 699-49-55A 04-11-95 ***Tc 381 pCi/L 699-49-57A 04-11-94 ***Tc 1,525 pCi/L 699-50-53A 04-11-94 ***Tc 1,480* pCi/L 699-52-54 04-21-94 ***Tc 4,310 pCi/L 699-52-57 04-20-94 ***Tc 2,47 pCi/L 699-55-57 04-19-94 ***Tc	699-57-59	04-15-94		-4.53	pCi/L
49-53b 07-30-94 60Co 9.7 pCi/L 49-54b 07-26-94 60Co 20.1 pCi/L 50-54b 07-28-94 60Co 19.2 pCi/L 299-E33-7 04-26-94 90Tc 619 pCi/L 299-E33-7 08-29-94 90Tc 685 pCi/L 299-E33-13 01-14-94 90Tc 1,380 pCi/L 299-E33-14 04-28-94 90Tc 34.7 pCi/L 299-E33-26 04-28-94 90Tc 775 pCi/L 699-49-55A 04-11-95 90Tc 381 pCi/L 699-49-57A 04-11-94 90Tc 1,525 pCi/L 699-50-53A 04-11-94 90Tc 1,690 pCi/L 699-52-54 04-21-94 90Tc 1,4800 pCi/L 699-52-55 04-21-94 90Tc 2,47 pCi/L 699-55-57 04-19-94 90Tc 2,100 pCi/L 699-57-59 04-15-94 90Tc 2,10	699-59-58	04-21-94	. ⁶⁰ Co	1.2	pCi/L
49-54h 07-26-94 60 Co 20.1 pCi/L 50-54h 07-28-94 60 Co 19.2 pCi/L 299-E33-7 04-26-94 60 Co 19.2 pCi/L 299-E33-7 04-26-94 60 Co 19.2 pCi/L 299-E33-13 01-14-94 90 Co 1,380 pCi/L 299-E33-14 04-28-94 90 Co 34.7 pCi/L 299-E33-26 04-28-94 90 Co 775 pCi/L 699-E33-26 04-28-94 90 Co 381 pCi/L 699-49-55A 04-11-95 90 Co 381 pCi/L 699-49-57A 04-11-94 90 Co 1,525 pCi/L 699-50-53A 04-11-94 90 Co 1,690 pCi/L 699-50-54 04-21-94 90 Co 1,480 Oo pCi/L 699-52-57 04-20-94 90 Co 2,47 pCi/L 699-55-57 04-19-94 90 Co 2,100 pCi/L 699-59-58 04-21-94 90 C	699-60-60	04-21-94	1	-0.46	pCi/L
50-54b 07-28-94 60Co 19.2 pCi/L 299-E33-7 04-26-94 9°Tc 619 pCi/L 299-E33-13 01-14-94 9°Tc 1,380 pCi/L 299-E33-14 04-28-94 9°Tc 34.7 pCi/L 299-E33-26 04-28-94 9°Tc 775 pCi/L 699-49-55A 04-11-95 9°Tc 381 pCi/L 699-49-57A 04-11-94 9°Tc 1,525 pCi/L 699-50-53A 04-11-94 9°Tc 1,690 pCi/L 699-52-54 04-21-94 9°Tc 1,480° pCi/L 699-52-57 04-20-94 9°Tc 2,47 pCi/L 699-55-57 04-19-94 9°Tc 2,100 pCi/L 699-57-59 04-15-94 9°Tc 49.9 pCi/L 699-59-58 04-21-94 9°Tc 221 pCi/L 699-60-60 04-21-94 9°Tc 144 pCi/L 49-53b 07-30-94 9°Tc 4	49-53 ^b	07-30-94	⁶⁰ Co	9.7	pCi/L
299-E33-7 04-26-94 9°Tc 619 pCi/L 299-E33-13 01-14-94 9°Tc 1,380 pCi/L 299-E33-14 04-28-94 9°Tc 34.7 pCi/L 299-E33-26 04-28-94 9°Tc 775 pCi/L 699-49-55A 04-11-95 9°Tc 381 pCi/L 699-49-57A 04-11-94 9°Tc 1,525 pCi/L 699-50-53A 04-11-94 9°Tc 1,690 pCi/L 699-50-53A 06-30-94 9°Tc 1,480° pCi/L 699-52-54 04-21-94 9°Tc 4,310 pCi/L 699-52-57 04-20-94 9°Tc 2,47 pCi/L 699-55-57 04-19-94 9°Tc 2,100 pCi/L 699-57-59 04-15-94 9°Tc 49.9 pCi/L 699-59-58 04-21-94 9°Tc 221 pCi/L 699-60-60 04-21-94 9°Tc 144 pCi/L 49-54* </td <td>49-54^b</td> <td>07-26-94</td> <td></td> <td>20.1</td> <td>pCi/L</td>	49-54 ^b	07-26-94		20.1	pCi/L
299-E33-7 08-29-94 %Tc 685 pCi/L 299-E33-13 01-14-94 %Tc 1,380 pCi/L 299-E33-14 04-28-94 %Tc 34.7 pCi/L 299-E33-26 04-28-94 %Tc 775 pCi/L 699-49-55A 04-11-95 %Tc 381 pCi/L 699-49-57A 04-11-94 %Tc 1,525 pCi/L 699-50-53A 04-11-94 %Tc 1,690 pCi/L 699-50-53A 04-21-94 %Tc 1,480% pCi/L 699-52-54 04-21-94 %Tc 4,310 pCi/L 699-52-55 04-20-94 %Tc 2,47 pCi/L 699-55-57 04-19-94 %Tc 2,100 pCi/L 699-57-59 04-15-94 %Tc 49.9 pCi/L 699-59-58 04-21-94 %Tc 221 pCi/L 699-60-60 04-21-94 %Tc 144 pCi/L 49-54 07-30-94 %Tc 200	50-54 ^b	07-28-94	[∞] Co	19.2	pCi/L
08-29-94 97C 685 pCi/L 299-E33-13 01-14-94 99Tc 1,380 pCi/L 299-E33-14 04-28-94 99Tc 34.7 pCi/L 299-E33-26 04-28-94 99Tc 775 pCi/L 699-49-55A 04-11-95 99Tc 381 pCi/L 699-49-57A 04-11-94 99Tc 1,525 pCi/L 699-50-53A 04-11-94 99Tc 1,690 pCi/L 699-52-54 04-21-94 99Tc 4,310 pCi/L 699-52-57 04-20-94 99Tc 2,47 pCi/L 699-55-57 04-19-94 99Tc 2,100 pCi/L 699-57-59 04-15-94 99Tc 49.9 pCi/L 699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53b 07-30-94 99Tc 200 pCi/L 49-54b 07-26-94 99Tc 480 pCi/L	200 F22 7	04-26-94	⁹⁹ Tc	619	pCi/L
299-E33-14 04-28-94 99Tc 34.7 pCi/L 299-E33-26 04-28-94 99Tc 775 pCi/L 699-49-55A 04-11-95 99Tc 381 pCi/L 699-49-57A 04-11-94 99Tc 1,525 pCi/L 699-50-53A 04-11-94 99Tc 1,690 pCi/L 699-50-53A 04-21-94 99Tc 1,4809 pCi/L 699-52-54 04-21-94 99Tc 4,310 pCi/L 699-52-57 04-20-94 99Tc 2,47 pCi/L 699-55-57 04-19-94 99Tc 2,100 pCi/L 699-57-59 04-15-94 99Tc 49.9 pCi/L 699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53b 07-30-94 99Tc 200 pCi/L 49-54b 07-26-94 99Tc 480 pCi/L	299-E33-7	08-29-94	⁹⁹ Tc	685	pCi/L
299-E33-26 04-28-94 99Tc 775 pCi/L 699-49-55A 04-11-95 99Tc 381 pCi/L 699-49-57A 04-11-94 99Tc 1,525 pCi/L 699-50-53A 04-11-94 99Tc 1,690 pCi/L 699-50-53A 06-30-94 99Tc 1,480° pCi/L 699-52-54 04-21-94 99Tc 4,310 pCi/L 699-52-57 04-20-94 99Tc 2.47 pCi/L 699-55-57 04-19-94 99Tc 2,100 pCi/L 699-57-59 04-15-94 99Tc 49.9 pCi/L 699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53° 07-30-94 99Tc 200 pCi/L 49-54° 07-26-94 99Tc 480 pCi/L	299-E33-13	01-14-94	⁹⁹ Tc	1,380	pCi/L
699-49-55A 04-11-95 99Tc 381 pCi/L 699-49-57A 04-11-94 99Tc 1,525 pCi/L 699-50-53A 04-11-94 99Tc 1,690 pCi/L 699-52-54 04-21-94 99Tc 1,480° pCi/L 699-52-57 04-20-94 99Tc 2,47 pCi/L 699-52-57 04-19-94 99Tc 2,100 pCi/L 699-55-57 04-19-94 99Tc 49.9 pCi/L 699-57-59 04-15-94 99Tc 49.9 pCi/L 699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53b 07-30-94 99Tc 200 pCi/L 49-54b 07-26-94 99Tc 480 pCi/L	299-E33-14	04-28-94	⁹⁹ Tc	34.7	pCi/L
699-49-57A 04-11-94 9°Tc 1,525 pCi/L 699-50-53A 04-11-94 9°Tc 1,690 pCi/L 699-50-53A 06-3094 9°Tc 1,480° pCi/L 699-52-54 04-21-94 9°Tc 4,310 pCi/L 699-52-57 04-20-94 9°Tc 2.47 pCi/L 699-55-57 04-19-94 9°Tc 2,100 pCi/L 699-57-59 04-15-94 9°Tc 49.9 pCi/L 699-59-58 04-21-94 9°Tc 221 pCi/L 699-60-60 04-21-94 9°Tc 144 pCi/L 49-53° 07-30-94 9°Tc 200 pCi/L 49-54° 07-26-94 9°Tc 480 pCi/L	299-E33-26	04-28-94	⁹⁹ Tc	775	pCi/L
699-50-53A 04-11-94 9°Tc 1,690 pCi/L 699-50-53A 06-3094 9°Tc 1,480° pCi/L 699-52-54 04-21-94 9°Tc 4,310 pCi/L 699-52-57 04-20-94 9°Tc 2.47 pCi/L 699-55-57 04-19-94 9°Tc 2,100 pCi/L 699-57-59 04-15-94 9°Tc 49.9 pCi/L 699-59-58 04-21-94 9°Tc 221 pCi/L 699-60-60 04-21-94 9°Tc 144 pCi/L 49-53° 07-30-94 9°Tc 200 pCi/L 49-54° 07-26-94 9°Tc 480 pCi/L	699-49-55A	04-11-95		381	pCi/L
699-50-53A 06-3094 9°Tc 1,480° pCi/L 699-52-54 04-21-94 9°Tc 4,310 pCi/L 699-52-57 04-20-94 9°Tc 2.47 pCi/L 699-55-57 04-19-94 9°Tc 2,100 pCi/L 699-57-59 04-15-94 9°Tc 49.9 pCi/L 699-59-58 04-21-94 9°Tc 221 pCi/L 699-60-60 04-21-94 9°Tc 144 pCi/L 49-53° 07-30-94 9°Tc 200 pCi/L 49-54° 07-26-94 9°Tc 480 pCi/L	699-49-57A	04-11-94	⁹⁹ Тс	1,525	pCi/L
699-52-54 04-21-94 99Tc 4,310 pCi/L 699-52-57 04-20-94 99Tc 2.47 pCi/L 699-55-57 04-19-94 99Tc 2,100 pCi/L 699-57-59 04-15-94 99Tc 49.9 pCi/L 699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53h 07-30-94 99Tc 200 pCi/L 49-54h 07-26-94 99Tc 480 pCi/L	600 50 52 4	04-11-94	⁹⁹ Tc	1,690	pCi/L
699-52-57 04-20-94 %Tc 2.47 pCi/L 699-55-57 04-19-94 %Tc 2,100 pCi/L 699-57-59 04-15-94 %Tc 49.9 pCi/L 699-59-58 04-21-94 %Tc 221 pCi/L 699-60-60 04-21-94 %Tc 144 pCi/L 49-53* 07-30-94 %Tc 200 pCi/L 49-54* 07-26-94 %Tc 480 pCi/L	099-30-33A	06-3094	⁹⁹ Tc	1,480°	pCi/L
699-55-57 04-19-94 99Tc 2,100 pCi/L 699-57-59 04-15-94 99Tc 49.9 pCi/L 699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53h 07-30-94 99Tc 200 pCi/L 49-54h 07-26-94 99Tc 480 pCi/L	699-52-54	04-21-94		4,310	pCi/L
699-57-59 04-15-94 9°Tc 49.9 pCi/L 699-59-58 04-21-94 9°Tc 221 pCi/L 699-60-60 04-21-94 9°Tc 144 pCi/L 49-53b 07-30-94 9°Tc 200 pCi/L 49-54b 07-26-94 9°Tc 480 pCi/L	699-52-57	04-20-94	⁹⁹ Tc	2.47	pCi/L
699-59-58 04-21-94 99Tc 221 pCi/L 699-60-60 04-21-94 99Tc 144 pCi/L 49-53h 07-30-94 99Tc 200 pCi/L 49-54h 07-26-94 99Tc 480 pCi/L	699-55-57	04-19-94	⁹⁹ Tc	2,100	pCi/L
699-60-60 04-21-94 99Tc 144 pCi/L 49-53h 07-30-94 99Tc 200 pCi/L 49-54h 07-26-94 99Tc 480 pCi/L	699-57-59	04-15-94	1	49.9	pCi/L
49-53 ^h 07-30-94 99Tc 200 pCi/L 49-54 ^b 07-26-94 99Tc 480 pCi/L	699-59-58	04-21-94	⁹⁹ Tc	221	pCi/L
49-54 ^b 07-26-94 ⁹⁹ Tc 480 pCi/L	699-60-60	04-21-94		144	pCi/L
	49-53 ^ħ	07-30-94	1	200	pCi/L
50-54 ^b 07-28-94 ⁹⁹ Tc 1,380 pCi/L	49-54 ^b	07-26-94		480	pCi/L
	50-54 ^b	07-28-94	⁹⁹ Tc	1,380	pCi/L

NOTES:

^aValue used for initial resin testing. ^bSonic push groundwater samples.

Table 4-5. Sonic Push Activity Summary.

Date	Hole Number	Activity
07/19	49-53	Mobilize to site, push rod to refusal at 26 ft, push to refusal at 24 ft, run pilot casing.
07/20	49-53	Run pilot casing, push rod to refusal at 124.6 ft, plug and abandon hole.
07/21	48-52	Mobilize to site, push rod to refusal at 36 ft, run pilot casing, push rod to refusal at 168.9 ft.
07/22	48-52	Repairs to rig, plug and abandon hole.
07/23		Rig Repairs, Weekend.
07/24		Weekend.
07/25	49-54	Mobilize to site, run pilot casing.
07/26	49-54	Push rod, fish rod, push to refusal below water table at 133 ft, establish Radiation Control Area (RCA), collect groundwater sample, plug and abandon hole.
07/27	50-54	Mobilize to site, run pilot casing, fish pilot casing, run pilot casing push rods to refusal and breakage at 112 ft, plug and abandon hole
07/28	50-54	Push rods to refusal below water table at 162 ft, establish RCA, collect groundwater samples, plug and abandon hole.
07/29	49-53	Water Development Company, Bechtel Hanford, Inc. contract meeting, mobilize to site, run pilot casing.
07/30	49-53	Run Pilot casing, push rods to refusal below water table at 135.6 establish RCA, collect samples, plug and abandon hole.
07/31		Weekend.
08/01	48-52	Mobilize to site, rig repairs, run pilot casing, push rod to 37 ft.
08/02	48-52	Continue to push rod to refusal and breakage at 61 ft, Magnaflux rods available, push to refusal and breakage at 161 ft.
08/03	49-51	Plug and abandon 48-52, mobilize to 49-51, run pilot casing, Magnaflux rods used yesterday, push rod to refusal and breakage 135 ft, decide new rod needed.
08/04	49-51	Start long weekend.
08/05		Weekend.
08/06	1	Weekend.
08/07		Weekend.
08/08		Weekend.
08/09		Waiting for new rod.
08/10	49-51	Ready new rod, push rod to refusal and breakage at 131 ft (same pilot hole), plug and abandon 49-51, mobilize to 48-54, run pilot casing, push rods to breakage at 53 ft, fish rod.
08/11	48-54	Push rod to refusal and breakage at 193 ft, plug and abandon 48-5 return to 49-51, run starter casing, Magnaflux rods used for last push, push rod to suspected refusal at 134.9 ft, stop for the day.
08/12	49-51	Continue to push, refusal and breakage at 135 ft, major rig repairs required, hole plugged and abandoned on 08/13.

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Table 4-6. Measurement Results from Sonic Push.

Hole Number	Depth to Water (m bgs)	Depth to Basalt (m bgs)	Aquifer Thickness (m)	60Co (pCi/L)	%Tc (pCi/L)
49-54	38.1	40.5	2.19	ND	480
50-54	47.2	49.4	2.10	ND	1,380
49-53	39.3	41.1	1.95	ND	200

bgs = below ground surface

ND = not detected.

Table 4-7. Groundwater Data Collected Over Time at Well 699-55-57 During Production Testing.

Elapsed Time (min)	Liters pumped	⁶⁰ Co (pCi/L)	137Cs (pCi/L)	⁹⁹ Tc (pCi/L)
32	356	27.1	3.3	1,790
152	1,684	34.9	2.83	1,970
272	3,016	34.9	2.26	1.950
392	4,346	22.6	1.89	1,960
512	5,678	35.6	0.55	2,160

Table 4-8. In Situ Flow Meter Results. (2 Sheets)

***	70.70XXXX (60)	Velocit	ty (ft/d)	D'	
Well	DBWL (ft)	$\phi = 0.20\%$	$\phi = 0.25\%$	Direction	
299-E32-2	3.7	0.89	0.7	S30°E	
	7.7	1.8	1.4	S34.3°E	
Average			1.0	S24°E	
299-E33-30	4.3	0.73	0.58	S69.9°W	
	8.3	0.66	0.53	S63°W	
	12.3	0.45	0.36	S68°W	
Average			0.49	S67°W	
299-E33-31	2.9	1.7	1.3	S22.6°W	
	6.9	2.1	1.7	S18.7°E	
	8.9	0.59	0.47	S20.0°W	
Average			1.1	S2°W	
299-E33-35	2.8	0.33	0.27	S42.2°W	
	5.8	1.2	0.94	S78.4°W	
	8.8	0.23	0.18	S21.3°W	
Average			0.43	S64.5°W	
299-E33-38	4.2	1.4	1.1	S23.6°W	
	8.2	2.4	2.0	S15.6°W	
Average			1.6	S18.4°W	
299-E34-5	1.4	2.37	1.9	\$46.6°W	
299-E34-11	2.0	1.2	0.96	N51.4°E	
	3.5	1.1	0.87	N51.4°E	
Average			0.92	N51.4°E	
699-48-50	1.2	< 0.04	< 0.04	a	
	5.7	0.60	0.48	S18.6°E	
	9.7	0.04	< 0.04	(S52.7°E) ^a	
Average			0.48	S18.6°E	
699-52-54	0.9	0.04	< 0.04	(S46.5°E)a	
	1.4	< 0.04	< 0.04	a	
Average		No	valid value		
699-52-57	0.8	1.4	1.1	N2.9°E	
699-55-55	2.1	1.5	1.2	N1.0°E	
	5.1	0.99	0.79	N21.7°W	
	8.1	1.1	0.91	N4.3°E	
Average			0.95	N4.1°W	

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Table 4-8. In Situ Flow Meter Results. (2 Sheets)

Well	DDWI (64)	Velocity (ft/d)		Velocity (ft/d)		
VV E11	DBWL (ft)	$\phi = 0.20\%$	$\phi = 0.25\%$	Direction		
699-59-58	2.2	0.60	0.48	S11.2°W		
	6.3	0.77	0.61	\$35.0°W		
	9.3	1.2	0.93	\$43.9°W		
Average			0.66	\$33.5°W		
699-60-57	3.0	2.0	1.6	\$79.3°W		
	65.5	0.86	0.69	N55.6°E		
	69.5	1.6	1.2	N58.1°E		
Average			0.96 ^b	N57.2°E		

NOTES:

bOnly the two lower flow velocities and directions were used in determining the average values.

Average = average value calculated using vector addition methods. NOTE: not an arithmetic average.

DBWL = depth below water level

 ϕ = effective porosity

^aFlow velocities below 0.04 ft/day are below practical measurement capabilities of instrument. Values in parentheses are directions for velocities calculated using 0.20% porosity.

Table 4-9. Synopsis of Tracer Test 1 Events.

	Day	Hour	Flow Rate (L/min)	Volume (L)
Begin Bromide Injection	02/06/95	08:24		
End Bromide Injection	02/06/95	~ 10:25ª		
Average Bromide Injection Rate			198.7	
Bromide Injection Volume				-21,198 ^b
Bromide Injection Sample 1	02/06/95	08:39		
Bromide Injection Sample 2	02/06/95	09:35		
Bromide Injection Sample 3	02/06/95	10:05		
Begin Effluent Injection	02/06/95	14:15		
End Effluent Injection	02/11/95	04:00		
Average Effluent Injection Rate			98	
Begin Effluent Extraction	02/06/95	08:02		
End Effluent Extraction	02/11/95	04:00		
Average Extraction Rate			107	
Effluent extraction off February 7, 1995, 07:20	0 to 07:55 for g	enerator serv	ice.	····
Begin Effluent Sampling (Continue Hourly)	02/06/96	08:56		

NOTES:

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^aTime estimated from the effluent tank volume and the average injection rate. ^bVolume calculated from effluent tank dimensions and height of water in tank.

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Table 4-10. Synopsis of Tracer Test 2 Events.

	Day	Hour	Flow Rate (L/min)	Volume (L)
Begin Bromide Injection	05/21/95	08:36		
End Bromide Injection	05/21/95	~ 10:30ª		
Average Bromide Injection Rate			190	
Bromide Injection Volume				23,352 ^h
Bromide Injection Sample 1	05/21/95	08:40		
Bromide Injection Sample 2	05/21/95	09:10		
Bromide Injection Sample 3	05/21/95	09:40		
Bromide Injection Sample 4	05/21/95	10:10		
Begin Effluent Injection	05/21/95	10:30		
End Effluent Injection	05/26/95	11:00		
Average Effluent Injection Rate			83	
Begin Effluent Extraction	05/21/95	08:02		
End Effluent Extraction	05/26/95	11:00		
Average Extraction Rate			107	
Effluent extraction off May 23, 1995, 07:27 to	07:47 for gene	rator service.		
Begin Effluent Sampling (Continue Hourly)	05/21/96	08:46		

NOTES:

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^aTime estimated from the effluent tank volume and the average injection rate. ^bVolume calculated from effluent tank dimensions and height of water in tank.

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5.0 ASSESSMENT OF 200-BP-5 PLUMES

In addition to reporting the results of the pilot-scale treatability tests, a major objective of the 200-BP-5 Treatability Test Report as discussed briefly in Section 2.6.8 is to present assessments of the three major plumes in the 200-BP-5 Operable Unit. To achieve this objective, Section 5.1 presents background data on the processes and plants that were the source of the groundwater contamination. Section 5.2 provides details of the history of changes in the water-table level and groundwater chemical concentrations because of the Hanford Site operations. The comprehensive plume-specific data are then used in Section 5.3 to develop interpretations relating to the contaminant plumes originating from the B-5 Reverse Well, the BY-Cribs, and the Gable Mountain Pond. For each contaminant plume, the interaction and interrelationship of Hanford Site process operations, the chemistry of the contaminant constituents, and the local geology and hydrology are evaluated to determine the most probable models of contaminant behavior and transport in the subsurface environment. This information is then incorporated in Section 5.4 into determinations of the risk to human health presented by each plume. Data uncertainties are evaluated and data needs are identified in Section 5.5.

5.1 OPERATIONAL HISTORY

The Hanford Site, established in 1943, was originally designed, built, and operated to produce plutonium for nuclear weapons using production reactors and chemical reprocessing plants. In March 1943, construction began on three reactor facilities (B, D, and F Reactors) in the 100 Areas and three chemical processing facilities (B, T, and U Plants) in the 200 Areas. After World War II, six more reactors were built (H, DR, KW, KE, and N Reactors). Shutdown of the reactors began in 1964. By 1971, eight of the nine reactors were shut down. Shutdown of N Reactor began in October 1991.

Operations in the 200 Areas (200 East and 200 West Areas) are related to separation of plutonium and uranium from irradiated nuclear fuel. Irradiated nuclear fuel from the reactors was stored in facilities in the 200 North Aggregate Area before transfer to separation facilities in the 200 East and 200 West Areas. Each separation facility consisted of a canyon building that contained most of the chemical processing systems, tank farms, and ancillary process control buildings, many with their own waste streams. The 200 Areas also contain nonradioactive support facilities, including transportation maintenance buildings, service stations, coal-fired powerhouses for process steam production, steam transmission lines, raw water treatment plants, water-storage tanks, electrical maintenance facilities, and subsurface sewage disposal systems.

Highly radioactive waste streams from the separations process were predominantly discharged to tank farms; moderate-activity wastes were discharged to the soil column using cribs, trenches, French drains, and reverse wells; and low-activity waste streams were predominantly discharged to the soil column using ponds and ditches.

5.1.1 200 Area Facility Operational Histories

The 200 East Area has one active processing area, B Plant, and two inactive processing areas [the PUREX Plant and Semi-Works (C Plant)] (Figure 5-1). The 200 North Aggregate Area is north of and approximately midway between the 200 East and 200 West Areas. The 200 West Area has four inactive processing areas: S Plant, T Plant, U Plant, and Z Plant (Figure 5-2). The following sections summarize operations and waste disposal information for the 200 East and 200 West Area separation facilities and the 200 North Aggregate Area facilities.

5.1.1.1 B Plant Aggregate Area. The major processes at the 200 East B Plant Aggregate Area involved extraction of plutonium from irradiated uranium fuel; purification, precipitation, and encapsulation of cesium and strontium from PUREX-derived waste streams; various waste-handling processes such as evaporation; and transfer of single-shell tank waste.

The 221-B Building is one of the main B Plant facilities. It began operation in 1945, separating plutonium from irradiated nuclear fuel by the bismuth phosphate (BiPO₄) process. The BiPO₄ process produced a dilute plutonium nitrate solution that was then purified and concentrated in the 224-B Building using a lanthanum fluoride process. The concentrated plutonium nitrate solution was then transferred to the Z Plant Aggregate Area for further refinement. Plutonium separation operations at B Plant ceased in 1952. Various waste treatment operations began at the 221-B Building in 1965, such as ¹³⁷Cs and ⁹⁰Sr recovery from 216-BY Tank Farm liquids and from the U Plant Uranium Recovery Project waste stream. Several additions to the 221-B Building, such as the 225-B Waste Encapsulation and Storage Facility (WESF) and the 221 Cask Transfer Facility, were constructed to support waste treatment operations.

Some B Plant tank wastes were transferred to the U Plant area for uranium recovery between 1952 and 1956. The wastes were then returned to the B Plant area and disposed to the ground using the BY-Cribs. Waste evaporators and ITS units have been used in the 241-B, 241-BX, and 241-BY Tank Farms to minimize the volume of waste in the tanks. Condensate from ITS B-5 Reverse Well and BY-Cribs was discharged to the 216-B-50 and 216-B-57 Cribs, respectively.

5.1.1.2 PUREX Plant Aggregate Area. Recovery of uranium and plutonium from irradiated nuclear fuel was the principal activity at the 200 East PUREX Plant Aggregate Area. In the PUREX process, irradiated uranium fuel was dissolved in a nitric acid solution, and then the uranium and plutonium were separated from the solution by a solvent extraction process using tributyl phosphate (TBP) in normal paraffin hydrocarbon solvent. The 202-A Canyon Building, the main facility in the PUREX Plant Aggregate Area, operated from 1955 until 1972 and was shut down from 1972 until 1983. Operations at the 202-A Canyon Building resumed in 1982 and were terminated in December 1992. Other facilities active in the PUREX Plant Aggregate Area include the 293-A Building, the 204-AR Waste Unloading Station, and the 242-A and 243-A Evaporators. The 242-A Evaporator started in 1977 and continues to treat wastes stored in tanks. The 242-A Evaporator steam condensate and cooling water continues to be discharged to the 216-B-3C Pond in the near term.

The PUREX Plant Aggregate Area contains eight tank farms. The 241-A, 241-AX, and 241-C Tank Farms are inactive and have undergone initial stabilization. The 241-AN, 241-AP, 241-AW, 241-AY, and 241-AZ Tank Farms are active.

5.1.1.3 Semi-Works Aggregate Area. The 201-C Process Building and the Critical Mass Laboratory (209-E Building) are the principal facilities in the 200 East Semi-Works Aggregate Area.

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The 201-C Process Building was constructed in 1949 as a pilot plant for separation of uranium and plutonium from irradiated nuclear fuel using the reduction-oxidation (REDOX) process. The 201-C Process Building was converted in 1954 to a pilot plant for the PUREX process and functioned in this capacity until 1956, when operations were terminated. In 1961, the 201-C Process Building was converted to recover strontium from fission product waste and also to recover cesium, technetium, and promethium. This facility operated until 1967. Decommissioning work began in 1983, and more than 49,210,200 L (30,000,000 gal) was discharged to the ground during operations.

The Critical Mass Laboratory (209-E Building) in which criticality experiments and research were conducted operated from 1960 to 1983.

- **5.1.1.4 200 North Aggregate Area**. Irradiated nuclear fuel from the plutonium production reactors in the 100 Areas was stored in the 200 North Aggregate Area to allow decay of short-lived radionuclides. The 212-N, 212-P, and 212-R Buildings were constructed in 1944 and used until 1952 for this purpose. Large quantities of water were pumped from wells 699-55-60A and 699-55-60B to cool the stored fuel. The cooling water was then discharged to the 216-N-1, 216-N-4, and 216-N-6 Ponds. The 212-N, 212-P, and 212-R Buildings have been used for other purposes since 1952.
- 5.1.1.5 S Plant Aggregate Area. S Plant was constructed in the 200 West Area between May 1950 and August 1951 to separate plutonium and uranium from irradiated nuclear fuel using the REDOX chemical separation process. In the REDOX process, irradiated uranium fuel was dissolved in a nitric acid solution, and then the uranium and plutonium were separated from the solution by methyl isobutyl ketone solvent extraction. The plant operated from 1951 until it was shut down in July 1967. The major waste streams from S Plant were dissolution waste, aqueous process waste, and organic process wastes that were discharged to tanks and cribs. Also, during the period 1951 through 1991, a total of 59,700,000,000 L (15,800,000,000) gal of cooling water was discharged to six ponds: 216-S-10P, 216-S-11, 216-S-15, 216-S-16P, 216-S-17, and 216-S-19; and two associated ditches: 216-S-10D and 216-S-16D (DOE-RL 1993b). The ponds and ditches are situated southwest and west of the plant.
- 5.1.1.6 T Plant Aggregate Area. T Plant was built in the 200 West Area in 1944. It was one of the first plutonium separation facilities at the Hanford Site and used the same BiPO₄ process used by B Plant to separate plutonium from irradiated nuclear fuel in the 221-T Building. The dilute plutonium nitrate solution was purified and concentrated in the 224-T Bulk Reduction Building and then sent to the Z Plant Aggregate Area for further refinement. Operations at the 221-T and 224-T Buildings ceased in 1956. Primary waste streams from the 221-T and 224-T Buildings included process waste and aqueous process waste that was discharged to tanks, cribs, and trenches. Decontamination water was discharged to a crib. Also, a total of 60,945,000,000 L (16,100,000,000 gal) of cooling water was discharged to 216-T-4 Ponds, 610 m (2,000 ft) west of T Plant, from 1944 through 1976 (DOE-RL 1993b).
- The 221-T Building, presently known as the Containment Systems Testing Facility, was used for a series of testing programs from 1964 to 1990. Activities in the 221-T Building now include radioactive decontamination, reclamation, and decommissioning of process equipment.
- 5.1.1.7 U Plant Aggregate Area. U Plant was constructed in the 200 West Area in 1944 to separate plutonium from irradiated nuclear fuel using the BiPO₄ process. However, the process was never used at U Plant because B Plant and T Plant were sufficient to meet plutonium production requirements. The 221-U Building was converted for the Uranium Recovery Project when existing

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tank space became limited. Recovery of uranium from the BiPO₄ waste took place in the 221-U Building between 1952 and 1958 using a TBP process. The 221-U Building went on standby in 1958 and has not been used for uranium separation since that time. Primary waste streams from the 221-U Building include process waste and water from uranium recovery that was discharged to cribs, French drains, ponds, and ditches, and spent solvents and carbonate scrub solution from solvent treatment that was discharged to cribs. From 1944 through 1985, a total of 165,000,000 L (43,600,000,000 gal) of cooling water was discharged to the 216-U-10 Pond (DOE-RL 1993b).

The 224-U Building began operation in 1952 as a uranium reduction facility and was converted in 1955 to support PUREX Plant activities. A stabilization run was completed in 1994. Primary waste streams from the 224-U Building include process waste and water from the UO₃ conversion that was discharged to the 216-U-10 Pond, cribs, and ditches.

5.1.1.8 Z Plant Aggregate Area. Operations at Z Plant in the 200 West Area began in 1945 at the 231-Z Plutonium Isolation Facility (PIF), which concentrated plutonium nitrate solution produced by the separation facilities and converted the concentrate to a plutonium nitrate paste for shipment to Los Alamos, New Mexico, for further refinement. This operation took place from 1945 to 1949. Primary waste streams from the PIF included process waste and water that was discharged to a trench, crib, and reverse well.

In 1949, Z Plant was converted to produce plutonium metal and/or plutonium oxide from plutonium nitrate solutions produced in the separations plants. This Z Plant complex, also known as the PFP, operated from 1949 to 1973 and intermittently from 1985 to 1988. Primary waste streams from the PFP included process waste and waters that were discharged to cribs, tanks, ponds, ditches, and seepage basins. The RECUPLEX Facility (234-5Z Building) and the Plutonium Reclamation Facility (PRF) (236-Z Building) also operated in Z Plant to recover plutonium from the PFP liquid waste stream. The RECUPLEX Facility began operation in 1955 and shut down in 1962 following a criticality event. The PRF operated from 1964 to 1979 and from 1984 to 1987. The primary waste streams from the RECUPLEX Facility included aqueous process waste, organic solvent waste, and spent silica gel that were discharged to a ditch, pond, trench, and French drain. The primary waste streams from PRF were aqueous process waste and organic process waste that was discharged to cribs and tile fields. A process line also operated in the 242-Z Building from 1949 to 1959 and from 1964 to 1976 to recover americium for the PFP waste stream. This line was shut down in 1976 after an explosion occurred in one of the americium recovery units. The primary waste stream was spent ion-exchange resin that discharged to ditches and a pond.

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5.1.2 200-BP-5 Contaminant and Effluent Sources

Operational details and effluent disposal histories for the B-5 Reverse Well, BY-Cribs, the 216-B-3 Pond System, and the 216-A-25 Gable Mountain Pond are presented below. The B-5 Reverse Well and BY-Cribs are discussed because they were sources of the contaminants of interest to the treatability test. The 216-B-3 Pond system is discussed because discharges of large volumes of cooling water at the system modified the groundwater flow in and from the 200 East Area, and probably influenced migration of contaminants from the B-5 Reverse Well and BY-Cribs. The 216-A-25 Gable Mountain Pond is discussed because discharges of large volumes of cooling water at this pond modified the groundwater flow from the 200 East Area and because the pond is the source of a 90Sr plume.

5.1.2.1 216-B-5 Reverse Well. The B-5 Reverse Well is an inactive, 92-m- (302-ft) deep, 20-cm- (8-in.) diameter (Brown and Ruppert 1950) waste management unit located approximately 305 m (1,000 ft) northeast of the 221-B, 221-T, and 221-U Buildings and east of Baltimore Road. It received overflow waste from the 241-B-361 Settling Tank, which received waste from the 224-B Concentration Facility and from Tank 5-6 in the 221-B Building from April 1945 until September 1946. Between September 1946 and October 1947, cell drainage and other liquid waste from Tank 5-6 was injected into the well (WHC 1991, Brown et al. 1990). Approximately 3.10 x 10⁷ L of liquid was discharged to the 216-B-361 Settling Tank from the 224-B Concentration Facility and the 221-B Building, containing an estimated 4.28 x 10³ g of plutonium and 3.80 x 10³ Ci of mostly short-lived beta-gamma activity (Brown and Ruppert 1950). The waste stream contained an estimated 76 Ci of ⁹⁰Sr, 106 Ci of ¹⁰⁶Ru, and 81 Ci of ¹³⁷Cs (Maxfield 1979). Dates of operation, volumes of waste received, and radiological and chemical inventories for the B-5 Reverse Well are included in Tables 5-1, 5-2, and 5-3.

In 1947, the water-table elevation in well 299-E33-18 at the 241-BY Tank Farms demonstrated that the reverse well penetrated approximately 3 m (10 ft) into the groundwater, and that radioactive waste was being discharged directly into the groundwater. The B-5 Reverse Well was deactivated on September 27, 1947, and the Tank 5-6 wastes were rerouted to the 216-B-7A and 216-B-7B Cribs (Maxfield 1979). In November 1947, drilling began on the first of 11 wells to investigate groundwater contamination associated with B-5 Reverse Well. The last well was completed in May 1948. The drilling and regular groundwater sampling defined a broad plume of short-lived beta emitters (106 Ru) and a smaller plume of alpha emitters (Brown and Ruppert 1950).

5.1.2.2 216-BY Cribs. The BY-Cribs consist of nine inactive cribs, 216-B-43 through 216-B-50, and 216-B-57 in the 200-BP-1 Operable Unit (Figure 5-3). The 216-B-43 through 21-B-50 Cribs are located adjacent to the northern boundary of the 241-BY Tank Farm. The 216-B-57 Crib is located next to the northwest corner of the 241-BY Tank Farm and southwest of the BY-Cribs. The 216-B-43 through 216-B-50 Cribs are arranged in two north-trending lines of four cribs each; two lines run north from the 201-B Flush Tank, and individual feed pipes extend perpendicular from the central lines to individual cribs. Each crib consists of a 4-m- (15-ft) deep by 9-m (30-ft) by 9-m (30-ft) excavation with a basal layer of gravel covered by layers of gravel, sand, and soil fill. Effluent was discharged by vertical pipes to the basal gravel layer and then infiltrated into the underlying vadose zone. The 216-B-57 Crib is a 61-m- (200-ft) long, 3-m- (10-ft) deep, and 4-m- (15-ft) wide (at the bottom) excavation in which a 30-cm (12-in.) in which a 30-cm- (12-in.) diameter perforated pipe runs lengthwise. The bottom 1.2 m (4 ft) of the excavation was gravel filled, and the remaining 1.8 m (6 ft) was backfilled with soil to original ground level. The *B Plant Source*

Aggregate Area Management Study Report (DOE-RL 1993c) includes construction details and as-built drawings in plan- and cross-section views for the cribs.

The 216-B-43 through 216-B-49 Cribs received scavenged TBP supernatant waste from the 221-U Building and the 241-BY tanks. TBP acid waste from the 221-U Building was made alkaline for transport and sent to the 241-BY Tank Farm, where it was treated with potassium ferrocyanide as a cesium scavenger. The supernatant from the tanks was allowed to cascade from tank to tank to allow precipitation of cesium and was then discharged to the cribs (DOE-RL 1990, Jungfleisch 1984). This supernatant effluent contained large concentrations of radionuclides including ¹³⁷Cs, ⁶⁰Co, ¹⁰⁶Ru, ⁹⁰Sr, plutonium, and uranium (Maxfield 1979, Brown et al. 1990, WHC 1991, DOE-RL 1992). Inorganic compounds in the effluent included ferrocyanide, nitrate, phosphate, sodium, and sulfate-based compounds. It was intended that discharge at each crib would be stopped before the specific retention capacity of the vadose zone for radionuclides was exceeded. Discharges to these cribs began in November 1954 at the 216-B-43 Crib and ended in December 1955 at the 216-B-49 Crib. These seven cribs received a total of 34,000,000 L (9,034,000 gal) of TBP supernatant. Discharge of the TBP supernatant was halted in 1956 before use of the 216-B-50 Crib could begin, in 1956, when groundwater concentrations of ⁶⁰Co from a monitoring well adjacent to the cribs exceeded the discharge limit of 40,000 pCi/L by a factor of more than 100 (Thomas et al. 1956). Dates of operation, volumes of waste received, and radiological and chemical inventories for the cribs are included in Tables 5-1, 5-2, and 5-3.

The 216-B-50 and 216-B-57 Cribs received low-level radioactive condensate generated by the ITS B-5 Reverse Well and BY-Cribs, respectively. The ITS units operated in the 241-BY Tank Farm and reduced the volume of liquid in the single-shell waste storage tanks. Radionuclides in the ITS condensate included ¹³⁷Cs, ¹⁰⁶Ru, ⁹⁰Sr, plutonium, and uranium (WHC 1991, Brown et al. 1990). Dates of operation, volumes of waste received, and radiological and chemical inventories for the cribs are presented in Tables 5-1, 5-2, and 5-3. The 216-B-50 and 216-B-57 Cribs received a total of 139,300,000 L (36,800,000 gal) from January 1965 through January 1974.

Stabilization of the BY-Crib area began in 1975 and was completed in November 1977 (Maxfield 1979). In 1991, interim stabilization actions under the *Comprehensive Environmental Response, Compensation, and Liability Act* were completed at the 216-B-43 to 216-B-50 Cribs and the 216-B-57 Crib. The interim stabilization was performed to eliminate surface contamination and migration deficiencies, to maintain environmental compliance in preparation for remedial investigation/feasibility study activities, and to maintain environmental compliance until the final remediation strategy is implemented.

The Phase I remedial investigation field activities at the 200-BP-1 Operable Unit began in 1990 and were completed in 1993, and included the BY-Cribs as part of the source and vadose zone investigations. Crib materials and the underlying unsaturated sediments were sampled and analyzed to determine contamination levels (DOE-RL 1993d). Column leach testing of materials collected during the source and vadose zone sampling was performed to assess potential contaminant fluxes that might be generated by infiltration of water through the vadose zone (DOE-RL 1993d). The Phase I remedial investigation of the 200-BP-1 Operable Unit included the following hydrologic and geologic investigation activities:

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- A seismic refraction survey
- Groundwater monitoring well installation
- Groundwater sampling and analysis

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- Aguifer testing
- Sorption testing.

Results of these investigation activities and other 200-BP-1 Operable Unit Phase I remedial investigation activities are presented in the *Phase I Remedial Investigation Report for the 200-BP-1 Operable Unit* (DOE-RL 1993d).

5.1.2.3 216-B-3 Pond System. The 216-B-3 Pond System consists of the 216-B-3 Main Pond and the 216-B-3A, 216-B-3B, and 216-B-3C Expansion Ponds. This pond system is a RCRA facility located approximately 1,100 m (3,500 ft) east of the 200 East Area perimeter fence and approximately 1,500 m (5,000 ft) northeast of the 202-A Building, as shown in Figure 5-3. The 216-B-3 Pond System received an estimated total waste volume of 261,000,000,000 L (68,901,000,000 gal) between 1945 and 1995 (WHC 1991, DOE-RL 1994). The 216-B-3C Pond continues to receive effluent. Dates of operation, volumes of waste received, and radiological and chemical inventories for the 216-B-3 Ponds are included in Tables 5-1, 5-2, and 5-3. Figure 5-4 shows the estimated annual effluent discharges in liters from all sources inside the 200 East Area, and from the 216-B-3 Ponds and the 216-A-25 Gable Mountain Pond for the period 1945 through 1994. Discharges from the 216-B-3 Ponds and the 216-A-25 Gable Mountain Pond locally raised the water-table elevations in the unconfined aquifer under the 200 East Area, modified the groundwater flow directions, and may have influenced the movement of the BY-Cribs and B-5 Reverse Well plumes. Groundwater flow directions are illustrated by six water-table and groundwater flow maps presented in the 200 East Groundwater Aggregate Area Management Study Report (DOE-RL 1993a) for the period 1944 through 1991.

B Plant and PUREX Plant were the main sources of effluent discharged to the 216-B-3 Pond System. The 283-E Water Treatment Facility, 284-E Powerhouse, and Semi-Works also contribute, but to a lesser extent. Effluent was transported to the pond using a series of unlined ditches, which also increased the area available for infiltration of effluent, and through buried pipelines. The ditches and pipelines are shown in Figure 5-5.

The 216-B-2-1, 216-B-2-2, and 216-B-2-3 Ditches transported effluent from sources including B Plant, 284-E Powerhouse, and Semi-Works to the buried 216-B-2-3 corrugated metal pipeline (Maxfield 1979), which conveyed the effluent to the 216-B-3-1, 216-B-3-2, and 216-B-3-3 Ditches feeding the 216-B-3 Pond. The 216-B-2-1 Ditch was used from April 1945 until November 1963; the 216-B-2-2 Ditch, from November 1963 until May 1970; and the 216-3-3 Ditch, from April 1970 until 1987. The 216-B-3-1 Ditch was used from April 1945 until July 1964; the 216-B-3-2 Ditch, from July 1964 until September 1970; and the 216-3-3 Ditch, from September 1970 until October 1994. The 216-B-2-1, 216-B-2-2, 216-B-3-1, and 216-B-3-2 Ditches were removed from service after they were contaminated by unplanned releases of fission products at the PUREX Plant and B Plant Canyon Buildings. The 216-B-2-3 Ditch was replaced by a buried polyethylene pipeline in 1987, and the 216-B-3-3 Ditch was removed from service when the 216-B-3 Pond was shut down.

Waste streams from B Plant consisted of 221-B Building steam condensate, process cooling water, and chemical sewer effluent; 241-BY ITS B-5 Reverse Well and BY-Cribs condenser cooling water; effluent from cleanup of the 207-B Retention Basin; and the WESF cooling water (Stenner et al. 1988). 284-E Powerhouse contributed water, and 241-CR Vault cooling water came from the Semi-Works (Stenner et al. 1988).

Effluent from the PUREX Plant discharged into the western end of the 216-B-3-3 Ditch via the PUREX Cooling Water Line and the 216-A-29 Ditch. The PUREX Cooling Water Line effluent included 202-A Building cooling water, process waste, steam condensate, air sampling vacuum pump seal cooling water, chemical sewer effluent, and acid fractionator condensate; 241-A Ventilation System Complex cooling water; 242-A Evaporator steam condensate and cooling water; 244-AR Vault cooling water; 283-E Water Treatment Facility overflow water; and 284-E Powerhouse water (DOE-RL 1993e, Maxfield 1979, Stenner et al. 1988, WHC 1991). The effluent contained low levels of radionuclides including ²⁴¹Am, ¹³⁷Cs, ³H, ¹⁴⁷Pm, ¹⁰⁶Ru, ⁹⁰Sr, and plutonium (Brown et al. 1990, WHC 1991). The 216-A-29 Ditch operated from November 1955 until July 1991. It conveyed 202-A Building chemical sewer effluent, cooling water, and air-sampling vacuum pump effluent to the 216-B-3 Pond via the 216-B-3-3 Ditch (DOE-RL 1993e).

The PUREX Plant steam condensate and cooling water waste streams contributed large volumes of effluent to the pond. This effluent was primarily river water with little potential for chemical or radioactive contamination. Although releases into these waste streams occurred, they represent a small fraction of the waste volume discharged to the pond. Operational records from the 1980's for the B Pond system show effluent discharge rates of 49,210 to 56,781 L/min (13,000 to 15,000 gal/min).

5.1.2.3.1 216-B-3 Main Pond. The 216-B-3 Main Pond began receiving effluent in 1945 following construction of an earthen dam that impounded a natural topographic depression. The pond received effluent until 1994 when it was closed. During the 1970's, an area west of the 216-B-3 Main Pond was diked to provide a 1.7-ha (4.1-acre) overflow area adjacent to the pond.

During its 49 years of operation, the area of the 216-B-3 Main Pond ranged from 3.6 to 18.6 ha (19 to 46 acres). In 1994, the pond covered approximately 13 ha (32 acres) and had a maximum depth of 13 ft (Smith 1993). The 1.7-ha (4.1-acre) overflow area west of the 216-B-3 Main Pond was decommissioned and backfilled in 1985. Discharge of effluent to the 216-B-3 Main Pond ceased in early 1994 and was followed by interim stabilization actions that were completed in October 1994 (BHI 1995a). These actions also included the 216-B-3-3 Ditch and the 216-B-3A Expansion Pond.

5.1.2.3.2 216-B-3A Expansion Pond. The 216-B-3A and 216-B-3B Expansion Ponds were built in 1983 to handle the increased discharge flow rate resulting from the restart of the 202-A Building (PUREX Plant) operations. In January 1984, the dike separating the 216-B-3A and 216-B-3B Expansion Ponds breached at the channel connecting the two ponds. The dike was repaired, a slide gate was installed, and a north-south-trending ditch, 1.8 m (6 ft) deep, 6 m (20 ft) wide, and approximately 244 m (800 ft) long, was excavated into the bottom of the 216-B-3A Expansion Pond to improve percolation. The ponds were reopened for use in 1984. The trench increased the maximum depth of the pond to approximately 1.8 m (6 ft). The 216-B-3A Expansion Pond remained in service until 1994 when interim stabilization was completed (BHI 1995a).

The 216-B-3A Expansion Pond covered approximately 4.5 ha (11 acres) and was originally approximately 0.6 to 0.9 m (2 to 3 ft) deep. The surface elevation of this pond was approximately 5.5 m (18 ft) lower than the 216-B-3 Pond. The pond had a very low infiltration rate.

5.1.2.3.3 216-B-3B Expansion Pond. This pond was returned to service in June 1984, after the dike repair. It is a roughly rectangular, 4.5-ha (11-acre) excavation that is currently dry and has not been used since 1985. It was dredged in 1986.

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- 5.1.2.3.4 216-B-3C Expansion Pond. This pond has been active since its construction in 1985. The 216-B-3C Expansion Pond was built to handle the increased discharge to the 216-B-3 Pond System that arose from the decommissioning of the 216-A-25 Gable Mountain Pond. The 216-B-3C Expansion Pond is an excavated structure that does not use embankments to impound effluent. The 216-B-3C Expansion Pond has the lowest elevation of the 216-B-3 Ponds. The 216-B-3C Expansion Pond was excavated in a coarse gravel layer to increase the infiltration rate. Within the roughly rectangular depression of the pond, a series of distribution channels run the length of the pond bottom.
- 5.1.2.4 216-A-25 Gable Mountain Pond. The 216-A-25 Gable Mountain Pond, the largest of the Hanford Site pond-type effluent disposal facilities, was a 29-ha (71-acre) natural depression located 1.6 km (1 mi) south of the west end of Gable Mountain. An estimated total 307,000,000,000 L (81,100,000,000 gal) of effluent, principally from the 202-A Building, was discharged to the 216-A-25 Pond between 1957 and 1987 (Coony and Thomas 1989) as shown in Figure 5-4. Dates of operation and volumes of waste received by the 216-A-25 Gable Mountain Pond are shown in Table 5-1. Discharges to the 216-A-25 Pond locally raised the water-table elevation north of the 200 East Area, modified the groundwater flow directions, and may have influenced the movement of the BY-Cribs and B-5 Reverse Well plumes.

Waste reached Gable Mountain Pond through the PUREX Cooling Water Line. This corrugated metal pipeline was broken and capped when the Gable Mountain Pond was decommissioned in 1987. Effluent discharged to the 216-A-25 Gable Mountain Pond by the PUREX Cooling Water Line is discussed in Section 5.1.2.3. Although the 216-A-25 Gable Mountain Pond received low levels of chemically and radioactively contaminated wastes since its startup, a single unplanned release (UPR-200-E-34) on June 11, 1964 caused the discharge of about 10,000 Ci of short- and long-lived mixed fission products to this pond, as well as to the 216-B-3 Pond, and 216-B-3-1 Ditch. Approximately 7,500 Ci of this release went to the Gable Mountain Pond. Actions were taken in 1964 at Gable Mountain Pond to prevent the spread of contamination, to reduce exposure by increasing water depth above the contaminated sediments, and to prevent contamination of migratory wild fowl. The original contaminated pond inlet was replaced with a new inlet (Maxfield 1979)

Cleanup stabilization actions started in July 1984 and were completed in December 1988. The unit was backfilled with clean pit run soil and cobbles to a minimum of 61 cm (2 ft) above the original shoreline (Hayward 1989). The pond was revegetated after the entire backfilled area was covered with 30 cm (1 ft) of topsoil. Wells 699-53-47, 699-55-50C, and 699-52-52 monitor the groundwater beneath the site of the backfilled pond (WHC 1991). The estimated radionuclide inventory in 1995 for the 216-A-25 Gable Mountain Pond, decayed from the data presented in Table 5-2, is 1.66 x 10² Ci of ¹³⁷Cs, 1.28 x 10² Ci of ³H, 3.37 x 10⁻⁷ Ci of ¹⁰⁶Ru, 2.06 x 10² Ci of ⁹⁰Sr, 428 g of plutonium, 4.24 Ci of uranium, and 5.21 x 10⁻⁴ Ci of ²⁴¹Am.

5.1.3 200 West Area Groundwater Movement

Three pond systems were used in the 200 West Area to dispose of large volumes of liquid effluent, consisting predominantly of cooling water, from 200 West Area separations and processing plants (i.e., S Plant, T Plant, and U Plant). These pond systems operated from 1944 until 1985 and disposed of 265,000,000,000 L (70,100,000,000) gal of effluent. Figure 5-5 shows the estimated annual effluent discharges in liters to the S Ponds, T Pond, and U Pond for 1944 through 1993. The largest effluent discharges occurred between 1952 and 1958, and the maximum for the entire 49 years

occurred in 1953 (Figure 5-6). In later years, the second and third largest volumes of effluent discharge occurred between 1960 and 1965 and between 1974 and 1980. From 1944 through 1951, the major effluent discharges occurred at T Pond. Between 1952 and 1959, and between 1966 and 1984, the majority of effluent disposal occurred at U Pond. The S Ponds received most of the effluent from 1959 through 1965 and since 1984 have been active. Figure 5-7 shows the estimated annual effluent discharges in liters from all sources in S Plant, T Plant, U Plant, and Z Plant for 1944 through 1993.

This disposal of effluent raised the elevation of groundwater in the unconfined aquifer beneath the 200 West Area, creating a groundwater mound that changed groundwater flow in the unconfined aquifer (DOE-RL 1993b). The elevation of groundwater increased approximately 14 m (45 ft) by 1951, relative to the 1944 elevation, and by 1955 the groundwater elevation was 23 m (75 ft) higher than in 1944 (DOE-RL 1993b). Through 1987, the groundwater elevation was 17 to 18 m (55 to 60 ft) higher than in 1944 (DOE-RL 1993b). The apex of the mound was centered over the U Plant area until 1955; by 1965, it had shifted south approximately 3 km (1.9 mi) to an area near the S Ponds. By 1973, it had moved north to the U Pond area (DOE-RL 1993b). Because of the decommissioning of the 216-U-10 Pond in 1984, the mound has fallen significantly; groundwater elevations decreased by 3 m (10 ft) between 1985 and 1990 near the pond (DOE-RL 1993b).

Before the 200 West Area effluent discharges began at the Hanford Site in 1944, the general groundwater flow direction in the unconfined aquifer was from west to east, with some flow north through the gap between the Yakima Ridge and Gable Butte (DOE-RL 1993b). Groundwater flow directions are illustrated by seven water-table and groundwater flow maps presented in the 200 West Groundwater Aggregate Area Management Study Report (DOE-RL 1993b) for the period 1944 through 1987. These maps show a generally radial groundwater flow pattern resulting from the 200 West Area effluent discharges, with some groundwater flow northward through the gap between Gable Butte and Gable Mountain in addition to the flow through the gap between the Yakima Ridge and Gable Butte (DOE-RL 1993b). The groundwater flow directions have also been influenced by agricultural irrigation in the Cold Creek Valley, which is located west of the 200 West Area (DOE-RL 1993b).

5.2 HISTORICAL GEOSCIENCES INFORMATION

This section explains how the regional aquifer has been modified by local waste stream discharges and how the radionuclide and chemical plumes have migrated during the operational history of the site. Water-table distribution and plume migration have been monitored since the early days of the Hanford Site and are discussed in this section.

5.2.1 Water Levels

Artificial recharge of the unconfined aquifer from liquid effluent disposal to surface ponds, cribs, and ditches has had a profound impact on the water-table distribution beneath the 200 East Area. Approximately one-half of the liquid effluent waste disposed of to the ground in the 200 East Area was routed to two surface ponds, 216-B-3 (B Pond) and 216-A-25 (Gable Mountain Pond). The remainder was disposed of to numerous cribs and ditches (Figure 5-8).

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The effect of groundwater mounding on the water table beneath the 200 East Area is illustrated in Figure 5-9. The January 1944 map is an estimated pre-Hanford Site water-table map. The flow illustrated in the figure is generally from west to east/southeast. As stated in Section 5.1, B Pond began receiving liquid effluent waste in 1945, and the development of a groundwater mound beneath the facility is clearly evident in the March 1951 map. Disposal to the facility has continued to the present, and the mound appears in each subsequent water-table map. Gable Mountain Pond began receiving effluent wastes in 1957 and continued operations until 1985. In the June 1960 contour map the Gable Mountain Pond and B Pond groundwater mounds are interpreted to coalesce. It was not until 1980 that the two mounds were contoured as separate mounds. The Gable Mountain Pond groundwater mound quickly disappeared following closure of the facility in 1987, and it is not discernable in the June 1990 map.

The effects of water disposal on the water table in the vicinity of the areas of concern for this report are illustrated in Figure 5-10. Hydrographs for wells in the vicinity of each disposal site are illustrated, as well as near B Pond. Increased production at the fuel separation operations in the late 1950's resulted in dramatic increases in liquid effluent disposal to the two ponds and resultant elevated water tables.

Effluent disposed to ponds in the vicinity of the 200 East Area created groundwater mounds, which altered the predominantly west-to-east direction of flow that existed before Hanford Site operations. The primary effect of this mounding has been to establish a hydrologic dam on the eastern boundary of the 200 East Area, forcing flow to divert either to the north/northwest through Gable Gap or to the southeast. The increased effluent discharges in this area resulted in a groundwater divide that controls flow diversion. The divide axis is oriented approximately east-west, and its location has migrated along a northwest-to-southeast axis beneath the 200 East Area in response to varying amounts and locations of effluent discharge. Effluent and contaminants that migrated to, or flowed with, groundwater to the south of the divide were driven to the southeast. Those that entered groundwater to the north of the divide were transported to the north/northwest and eventually through Gable Gap.

Chemical contaminant information discussed in Section 5.2.2 provides the best evidence of when the hydrologic divide developed beneath the 200 East Area. Gross beta contamination from the B-5 Reverse Well and the BY-Cribs is represented as having been transported to the southeast in contaminant plumes from the mid- to late 1950's. The first good evidence of northward movement of gross beta contamination is represented in the October-December 1959 contaminant map. From that time to the present, hydrologic conditions have existed that have resulted in a continued north/northwesterly movement of groundwater beneath these disposal facilities. Contamination evidence that supports this evaluation is presented in more detail in Section 5.2.2. However, the evidence for contaminant movement for Gable Mountain Pond is not as well defined. Flow during operation was directed radially out from beneath the facility because of the development of a groundwater mound. The basalt subcrop feature at the water table restricted flow to the south, and it appears that B Pond mounding limited flow to the east. The majority of flow beneath the facility is believed to have been directed to the northwest towards Gable Gap. Once discharges to the facility ceased in late 1987, the influence of B Pond strongly dominated flow.

The recent (1993) flow regime is represented in Figure 5-11. Streamlines from the BY-Cribs Area (A), B-5 Reverse Well (B), and Gable Mountain Pond (C) all indicate that flow from beneath all three facilities is currently moving to the north/northwest through Gable Gap. As the B Pond mound dissipates in the future and the hydrologic divide disappears, flow may return to a more southerly direction.

5.2.2 Groundwater Contamination Concentrations and Plume Migration

Groundwater monitoring activities have been conducted since the earliest days of Hanford Site operations and were first reported in 1945 (Parker and Gamertsfelder 1945). A general summary of past monitoring activities, focusing on 200 Area operations, is reported in Ground-Water Contribution to Dose from Past Hanford Operations (Freshley and Thorne 1992). Over the years, sample analytical techniques, as well as analytes of interest, have been modified. Gross beta has been measured from the beginning of monitoring activities to provide indications of liquid effluent migration to groundwater from disposal facilities. The methodology for routine gross beta analysis was refined to improve the detection limit from 300 pCi/L in 1945, to 150 pCi/L during the 1950's, to 80 pCi/L in 1959, to 16 pCi/L in the mid-1980's, and to 4 pCi/L in 1991 (Freshley and Thorne 1992). In 1959, Bierschenk (1959a), reported that any groundwater sample that exceeded 100,000 pCi/L for gross beta was analyzed for 60Co, 137Cs, and 90Sr. By 1962, many samples were analyzed for uranium, 60Co, 137Cs, 106Ru, 90Sr, and tritium (Brown and Raymond 1962). A primary contaminant of concern for this treatability test report, 99Tc, was not listed for analysis. Technetium-99 analyses were not routinely performed until the mid-1980's, when it was recognized that the gross beta results did not balance well with the summation of the individual gross beta emitters. This may have been made more obvious as the activity contribution from 106Ru declined because of natural decay. The unknown contributor to the gross beta activities was suspected to be either ¹²⁹I or ⁹⁹Tc, and both were included for routine monitoring from that time to the present.

Groundwater monitoring reports as early as 1945 noted detections of contamination in individual wells around the site. The first discussion of widespread contaminant movement in groundwater appeared in a 1956 report (Parker 1956). It was also in 1956 that the first detection of groundwater contamination outside of the 200 East and 200 West Areas was reported and the distribution of gross beta contamination was mapped (Earth Science Personnel 1956). A summary containing the 1956 map and subsequent gross beta distributions maps is presented in Figure 5-12. This figure serves as a time reference for individual discussions particular to the three areas of groundwater contamination of interest in this report. The concentration value of the contour used to define the extent of the gross beta contamination was not identified in maps until the October-December 1957 map was produced (Bierschenk 1959b). It is presumed to have been equal to 150 pCi/L. The shape of the area of contamination has varied through time because of changing detection limits, interpretations of water-table distribution, increased well control, and decay of radioisotopes (primarily ¹⁰⁶Ru). However, a locus of contamination has always been associated with the B-5 Reverse Well and subsequently the BY-Cribs contamination, which remains as an area of elevated concentrations.

5.2.2.1 216-B-5 Reverse Well Groundwater Contamination. As mentioned in Section 5.1, during a 2-year period from 1945 to 1947, approximately 3.03 x 10⁷ L (8. x 10⁶ gal) of intermediate-level liquid wastes from BiPO₄ process operations at B Plant was discharged to the 241-B-361 Settling Tank. The overflow was disposed of to the B-5 Reverse Well. The settling tank was designed to remove waste solids before water disposal to the reverse well. At the time of deactivation of the system, approximately 121,000 L (32,000 gal) of sludge remained in the 132,000-L (35,000-gal) tank. The well penetrated approximately 3.7 m (12 ft) below the water table, and waste was introduced into the aquifer at this level. Descriptions of waste disposal activities and subsequent characterization work are contained in Brown and Ruppert (1950), Smith (1980), and Hodges (1995).

Operational estimates of radioactive waste inventories indicate that approximately 4.27 kg of plutonium, 81 Ci of ¹³⁷Cs, 163 Ci of ¹⁰⁶Ru, 76 Ci of ⁹⁰Sr, and 3,800 Ci of miscellaneous beta activity

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were sent to the settling tank. The quantity of uranium was not estimated. Characterization of the sludge remaining in the settling tank in 1979 (Smith 1980) indicates that the reported operational numbers underestimated the inventory discharged to the settling tank, in some cases by many orders of magnitude (e.g., 30 times the reported inventory of 90 Sr was estimated to exist in the settling tank alone).

The B-5 Reverse Well was removed from service on September 19, 1947, when a water sample from well 299-E33-18, located 655 m (2,149 ft) north of the reverse well, indicated the presence of alpha contamination in groundwater (Brown and Ruppert 1950). Subsequent water samples from the well indicated that the initial sample was in error and no groundwater contamination from B-5 Reverse Well disposal operations occurred that far to the north. The B-5 Reverse Well was never returned to service, and by the time subsequent sampling and analyses had been conducted on well 299-E33-18. the waste stream had been rerouted to the 216-7A and 216-7B Cribs, and a study of the extent of groundwater contamination had been initiated. Eleven wells were drilled from November 1947 to May 1948 to attempt to characterize groundwater contamination in the vicinity of the reverse well. The wells were drilled 9 m (29.5 ft) below the water table. No radioactive contaminants were detected in any of the sediment samples, but beta-gamma (fission product) and alpha activity were detected in the groundwater. Brown and Ruppert's (1950) radionuclide distribution estimates for the time period November 1947 to July 1949 are illustrated in Figure 5-13. The rapid decrease in the area of fission product contamination was attributed to decay of short-lived radionuclides and dispersion and dilution of the plume. The November 1947 and July 1948 depictions of the plume appear to indicate a southeasterly movement. The same apparent southeast orientation is present in the November 1947 depiction of alpha contamination.

Three groundwater wells (i.e., 299-E28-23, 299-E28-24, and 299-E28-25) were drilled in 1979 to 1980 to characterize radionuclide distributions around the reverse well. The locations of these wells relative to the reverse well are shown in Figure 5-14. Estimates of the present distribution of two fission products (138Cs and 90Sr) and one alpha emitter (plutonium), which remain as groundwater contaminants in the vicinity of the reverse well, are presented in Figures 5-15 through 5-17. Determination of the groundwater distribution of radionuclides around the reverse well is hampered both by the lack of wells away from the immediate vicinity of the reverse well and by the practice of analyzing only unfiltered samples for radionuclides. As shown in the time history plots in Figures 5-15 through 5-17, analytical results for radionuclides in wells adjacent to the reverse well are highly variable, and discerning dissolved from particulate fraction is not possible. Contaminants appear to be localized, yet 90Sr appears to have been transported to the northwest to well 299-E28-2, indicating that the present-day direction of groundwater movement has shifted nearly 180 degrees from the transport direction proposed by Brown and Ruppert (1950). Discussion of a conceptual model of groundwater contamination from the B-5 Reverse Well is presented in Section 5.3.

5.2.2.2 216-BY Cribs Groundwater Contamination. Waste disposal to the eight BY-Cribs was conducted during two time periods. A summary of waste disposal activities at the cribs and subsequent groundwater monitoring beneath the cribs is contained in DOE-RL (1993a, 1993c, 1993e) and Kasza (1993). During 1954 and 1955, approximately 3.38 x 10⁷ L (8.94 x 10⁶) gal of waste solutions from the TBP process was discharged to the soil column at seven of the eight cribs. A primary concern at the time was to mediate disposal to the cribs so as not to exceed the sorptive capacity of the soil beneath the cribs, thereby preventing the release of ¹³⁷Cs and ⁹⁰Sr to the groundwater (Rhodes et al. 1954, Stedwell 1954). Disposal to the cribs was terminated in December 1955 when beta activity, specifically ¹³⁷Cs, was detected in the groundwater under the cribs. In early 1956, revised laboratory methodology allowed for the detection of ⁶⁰Co in the

groundwater, a previously unknown contaminant in the TBP waste stream. Cobalt-60 was present in the groundwater in concentrations that were many hundreds of times higher than then-current standards allowed. Between 1965 and 1974, 6.06 x 10⁷ L (1.6 x 10⁷ gal) of tank liquor condensate from the ITS B-5 Reverse Well was discharged to the remaining BY-Crib. An additional 8.33 x 10⁷ L (2.2 x 10⁷ gal) of tank liquor condensate from ITS BY-Cribs was discharged to the nearby 216-B-57 Crib between 1968 and 1973. Estimated quantities of the primary contaminants in the liquid effluent streams disposed of to the cribs include 0.45 Ci of ⁶⁰Co, 18,900 kg (41,667 lb) of ferrocyanide, 5.68 x 10⁶ kg (1.25 x 10⁷ lb) of nitrate, and an unknown quantity of ⁹⁹Tc.

Groundwater contamination emanating from beneath the crib sites is discussed in Parker (1956) as a primary source for the southeast-trending gross beta contaminant plume illustrated in the 1956 map in Figure 5-12. The contour level is not specified in the 1956 map, but is presumed to be 150 pCi/L. A chart containing the gross beta time history at well 299-E33-4, which monitors the upper approximate 3.1 m (10 ft) of the aquifer, is presented in Figure 5-18. In April 1956, a gross beta result of 9.8 x 108 pCi/L is reported for the well. With decay of the shorter-lived fission products, the gross beta concentrations in the well dropped 1,000-fold by September 1959 (see Figure 5-12). The growing influence of groundwater mounds beneath the B Pond and Gable Mountain Pond on the hydrologic flow regime during the late 1950's can be discerned from the contaminant distributions illustrated in Figure 5-12. By late 1956 and early 1957, the gross beta contaminant plume beneath the 200 East Area had been displaced to the west because of the increased disposal to B Pond. The startup of disposal to Gable Mountain Pond to the northeast may have created a slight damming effect to the north, but its effect on contaminant movement from the BY-Cribs may have been moderated by an intervening basalt high, which extends above the water table. By 1959, elevated gross beta contamination was present in well 699-50-53A (Figure 5-19 and the October-December 1959 map in Figure 5-12) indicating northward movement of the plume. Well 699-50-53A has remained a center of contamination for BY-Crib-sourced contamination to the present.

The present-day distribution of cyanide and two beta emitters (60Co and 99Tc) is presented in Figures 5-20 through 5-22. A similar distribution for all three contaminants can be recognized from review of the three figures. The highest contaminant concentrations are centered on wells 699-50-53A and 699-52-54. Of the two wells, 699-52-54 is showing increasing concentration trends indicating that contaminant movement is continuing to the north-northwest. Concentrations in well 699-50-53A had been trending downward the last few years and have shown slight increases in the last year (before pump-and-treat operations). Another interesting trend can be seen in well 699-49-57A, where concentrations for all three contaminants are elevated and possibly increasing for cyanide and 99Tc. This would indicate a continued westward movement of the plume in addition to the north-northwest trend. A discussion of the conceptual model of contaminant distribution sources from the BY-Cribs is presented in Section 5.4.

5.2.2.3 Gable Mountain Pond Groundwater Contamination. Between the years 1957 and 1987, an estimated 3.07 x 10¹¹ L (8.1 x 10¹⁰ gal) of liquid mixed waste primarily from the 202-A Building was disposed of to the 216-A-25 Pond (Gable Mountain Pond). The pond received low-level radioactive liquid wastes, primarily tritium (213 Ci), ²⁴¹Am (5.3 x 10⁻⁴ Ci), ¹³⁷Cs (204 Ci), ¹⁰⁶Ru (1.6 x 10⁻⁴ Ci), ⁹⁰Sr (257 Ci), and plutonium (428 g). As mentioned in Section 5.1, on June 1, 1964, a single unplanned release from a cooling coil in the PUREX Plant contributed about 100,000 Ci of mixed fission products to Gable Mountain Pond, B Pond, and the 216-B-3-1 Ditch. Following the incident, bentonite clay was added to the pond in an effort to bind radionuclides in the upper sediment layers and reduce the percolation rates (Maxfield 1979). In addition, new inlet

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ditches were constructed, and the water level in the pond was maintained until the contamination level in pond water decreased (Strait and Moore 1982).

Elevated gross beta was quickly detected in groundwater wells following the June 1964 unplanned release (e.g., gross beta results in well 699-55-50A illustrated in Figure 5-23). The area of gross beta contamination beneath the pond was reported and mapped in 1964 (map for January-December 1964 in Figure 5-12). By the mid-1970's, a gross beta plume was no longer being mapped beneath the pond (see January-December 1974 map in Figure 5-12). Concentrations in well 699-55-50A are shown to have declined to less than 100 pCi/L, which was the minimum contour interval on the contaminant distribution map. By 1985, however, a beta plume reappeared in the vicinity of the pond. Strontium-90 concentrations began to increase in the mid-1980's as illustrated in the time-history plots in Figure 5-24 for wells that monitor contamination in the vicinity of the Gable Mountain Pond site. Contaminant trends in most of these wells have continued to increase to the present day.

5.3 CONTAMINANT CONCEPTUAL MODELS

This section presents conceptual models for the B-5 Reverse Well, BY-Cribs, and Gable Mountain Pond contaminant plumes. Relevant details concerning the source chemistry, solubility, and response to aquifer conditions as well as mobility are discussed. The contaminant response during the treatability test is compared to the conceptual model. Where multiple conceptual models have been proposed, tests are discussed to relate the conceptual model to actual aquifer conditions.

5.3.1 216-B-5 Reverse Well

Waste from the B Plant operation, which used the BiPO₄ separation process, was disposed of in the B-5 Reverse Well between 1945 and 1947 (see Figure 5-13). Disposal of the waste, which contained significant quantities of plutonium, ¹³⁷Cs, ¹⁰⁶Ru, and ⁹⁰Sr, was halted in 1947 because of groundwater contamination. Subsequent work indicated that the B-5 Reverse Well penetrated approximately 3 m (10 ft) into the aquifer and that significant quantities of radionuclides are present beneath the water table. Descriptions of waste disposal activities and subsequent characterization work are contained in Brown and Ruppert (1950) and Smith (1980). Hodges (1995) provided the basis for the current conceptual model.

5.3.1.1 216-B-5 Plume Groundwater Contamination. As discussed previously in Sections 5.1 and 5.2, groundwater contamination occurred at the B-5 Reverse Well between 1945 and 1947, when approximately 3.07 x 10⁷ L (8.1 x 10⁶ gal) of high-activity effluent from the B Plant complex was sent to the reverse well for disposal. The effluent was routed to tank 241-B-361 for precipitation of the entrained solids, and overflow from tank 241-B-361 was disposed of in the B-5 Reverse Well. The settling tank had a volume of approximately 132,000 L (35,000 gal); thus, the total effluent volume represents approximately 230 tank volumes. Most of the water and an unknown portion of waste solids went to the reverse well. Approximately 121,000 L (32,000 gal) of sludge remained in the settling tank after waste disposal operations were terminated.

Estimates from operational records indicate that approximately 4.27 kg of plutonium, 81 Ci of ¹³⁷Cs, 163 Ci of ¹⁰⁶Ru, 76 Ci of ⁹⁰Sr, and 3,800 Ci of miscellaneous beta activity were sent to the 241-B-361 Settling Tank. Analysis of sludge in tank 241-B-361 in 1979 indicates that approximately one-half of

the 4.27 kg of plutonium discharged passed through the tank and entered B-5 Reverse Well. However, ¹³⁷Cs and ⁹⁰Sr concentrations reported for the sludge indicate that the settling tank contains considerably more ¹³⁷Cs (2 times) and ⁹⁰Sr (30 times) than was reportedly discharged to the system (decay corrected to 1979). This discrepancy indicates considerable uncertainty in the reported discharge numbers and may account for the difference between the expected contaminant response and the observed concentrations measured during the test operation (see Section 4.3.3).

Under the conditions reported for the effluent stream, plutonium would have far exceeded its solubility and would have been transported as a solid phase, either as oxide, hydroxide, or phosphate. The following evidence indicates that a large portion of the plutonium and 90 Sr disposed in the B-5 Reverse Well was in the form of BiPO₄ solids.

- Phosphate phases have a well-known affinity for actinides. In the BiPO₄ separation process, the precipitation of BiPO₄ from solution was used to remove plutonium from solution. Phosphates also have a strong affinity for strontium and other alkaline earth elements.
- Analysis of the sludge remaining in the settling tank indicates a strong partitioning of plutonium and ⁹⁰Sr into the solid phase, which is principally BiPO₄. Cesium-137 does not share in this strong partitioning into the solids.
- A silt layer, approximately 11.9 m (39 ft) above the 1948 water table but within the perforated interval, contains significant quantities of ¹³⁷Cs; however, plutonium and ⁹⁰Sr were not detected. This can be explained if effluent carrying dissolved ¹³⁷Cs were wicked into the silt by capillary attraction and particles carrying plutonium and ⁹⁰Sr were excluded (filtered) because of their size.
- Characterization of sediments from wells drilled in 1979 indicates that plutonium and %Sr within the sediments are highly localized, contained within a zone less than 20 m across. Cesium-137 was found to have a very different distribution; significant concentrations within the sediments extend past the boundaries of the study area, approximately 22 m from the reverse well. Plutonium, because of its low solubility and very high sorption K_d , would be expected to be localized around the injection site. In the case of ¹³⁷Cs and ⁹⁰Sr, if sorption on sediments is the major factor controlling distribution, ¹³⁷Cs and ⁹⁰Sr should behave in a similar manner. In addition, the sorption K_d for ⁹⁰Sr on Hanford Site sediments is approximately half that for ¹³⁷Cs. Thus, the ⁹⁰Sr should have moved further than the ¹³⁷Cs, the opposite of what was observed.

These differences and similarities support phosphate as the major control on the distribution of plutonium and 90 Sr within the aquifer. This mechanism would be reinforced by the fact that any dissolved phosphate in the waste stream, or any phosphate resulting from the dissolution of BiPO₄, would quickly react with calcium ions in groundwater to form the very insoluble mineral phase apatite [Ca₅(PO₄)₃OH, K_{SP} = $10^{-57.8}$ (Krauskopf 1979)], which has a strong affinity for strontium and actinides.

Determination of the groundwater distribution of radionuclides around the B-5 Reverse Well is difficult because of the lack of wells away from the immediate vicinity of the reverse well and the practice of analyzing only unfiltered samples for radionuclides. Analytical results for radionuclides in wells adjacent to the reverse well are highly variable, and it is impossible to determine, on the basis of existing data, how much of the measured activity is in solution and how much is contained within

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or on solid particles. Analytical data for the present concentrations of plutonium, ¹³⁷Cs, and ⁹⁰Sr in groundwater around the B-5 Reverse Well are indicated in Figures 5-15 through 5-17. The zone bounded by the 1-pCi/L plutonium contour is approximately 180 m (591 ft) across; the zone defined by the ⁹⁰Sr 8-pCi/L contour is approximately 300 m (984 ft) across; and the zone bounded by the ¹³⁷Cs 120-pCi/L contour is approximately 200 m (656 ft) across.

5.3.1.2 Source Characteristics. The waste discharged to the 241-B-361 Settling Tank, and thus to the B-5 Reverse Well, was a low-salt, alkaline waste containing approximately 1.4 x 10^{-4} g/L of plutonium, 2.6 x 10^{-6} Ci/L of 137 Cs, 5.3 x 10^{-6} Ci/L of 106 Ru, 2.5 x 10^{-6} Ci/L of 90 Sr, and 1.2 x 10^{-5} Ci/L of gross beta activity. Analyses of the sludge in tank 241-B-361 indicate that the major nonradioactive constituent was BiPO₄ with lesser quantities of manganese, iron, lanthium, sodium, nitrate, and fluoride, consistent with waste from the BiPO₄ separations process.

The indicated concentration of plutonium far exceeds the solubility of plutonium under these conditions, and it is likely that plutonium was transported in a solid form. The BiPO₄ that forms a large portion of the sludge in the settling tank would have also had very low solubility under the alkaline conditions of the effluent and would have consisted principally of solid particles. BiPO₄ was precipitated, during the BiPO₄ process, to scavenge plutonium from solution. It is reasonable to assume that a portion of the plutonium, and probably of the strontium as well, was transported as part of the BiPO₄ solids.

- **5.3.1.3 Solubility and Sorption Controls**. The transport and behavior of plutonium and associated fission products (¹³⁷Cs and ⁹⁰Sr) are highly dependent on both solubility and sorption/desorption controls. Accordingly, data relevant to both control mechanisms are discussed as follows.
- **5.3.1.3.1** Cesium. Cesium, like other alkali metals, has high solubility in aqueous systems, and the principal controlling mechanism will be sorption and not solubility. A wide range of sorption K_d have been reported in the literature; however, the most reasonable value for Hanford Site sediments seems to be approximately 50 mL/g (Ames and Serne 1991). Using this value for K_d , and assuming a bulk density of 2.1 g/cm³ and a porosity of 0.22, yields a retardation factor of 525. Thus, if the groundwater flow rate during the time (48 years) since the waste was disposed in the B-5 Reverse Well had been 1 m/day (3.3 ft/day), groundwater would have moved 18 km (11 mi); however, during this time, the cesium would have moved approximately 34 m (110 ft). The initial rate of movement may have been higher, however, because high concentrations of cesium adjacent to the reverse well may have saturated sorption sites.
- 5.3.1.3.2 Plutonium. Based on effluent discharge records, the average concentration of plutonium in alkaline aqueous waste discharged to the reverse well was approximately 6×10^7 M, which is well above the solubility of plutonium in the (IV) or (VI) oxidation states in alkaline solutions. For example, Serne and Wood (1990) reported a laboratory solubility for plutonium oxides of 1.8×10^{-8} M, which is equivalent to 3.26×10^{-7} Ci/L of 239 Pu. Barney (1988) indicates a solubility limit of 3.5×10^{-9} M (pH of 8+) for Pu(VI). Thus, much of the plutonium present in the waste stream would be present in a solid phase, possibly hydrous oxide or hydroxide phases, but probably in this case principally in the particulate BiPO₄ phase. Barney (1988) also conducted laboratory sorption tests using Hanford Site (200 East) sediments at plutonium concentrations below 3.5×10^{-9} M and found that sorption K_d values for plutonium initially added as Pu(VI) averaged $4,600 \pm 1,000$ mL/g after an 18-day contact time [the apparent K_d increased with time during the first few days, which was attributed to reduction of Pu(VI) to Pu(IV) by the sediments in the batch sorption tests]. The sediments used for the laboratory sorption determination may not be representative of the

aquifer sediments surrounding the B-5 Reverse Well. K_ds presented here are best estimates based on existing data.

Solubility considerations, as discussed previously, suggest that most of the plutonium initially injected into the aquifer was present as a particulate and/or colloidal phase. The solubility of plutonium included in BiPO₄ or apatite phases is uncertain; however, if the solid phase is an oxide or hydroxide, pore fluid in the immediate vicinity of the solid phase would theoretically reach a solubility controlled maximum concentration of 326,000 to 63,400 pCi/L, depending on the solubility limit used (see above). However, because of the affinity of sediments for the solute phase, the pore fluid concentrations would theoretically be reduced in proportion to the apparent K_d . Assuming the K_d value noted above is applicable to B-5 Reverse Well chemical conditions, the predicted final pore fluid concentration would be reduced by a factor of 4,600, or a pore fluid concentration of 326,000/4,600 = 71 pCi/L to 63,400/4,600 = 14 pCi/L.

- 5.3.1.3.3 Strontium. Strontium is more likely to have solubility controls than cesium, because it has a lower solubility than calcium carbonate (calcite) and is very insoluble in the presence of phosphate. On Hanford Site soils, it has a K_d of approximately 25 mL/g, approximately one-half the value for cesium (Ames and Serne 1991). If sorption on sediments is the primary control, the ⁹⁰Sr should have migrated approximately twice as far as ¹³⁷Cs during the same time period. During the 33 years between disposal and the 1979 characterization study, the ⁹⁰Sr would be expected to have migrated approximately 46 m (150 ft); however, drilling indicated that ⁹⁰Sr was restricted to approximately 10 m (33 ft) of the reverse well, indicating that sorption on sediments is not the major control on ⁹⁰Sr mobility. As indicated previously, the major control on strontium mobility is probably one or more phosphate phases. The nature of these phases and the solubility of ⁹⁰Sr contained in them is unknown.
- 5.3.1.4 Predicted Response to Pump-and-Treat Operations. If plutonium and strontium within the sediments are principally contained within phosphate phases, the ability to remove them by pump-and-treat technology is uncertain. If the main phosphate phase is BiPO₄, any pumping that resulted in increased solubilization of the phosphate would also introduce groundwater calcium that would favor the precipitation of the mineral apatite. In the absence of data on possible phosphate phases in the aquifer, the following discussion assumes that the plutonium is sorbed on sediments and/or exists as solid oxide or hydroxide phases. It also assumes, given uncertainty in the discharge numbers, that the total reported inventory of plutonium was discharged into the reverse well.

Assuming that the volume within which most of the plutonium is contained is a cylinder with a radius of 10 m (33 ft), depth of 41 m (133 ft), and a porosity of 0.26 [tracer test porosities are 0.22 and 0.30 (Section 4.4.4)], the volume (V) of plutonium-contaminated groundwater within the sediments is:

$$V = 3.14 \times 10^2 \times 10 \times 0.62 = 816 \text{ m}^3 \text{ or } 8.16 \times 10^5 \text{ L } (2.16 \times 10^5 \text{ gal}).$$

At the highest pore fluid concentration noted, the quantity of plutonium removable in one pore volume from the assumed volume of contaminated aquifer solids would be:

quantity =
$$8.16 \times 10^5 L \times 3.26 \times 10^5 pCi/L = 2.66 \times 10^{11} pCi/L \text{ or } 0.26 Ci.$$

The total injected was:

$$Ci = 4,270 \text{ g x } 0.0613 \text{ Ci/g} = 262 \text{ Ci (as }^{239}\text{Pu}).$$

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Thus, under the most favorable conditions, the amount removed would be:

 $0.26/262 \times 100 = 0.10\%$ per pore volume removed.

Assuming simple plug flow (no dilution of plume during withdrawal), 1,000 pore volumes (8 x 10^8 L) would be required to hypothetically remove the injected quantity, assuming also that the aquifer would be allowed to reequilibrate after each pore volume. With continuous pumping, extraction of a much larger volume would be required. In addition, if the plutonium is contained within a solid phosphate phase, the above number may be a lower estimate for the quantity of water required for a simple pump-and-treat operation. This can be compared to the 2.3 pore volumes (1.88 x 10^6 L) that would be required to remove 90% of the plutonium if it were not sorbed on or contained within solid phases.

Because of the similar behavior of 90 Sr and plutonium around the reverse well and the likelihood that both are contained within a solid phosphate phase, it is likely that similar techniques and similar volumes of groundwater would be required to remove a significant quantity of 90 Sr from the aquifer. Because the distribution of 137 Cs seems to be controlled by sorption on sediments, it is likely that any pump-and-treat process that remediated plutonium and 90 Sr around the reverse well would also remove most of the 137 Cs trapped on the sediments near the well.

5.3.2 216-BY Cribs Plume

Waste solutions from the TBP process were disposed of during 1954 and 1955 to a group of cribs in the northern portion of the 200 East Area commonly known as the BY-Cribs (216-B-43 through 216-B-49) (see Figure 5-3). Between 1965 and 1974, additional liquid waste in the form of condensate effluent from the ITS process was disposed of in the remaining unused BY-Crib (216-B-50) and the adjacent 216-B-57 Crib. As early as 1956, beta-gamma and ⁶⁰Co contamination were recognized in the groundwater beneath these cribs and farther south in the 200 East Area. Currently, ⁶⁰Co and ⁹⁹Tc contamination is recognized in the groundwater underneath the cribs and north of the 200 East Area. Descriptions of the waste disposal operations and recent characterization activities are described in DOE-RL (1993a, 1993c, 1993d) and Kasza (1993).

5.3.2.1 216-BY Cribs Plume Groundwater Contamination. As reported in earlier sections, scavenged TBP supernatant waste was generated during the ferrocyanide scavenging of TBP process effluent and was routed to the 241-BY Tank Farm to allow settling of suspended solids before releasing the liquids to the cribs. About 33,841,000 L (8,940,000 gal) of this liquid effluent passed through a three-tank cascade system and was discharged to seven of the eight BY-Cribs. Disposal of TBP process wastes to the BY-Cribs was terminated after beta activity was detected in the groundwater beneath the cribs. Between 1965 and 1974, 61,000,000 L (16,000,000 gal) of tank liquor condensate from the ITS B-5 Reverse Well was discharged to the 216-B-50 Crib. The nearby 216-B-57 Crib received 83,000,000 L (22,000,000 gal) of tank condensate from the ITS BY-Cribs during the period between 1968 and 1973. Estimated quantities of the primary contaminants in all liquid effluent streams disposed of to the BY-Cribs included 0.45 Ci ⁶⁰Co, 18,900 kg ferrocyanide, 5,691,500 kg nitrate, and an unknown amount of ⁹⁹Tc (DOE-RL 1993d).

Groundwater monitoring activities have identified a large groundwater contamination plume located at the northern fenceline and north of the 200 East Area toward Gable Gap (DOE-RL 1993a, 1993c, 1993d). Hanford Site records indicate that the BY-Cribs are the most probable source of groundwater

contamination at or near this location. Discharges from the 200 North Area (Section 5.1.1.4) were examined as a potential contaminant source; however, the position of the plume and the presence of specific process effluents (60Co and cyanide) suggest that the plume migrated to its current location and is the result of disposal practices at the BY-Cribs. A northerly local groundwater flow direction is reflected in the areal distribution of the plume.

- 5.3.2.2 Solubility and Sorption Controls. Data presented in the 200 East Groundwater Aggregate Area Management Study Report (DOE-RL 1993a) suggest there is little tendency for the primary contaminants ⁶⁰Co and ⁹⁹Tc to adsorb to the fine-grained soils beneath the Hanford Site. Cobalt-60 normally exists as a divalent cation in acidic to mildly alkaline solution and is prone to adsorbing onto soil via cation exchange. However, greater mobility is possible as ⁶⁰Co can form anion or neutral complexes. In particular, the complexing of ⁶⁰Co by cyanide has been proposed as a means of explaining the observed high mobility of ⁶⁰Co in the BY-Cribs plume (DOE-RL 1993d). Technetium-99 typically forms negative ions in oxidizing environments and does not readily complex with other chemical species. Technetium-99 also has a very low soil-water (K_d=0). Consequently, there is limited tendency for ⁹⁹Tc to adsorb to the Hanford Site sediments, which makes the radionuclide mobile.
- **5.3.2.3** Local Geohydrology. The geohydrology of the area containing the BY-Cribs plume is controlled to a major extent by the underlying Elephant Mountain basalt. Basalt structural highs generally restrict groundwater flow to the more conductive sedimentary deposits located between the basalt highs. Erosional features incised into the basalt may provide localized preferential flow paths. The erosional window, as identified in wells 699-55-55 and 699-55-53, may be acting as pathway for vertical migration between the uppermost unconfined aquifer and the underlying interbed. Borehole television survey logs of the basalt portion of several wells within the plume reveal that the basalt contains numerous subvertical to horizontal fractures, which could act as conduits for groundwater flow. The conductivity of these fractures is unknown.

The BY-Cribs ⁹⁹Tc plume is defined by five groundwater monitoring wells. It is important to note that these five wells penetrate the full thickness of the aquifer and also enter into the underlying basalt. Wells 699-50-53A and 699-52-54 have thin [<0.6-m-thick (<2-ft-thick)] saturated unconfined aquifer interval. Both wells are located near the westerly margin of a subsurface structural basalt high that extends above the current groundwater table. Wells 699-49-55A, 699-49-57A, and 699-55-57 are located west of the basalt high and are situated in a thicker portion of the unconfined aquifer. Based on the most recent groundwater level measurements, the saturated thickness is approximately 3.1 m (10 ft) at well 699-49-55A and about 2.7 m (9 ft) at well 699-49-57A.

Aquifer data in the vicinity of the BY-Cribs plume are limited by the age of the wells and the distance between them. Based on data provided in the 200 East Groundwater Aggregate Area Management Study Report (DOE-RL 1993a), the hydraulic conductivity and transmissivity of the aquifer are relatively high. Section 4.4 provides additional aquifer property information assembled during the treatability test aquifer investigations.

5.3.2.4 Contaminant Source Models. Much of the success of a groundwater remediation program depends on the ability to define the contaminant source model. It is imperative to determine if all of the available contaminant has already entered the aquifer, or if a continuing source of contamination exists that contributes to the future growth of the contaminant plume. If a continuing source exists, its location should be determined so that remedial activities can be directed at the areas of highest

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concentration. Two models currently exist that try to define the source of the BY-Cribs contaminant plumes. For these models to be acceptable, the following observed conditions must be explained.

- Cyanide and ⁹⁹Tc are still present in detectable, elevated concentrations in wells associated with the BY-Cribs (e.g., wells 299-E33-7 and 299-E33-13). It does not seem likely that the BY-Cribs continue to be an active source 20 years after receiving the final waste effluent. Site characterization activities conducted for the 200-BP-1 remedial investigation determined no large, active source of groundwater contamination currently exists in the soils beneath the BY-Cribs (DOE-RL 1993d).
- High contaminant concentrations in the groundwater north of the BY-Cribs, and especially associated with wells 699-50-53A and 699-52-54, have been known for many years.
 Technetium-99 concentrations at well 699-50-53A increased ten-fold over background values during the operation of the pilot-scale treatment plant.

One proposed explanation for the observed trends in contamination north of the BY-Cribs is that the lowered water tables in the area during the mid-1970's to mid-1980's left contaminants in the lower vadose zone as sorbed material on mineral surfaces and in interstitial water (see Figure 5-9 for well hydrographs in the vicinity of the BY-Cribs). Subsequent increases in the elevation of the water table beginning in the mid-1980's resaturated this contaminated zone and led to the remobilizing of contaminants and increased observed concentrations. The proposed correlation can be seen by comparing the fluctuations in water level for well 699-50-53A in Figure 5-9 with the ⁶⁰Co trend for the same well as illustrated in Figure 5-20.

An alternative model proposed by Kasza (1993) is based on the known high density of the process wastes disposed of to the BY-Cribs and on the results of depth-specific groundwater sampling conducted in the 1960's. Process records and laboratory simulation experiments indicate that the scavenged TBP supernatant disposed of to the BY-Cribs had a high specific gravity (1.21 to 1.32). Field studies conducted between 1956 and 1980 recognized that the scavenged TBP supernatant formed mobile contaminant mass, which had a density greater than water, that migrated southward along the bottom of the unconfined aquifer. Depth-specific groundwater sampling and borehole geophysical logging shows that the dense contaminant mass migrated several thousand feet to the south.

Kasza (1993) proposes that after saturation of the open pore space beneath the BY-Cribs, the dense waste was able to enter the aquifer with little attendant mixing with the groundwater. Beneath the BY-Cribs, the aquifer was approximately 3 m (10 ft) thick during the period of waste disposal, and the initial dilution of the dense scavenged TBP supernatant was likely minimal. Once at the bottom of the aquifer, the dense waste appears to have behaved as a separate dense phase and moved as a coherent mass primarily because of its density difference with the groundwater. The waste moved along the surface of the basalt, displacing groundwater from fractures and depressions, while it migrated down the topographic surface and away from the BY-Cribs. Movement was likely accompanied by mixing and diffusing into the surrounding groundwater. The principal contaminant migration direction away from the cribs was toward the south, down the southward-dipping top of the basalt unit underlying the aquifer. Smith (1980) and others record increased contaminant concentrations in the sediments immediately above the basalt at wells several hundred feet south of the BY-Cribs. Because detailed configuration of the basalt surface is not known, it is not possible to establish the migration path of the dense contaminant mass.

During the intervening years, density-driven movement probably slowed or ceased because dilution reduced the density differences between the groundwater and the waste. This decrease would be most likely caused by the dense waste mixing with, and diffusing into, the surrounding groundwater. The rising regional water table for some period of time established northward groundwater flow. Dissolved contaminant movement eventually followed the regional groundwater flow to the north and became the dominant contaminant transport mechanism.

Because minimal vertical advection and diffusion is likely to occur in a fracture-controlled aquifer system, any portion of the contaminant mass that had entered the fracture system in the basalt through density-driven displacement of the groundwater would likely remain stratified in the basalt and relatively undiluted. In that position, it is undetected by the majority of the groundwater monitoring wells, which sample only the upper portion of the aquifer. As shown in Figure 5-22, the wells that comprise the ⁹⁹Tc contaminant plume penetrate and collect groundwater samples from both the saturated thickness of the aquifer sediments and the upper portion of the basalt. This access to the fluids contained in the fractures of the basalt may be responsible for the increased contaminant levels measured at those wells. The observed increase in ⁶⁰Co and ⁹⁹Tc concentrations at the extraction well (699-50-53A) during the operation of the pilot plant may have occurred because the large amount of drawdown at the well caused more concentrated waste from the fractures to be drawn into the well's capture zone.

Both of the models introduced here satisfy the previously stated conditions to various degrees. The rising water-table model would not itself address contaminants found in wells associated with the BY-Cribs if the local groundwater flow direction were still to the north. The dense contaminant source located deep in the aquifer would not supply contaminants to the plume if groundwater beneath the BY-Cribs flowed to the south. Either model may have been dominant at various times in the history of the plume. A hybrid model combining aspects from both of the proposed models may be an appropriate predictor of future plume behavior. More hydrogeologic and contaminant character information is needed before the model can be validated.

5.3.3 216-A-25 Gable Mountain Pond Plume

The 216-A-25 Gable Mountain Pond received an estimated total waste volume of 307,000,000,000,000 L (81,100,000,000 gal) of effluent between 1957 and 1987 (Coony and Thomas 1989). Figure 5-4 shows the effluent discharge history. The effluent was principally 202-A Building process cooling water, 241-A-431 surface condenser cooling water, 242-A Evaporator cooling water, 244-AR Vault cooling water, and 284-E Powerhouse water (Maxfield 1979, WHC 1991). The effluent contained low levels of radionuclides, including ²⁴¹Am, ¹³⁷Cs, ³H, ¹⁴⁷Pm, ¹⁰⁶Ru, ⁹⁰Sr, and plutonium, and reached the 216-A-25 Gable Mountain Pond via the PUREX Cooling Water Line, which discharged into the southeastern end of the pond (Brown et al. 1990, WHC 1991). A single, unplanned release in 1964 resulted in the discharge of approximately 7,500 Ci of mixed fission products to the pond, of which about 99% were short-lived (Maxfield 1979). The estimated radionuclide inventory in 1995 for the 216-A-25 Gable Mountain Pond, decayed from the data presented in Table 5-2, are 5.21 x 10⁻⁴ Ci of Am²⁴¹, 1.66 x 10² Ci of ¹³⁷Cs, 1.28 x 10² Ci of ³H, 3.37 x 10⁻⁷ Ci of ¹⁰⁶Ru, 2.06 x 10² Ci of ⁹⁰Sr, 428 g of plutonium, and 4.24 Ci of uranium. The inventory at the pond is a consequence of the 30 years of routine effluent discharges combined with the unplanned release in 1964.

A plume of ⁹⁰Sr-contaminated groundwater exists at the pond. The extent of the 216-A-25 Gable Mountain Pond ⁹⁰Sr plume is shown in Figure 5-25 by 8- and 100-pCi/L ⁹⁰Sr contours generated from

1994 data. The area enclosed by the 8-pCi/L contour is about 5.9 x 10^5 m² (6.35 x 10^6 ft²). The area enclosed by the 100 pCi/L contour is about 4.5 x 10^4 m² (4.83 x 10^5 ft²).

Before decommissioning, the pond elevation was 131.9 m (432.6 ft) and had a maximum depth of about 2.4 m (8 ft) (Hayward 1989). Decommissioning of the pond began in 1984 and was completed in 1988 with the interim stabilization of the site (see Section 5.1.2.4).

5.3.3.1 Gable Mountain Pond Source Characteristics. The unplanned release (UPR-200-E-34) on June 11, 1964, released approximately 7,500 Ci of short- and long-lived fission products to the 216-A-25 Gable Mountain Pond. The following description of the release and responses is summarized in Maxfield (1979). The short-lived fission products (e.g., those with half-lives < 2.5 years) constituted approximately 99%, or 7,445 Ci, of the release. In 1964, most of the activity was attributed to 95Nb, 103Ru, 106Ru, and 95Zr. Approximately 25 Ci of 137Cs and 30 Ci of 90Sr, the half-lives of which are 29.1 and 30.2 years, respectively, were released. Bentonite clay was added to the 216-A-25 Gable Mountain Pond soon after the release to bind radionuclides in the upper sediment layers and raise the water level in the pond by reducing the percolation rate. However, the pond water level did not rise after application of bentonite. Concurrently, to reduce radiation exposure and prevent the spread of contamination, bulldozers were used to push fill into the pond to cover contaminated sediment at shoreline and in the shallows. As this earthmoving proceeded, it was noted that the pond water level began rising. When the earthmoving was completed, fill had been pushed into the pond along the entire perimeter to an average width of 6 m (20 ft). It was surmised that percolation had been occurring principally at the pond margins before the placement of the fill.

In 1971 and 1972, PNNL personnel conducted a study of Gable Mountain Pond (Cushing and Watson 1974). Cushing and Watson (1974) found that ¹³⁷Cs accounted for 90% of the radioactivity at the pond, and 90% of ¹³⁷Cs occurred in the top 5 cm (2 in.) of pond sediment. In contrast to ¹³⁷Cs, the ⁹⁰Sr concentration in the pond sediment was greater in deeper core sample intervals, although specific sample intervals and dimensions of the cores were not published (Cushing and Watson 1974).

5.3.3.2 Hydrologic and Hydrochemical Concept. Strontium-90 is the primary contaminant of concern at the 216-A-25 Gable Mountain Pond; groundwater monitoring since the 1980's has not disclosed other plumes. The ⁹⁰Sr is assumed to have been a dissolved constituent in the effluent and to have been adsorbed onto the aquifer materials.

Wells 699-53-47A, 699-53-47B, 699-53-48A, 699-53-48B, 699-54-48, 699-54-49, 699-55-50, and 699-55-50D provide geologic and hydrologic characterization data for the unconfined aquifer near the Gable Mountain Pond. These wells and others are shown in Figure 5-26. Five of the wells (i.e., 699-53-47B, 699-53-48A, 699-53-48B, 699-54-48, and 699-54-49) were drilled as part of an investigation of the pond in 1984 (Law et al. 1986).

The unconfined aquifer intercepted in the boreholes consists of silt; sands; sands and gravel; and gravel, cobbles, and boulders of the Hanford formation. The aquifer thickness (i.e., from static water level to top of the Elephant Mountain basalt) ranges from 2 m (7 ft) in well 699-53-47B to 17 m (55 ft) in well 699-55-50C. The Elephant Mountain basalt subcrop is 6 to 9 m (20 to 30 ft) below grade at the southeast end of the Gable Mountain Pond (well 699-53-47A) and 27 to 30 m (90 to 100 ft) below grade at the northwest end of the pond (well 699-54-50C). North and south of the pond, the Elephant Mountain basalt forms extensive subcrops above the water table. Figure 5-26 shows the basalt subcrops that extend above the water table and the resulting northwest flow of groundwater (i.e., toward Gable Gap).

Tests conducted in 1956 revealed the following characteristics of the unconfined aquifer: transmissivities ranged from 5.6 x 10^4 to 3.7 x 10^4 m²/day (6.0 x 10^5 to 4.0 x 10^5 ft²/day), equivalent hydraulic conductivities ranged from 7.6 x 10^3 to 2.6 x 10^3 m/day (2.5 x 10^4 to 8.5 x 10^3 ft/day), and storage coefficients were 0.21 and 0.21 at wells 699-55-50C and 699-55-50D, respectively (DOE-RL 1993a). The aquifer properties in the area tested by 699-55-50 series wells were summarized by Deju (1974) as follows: transmissivity of 5.52 x 10^4 m²/day (5.94 x 10^5 ft²/day), hydraulic conductivity of 2.8 x 10^3 m/day (9.1 x 10^3 ft/day), and coefficient of storage 0.07.

The 90 Sr K_d in seven sediment samples from wells 699-53-48A and 699-54-48 are reported by Law et al. (1986). The K_d s ranged from 21.9 to 95.1 mL/g. The K_d of 95.1 mL/g was from a sample of silt; the other K_d values were from coarser grained sediment. A K_d of 25 mL/g was selected as a conservative, representative value for the Hanford formation. This K_d correlates to a retardation factor of 120 (e.g., the rate of 90 Sr movement through the aquifer is 120 times less than the rate of groundwater movement) (Law et al. 1986).

The 1984 hydrologic investigation at Gable Mountain Pond was initiated after an increasing trend of ⁹⁰Sr in groundwater monitored by well 699-53-47A was noted in 1983 (Law and Allen 1984). The static water-level elevation in June 1984 ranged from 125.47 m (411.65 ft) above mean sea level in well 699-53-47B to 123.23 m (404.31 ft) above mean sea level in well 699-55-50C. The average groundwater concentrations of ⁹⁰Sr in 1984 exceeded the Rockwell Hanford Operations administrative guideline of 30 pCi/L in wells 699-53-47A (65 pCi/L), 699-53-47B (63 pCi/L), and 699-53-48B (608 pCi/L) (Law et al. 1986). Only low concentrations of ⁹⁰Sr were found in groundwater from other upgradient and downgradient monitoring, which indicated that significant ⁹⁰Sr contamination was localized near well 699-53-48B (Law et al. 1986).

5.3.3.3 Contaminant Source Model. Section 5.3.1.3 addresses the roles that solubility and sorption-desorption play in the mobility of ¹³⁷Cs and ⁹⁰Sr. Sorption-desorption is probably the primary control for ⁹⁰Sr and ¹³⁷Cs migration at the 216-A-25 Gable Mountain Pond. Cushing and Watson (1974) noted that ¹³⁷Cs appears to be strongly held in the sediments immediately below the pond. The same is not expected for ⁹⁰Sr; it is thought to occur at the top of the aquifer (Serkowski and Jordan 1989). Given the ⁹⁰Sr Rf of 120, appreciable mobility of ⁹⁰Sr is not expected and has not occurred.

The ⁹⁰Sr concentrations in groundwater from 1980 through 1995 for wells 699-53-47A, 699-53-47B, 699-53-48A, 699-53-48B, 699-54-48, and 699-54-49 are shown in Figure 5-27. Note that samples with nondetected or negative concentration results were plotted as "0" in the graphs of Figure 5-28. Interpretation of these data is complicated by the change in sampling frequency from quarterly to semiannually in 1990. The ⁹⁰Sr concentrations are greatest in well 699-53-48B and have increased roughly five-fold in wells 699-53-47B and 699-53-48B between 1990 and 1995. The cause of this phenomenon is not known. The changes in concentrations do not appear to be correlated, between wells, with the changes in water-table elevations shown in Figure 5-27. Concentrations of ⁹⁰Sr in well 699-53-47B increased as water-table elevations declined gradually while from January 1990 until October 1991, concentrations of ⁹⁰Sr in well 699-53-48B decreased as the water-table elevations declined. Since October 1991, both the water-table elevations and ⁹⁰Sr concentrations in well 699-53-48B have increased. The water-table elevations since 1986 in well 699-53-47B correspond to an interval of basalt boulders and fine sand.

The ¹³⁷Cs concentrations in groundwater from 1970 through 1995 for wells 699-53-47A, 699-53-47B, 699-53-48A, 699-53-48B, 699-54-48, and 699-55-50C are shown in Figure 5-29. Samples with

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nondetected or negative concentration results were plotted as "0" in the graphs of Figure 5-29. The maximum concentration of ¹³⁷Cs is 32 pCi/L from well 699-53-47A.

5.4 RISK ANALYSIS

Risk analyses were performed for the following three groundwater contaminant plumes within the 200-BP-5 Operable Unit: BY-Cribs plume, B-5 Reverse Well plume, and Gable Mountain Pond plume.

This section provides a summary of the risk analyses performed for these sites. The details are documented separately in *Risk-Based Decision Analysis for the 200-BP-5 Groundwater Operable Unit* (BHI 1995c).

The contaminants of potential concern (COPC) in the three plumes are the radionuclides ⁶⁰Co and ⁹⁹Tc in the BY-Cribs plume; ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr in the B-5 Reverse Well plume; and ⁹⁰Sr in Gable Mountain Pond plume. The methodology used in the analysis is based on *Risk-Based Decision Analysis for Groundwater Operable Units* (BHI 1995b).

Risk assumptions and scenarios were based on *Hanford Site Risk Assessment Methodology* (HSRAM) (DOE-RL 1995b). Hypothetical exposure scenarios were assessed based on both current and future plume conditions. For current conditions, a hypothetical industrial groundwater ingestion scenario was evaluated; for future conditions, both industrial and residential ingestion scenarios were assumed. The industrial ingestion scenario, which is derived from HSRAM, was used in this risk analysis for illustrative purposes only. There is currently no use of the 200 Area groundwater for drinking water nor is any anticipated in the foreseeable future; therefore, it should not be implied by this risk analysis that DOE is advocating use of this groundwater for direct human ingestion.

For future conditions, the transport of ^{239/240}Pu in the B-5 Reverse Well, the ⁹⁹Tc in the BY-Cribs plume, and the ⁹⁰Sr in the Gable Mountain Pond plume was modeled using a two-dimensional analytical flow and transport model called CONMIG (Walton 1989). Modeling simulated dispersion, adsorption, and radioactive decay of constituents during transport.

5.4.1 216-BY Cribs Plume

The maximum concentration of ⁹⁹Tc measured during the period between January 1993 and March 1995 was used as the input concentration for each well in the "existing conditions" risk assessment model. The 95% UCL concentration of ⁹⁹Tc measured since 1988 was used as the input concentration for well 699-50-53A in the "future conditions" risk assessment model.

5.4.1.1 Existing Conditions. The hypothetical industrial scenario risk based on contaminant concentrations and groundwater plume conditions was estimated, plotted, and contoured. The results show that most of the health risk is attributable to ⁹⁹Tc and not ⁶⁰Co. The baseline incremental lifetime cancer risk (ILCR) (6.9 x 10⁻⁵) is calculated for groundwater at monitoring well 699-50-53A, which during the baseline pre-treatability test period had a maximum ⁹⁹Tc concentration of 8,640 pCi/L. During the operation of the treatability test, the ⁹⁹Tc concentration increased to 11,200 pCi/L in this well, producing a corresponding ILCR of 8.9 x 10⁻⁵.

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The maximum ILCR for this plume, based on the historical highest concentration of ⁹⁹Tc measured at well 699-50-53A during the period 1988 to 1992 (27,548 pCi/L), would be 2.19 x 10⁻⁴. The risk contours for the existing plume of ⁹⁹Tc are plotted in Figure 5-30.

5.4.1.2 Future Conditions. The objective of transport modeling was to determine the flow path length at which the maximum ILCR for ⁹⁹Tc at any location in the plume was no greater than 1 x 10⁻⁵ (this objective was requested by the EPA). However, modeling results show that, because of the large size of the ⁹⁹Tc plume, ILCRs will exceed 1 x 10⁻⁵ at the end of the flow path (the Columbia River).

Because ⁶⁰Co has a short half-life of 5.3 years, it was eliminated from consideration for estimating future risk. The modeled transport of ⁹⁹Tc is essentially dependent on dispersivity because radioactive decay is negligible. There is, however, uncertainty in the specification of values for dispersivity. Based on the judgement of the modelers, a longitudinal dispersivity range of 61 to 305 m (200 ft to 1,000 ft) was selected for the analysis to show its sensitivity in the calculated risk results. Using these assumptions, modeling predicted the following:

	Longitudinal Dispersivity = 61 m (200 ft)	Longitudinal Dispersivity = 305 m (1,000 ft)
⁹⁹ Tc Concentration in Well 699-50-53A, pCi/L (existing plume)	2,633 m (8,640 ft)	2,633 m (640 ft)
Travel Time, Years	75	75
Travel Distance (to maximum risk point), m (ft)	8,778 m (28,800 ft)	8,778 m (28,800 ft)
Maximum ⁹⁹ Tc Concentration, pCi/L	4,190	1,270
Industrial Risk, ILCR	2.70 E-5	8.30 E-6
Residential Risk, ILCR	1.20 E-4	3.60 E-5

At the request of EPA, additional modeling runs were performed to estimate the location and magnitude of risks posed by the ⁹⁹Tc plume in the year 2018. Three modeling cases were addressed as follows:

- Case 1 A ⁹⁹Tc concentration observed in well 699-50-53A of 11,200 pCi/L during pump-and-treat operations. One dispersivity scenario was modeled for this concentration: 30 m (1,000 ft) longitudinal/30 m (100 ft) transverse.
- Cases 2a and 2b Historical high concentrations of ⁹⁹Tc in well 699-50-53A as represented by the 95% UCL of sample concentrations obtained during the period from June 26, 1987 to February 8, 1993. Two dispersivity scenarios were addressed: Case 2a 305 m (1,000 ft) longitudinal/30 m (100 ft) transverse and Case 2b 30 m (100 ft) longitudinal/3 m (10 ft) transverse. The 95% UCL was calculated to be 27,548 pCi/L.

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Results of these modeling cases are shown in the following table. Figure 5-30 shows the risk contours for Case 2a based on a residential scenario. The existing plume (industrial scenario) is plotted on the same base map in Figure 5-30 for comparison.

Case	1	2a	2b
⁹⁹ Tc concentration in well 699-50-53A, pCi/L	11,200	27,548	27,548
Dispersivity: longitudinal/transverse, m (ft)	305/30 (1,000/100)	305/30 (1,000/100)	30/3 (100/10)
Travel time (to 2018), yr	23	23	23
Travel distance to maximum risk point, m (ft)	2,682 (8,800)	2,682 (8,800)	a
Maximum 99Tc in future plume, pCi/L	3,800	7,843	a
Industrial Risk, ILCR	2.5E-5	5.1E-5	a
Residential Risk, ILCR	1.1E-4	2.2E-4	a
^a Unable to model this dispersivity and travel-time c	ondition.	·	

The Case 2b dispersivity and travel-time conditions could not be modeled. An attempted run indicated that future ⁹⁹Tc concentrations were greater than the maximum source concentration. This does not imply that the dispersivity and travel-time conditions are physically impossible, only that the analytical model will not provide a correct solution under the constraints imposed.

A comparison of Cases 1 and 2a shows that future maximum concentrations and the corresponding risks are approximately proportional to current historical maximums assuming the same dispersivity. For example, if the source concentration is assumed to be doubled, the maximum risk in the future plume is approximately doubled.

5.4.1.3 Conclusions. At the year 2018 and based on the historical 95% UCL ⁹⁹Tc concentration of 27,548 pCi/L, the modeling shows that the concentration of ⁹⁹Tc is predicted to be 7,843 pCi/L (compared to the MCL of 900 pCi/L) and will have moved about 2,682 m (8,800 ft) beyond its present location. This corresponds to a residential ILCR of 2.2 x 10⁻⁴. When it reaches the river, the concentration of ⁹⁹Tc is predicted to be higher than the existing MCL and the maximum risk will still exceed the *Model Toxics Control Act* (MTCA) standard.

5.4.2 216-B-5 Reverse Well Plume

Calculation of risk from ¹³⁷Cs and ⁹⁰Sr in the B-5 Reverse Well plume is based on the highest concentration measured in each well during the period January 1993 through March 1995. For all wells except 299-E28-25, the most recent sample collection provided the highest value.

Risk for ^{239/240}Pu in the B-5 Reverse Well plume was calculated from the analysis of groundwater samples collected during the same January 1993 through March 1995 time period. Input to the risk model was provided by the most recent sampling event, with the exception of wells 299-E28-23 and 299-E28-25, where the higher concentrations were found during earlier sampling events. The highest concentration, and coincidentally the most recent ^{239/240}Pu sampling from well 299-E28-24, was rejected as suspect data because it was over 100 times greater than the average of earlier analyses

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from that well. The well was resampled in June 1995 (after calculation of the risk) and had an activity that was 2% of the previous analysis. The next-highest concentration was used as input data for the risk calculation.

- **5.4.2.1** Existing Conditions. The risk analysis shows that no one radionuclide is consistently the major contributor to the ILCR but, rather, the major contributor differs in each monitoring well. The maximum ILCR (2.1 x 10⁻³) was calculated for groundwater at monitoring well 299-E28-23. In this well, ⁹⁰Sr is the major contributor (86% of the ILCR), with a concentration of 2,310 pCi/L. The highest ILCR because of ^{239/240}Pu alone is 1.4 x 10⁻⁴, because of 125 pCi/L of ^{239/240}Pu in groundwater at monitoring well 299-E28-25. The risk contours for the existing plume are plotted in Figure 5-31.
- 5.4.2.2 Future Conditions. Future conditions consider risk because of the B-5 Reverse Well plume COPCs at a point representing the boundary of the initial buffer zone surrounding the Central Plateau, as proposed by the Hanford Future Site Uses Working Group. Because of uncertainty in future flow conditions (due to the dissipation of a groundwater mound beneath the 216-B-3 Pond System), both northwest and southeast flow paths to the proposed buffer zone boundary were considered. The length of each flow path is approximately 6,096 m (20,000 ft). Travel-time calculations, along with radioactive decay calculations, demonstrated that ¹³⁷Cs and ⁹⁰Sr concentrations will decay to negligible levels before reaching the boundary for both flow paths. This is because of the moderate-to-high adsorption of these radionuclides combined with their relatively short half-lives. Therefore, the transport of these two isotopes did not need to be modeled; however, ^{239/240}Pu still exhibited significant concentrations at the boundary and, therefore, was modeled. The transport modeling results are tabulated as follows:

Risk Scenario	Flow Path Direction	239/240Pu Maximum Concentration (pCi/L)	Travel time (years)	Incremental Lifetime Cancer Risk
Industrial	Northwest	0.11	28,500	1.3 x 10 ⁻⁷
Industrial	Southeast	0.20	7,500	2.3 x 10 ⁻⁷
Residential	Northwest	0.11	28,500	5.7 x 10 ⁻⁷
Residential	Southeast	0.20	7,500	1.0 x 10 ⁻⁶

The risk contours for the future plume conditions are shown in Figure 5-31.

As requested by the EPA, a travel-time analysis was also performed to calculate travel distances of ¹³⁷Cs and ⁹⁰Sr at 50, 100, and 200 years into the future. Both isotopes will decay approximately three half-lives in 100 years and approximately six half-lives in 200 years. The travel-time analysis results are tabulated as follows:

Number of Years for Contaminant Migration to Occur	Distance (m) 90 Sr Migrates in Southeast Direction	Distance (m) Sr Migrates in Northwest Direction	Distance (m) 137Cs Migrates in Southeast Direction	Distance (m) 137Cs Migrates in Northwest Direction
50	198	45.7	7.6	1.5
100	396.2	91.4	15	3
200	792.5	182.9	30	6.1

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5.4.2.3 Conclusions. The risk analysis results show that the future offsite risk associated with the B-5 Reverse Well plume is well below levels of concern because of the relative immobility of the principal contaminants.

5.4.3 Gable Mountain Pond Plume

The highest ⁹⁰Sr concentrations during the period January 1993 through March 1995 were recorded during the most recent sampling of the wells that comprise the Gable Mountain Pond plume. These concentrations were used as input to the risk calculations.

- 5.4.3.1 Existing Conditions. Strontium-90 is the only COPC for the Gable Mountain Pond plume. The maximum industrial ILCR (1.8 x 10⁻⁴) is calculated for groundwater at monitoring well 699-53-48B, which had a ⁹⁰Sr concentration of 994 pCi/L. The risk contours for the existing plume are shown in Figure 5-32.
- **5.4.3.2 Future Conditions**. Based on EPA's request, the objective of transport modeling was to determine the flow path length at which the maximum ILCR for 90 Sr at any location in the plume attenuates to, below 1 x 10^{-5} , as the plume migrates to the northwest. Modeling results are tabulated as follows:

	Industrial	Residential
Maximum ⁹⁰ Sr source concentration (existing plume), pCi/L	1,008	1,008
Travel time to reach 1.0 x 10 ⁻⁵ ILCR, yr	79	127
Travel distance to reach 1.0 x 10 ⁵ ILCR, m	914.4	1,372
⁹⁰ Sr concentration at 1.0 x 10 ⁻⁵ ILCR, pCi/L	57	13

The risk contours for the plume based on future conditions are shown in Figure 5-32.

5.4.3.3 Conclusions. The risk analysis results show that the Gable Mountain Pond plume will migrate slowly toward the northwest, but the principal contaminant ⁹⁰Sr will decay to negligible levels before the plume travels a significant distance.

5.5 UNCERTAINTIES AND DATA NEEDS

This section identifies the uncertainties in the conceptual model for each plume in the 200-BP-5 Operable Unit and the data needs that were recognized during the treatability test operations and associated groundwater studies.

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5.5.1 216-B-5 Reverse Well Plume

The major uncertainty in the conceptual model of the B-5 Reverse Well Contaminant Plume is the physical-chemical form of the contaminants. The contaminants at the B-5 Reverse Well are expected to be highly insoluble, but this cannot be confirmed from physical samples. The anions present in the aquifer that combined with the plutonium, cesium, and strontium cations determine the solubility of the plume source. This, along with the character of the aquifer sediments and groundwater flow, may control the mobility and concentration of the contaminant plume. Physical samples from drill cuttings or core are needed to determine the K_ds of the contaminant source material.

Adequate monitoring well coverage and sampling are needed at the B-5 Reverse Well location to ensure detection of migration of the contaminant plume in any direction. The B-5 Reverse Well is located in an area with a very flat groundwater gradient, and no preferential flow direction has been defined. Groundwater flow to the southeast was recognized during the early years of plant operations. Currently, wells to the south and southeast of the B-5 Reverse Well (wells 299-E28-14 and 299-E28-4) are not in sample-ready condition and could not detect contamination spread in those directions. A series of monitoring wells constructed closer to the B-5 Reverse Well would provide an early warning if the contaminants were more mobile than expected.

5.5.2 216-BY Cribs Contaminant Plume

The first uncertainty about the BY-Cribs plume is the conceptual model for this plume and whether pockets of more highly concentrated contaminants are contributing to the growth of the plume. As discussed in Section 5.3.2, there are at least two models with which to explain the distribution of contaminants within the current plume. The existence of high-activity contaminants in sediments or basalt fractures near or currently above the water table or as dense solutions within basalt fractures at the bottom of the aquifer remains uncertain.

The BY-Cribs plume is relatively confined and thin, due in part to the configuration of the surface of the underlying basalt. Erosion of the upper surface of the basalt may have resulted in basalt highs that divert the flow of groundwater or, conversely, erosional channels that tend to provide preferential flow paths for the groundwater. These erosional channels might be the optimal target for extraction well locations because of the potential increased aquifer thickness.

The continued monitoring of the BY-Cribs plume will remain a data need as regular monitoring will be necessary to detect changes in the plume configuration or concentration.

5.5.3 216-A-25 Gable Mountain Pond Plume

There is uncertainty about groundwater flow at the Gable Mountain Pond plume and the potential for contaminant migration before radioactive decay reduces its level of risk. The monitoring well network at the Gable Mountain Pond appears to be adequate to recognize contaminant migration, but should be confirmed by flow modeling. In addition, a regular sampling program must be maintained.

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Figure 5-1. 200 East Processing Areas.

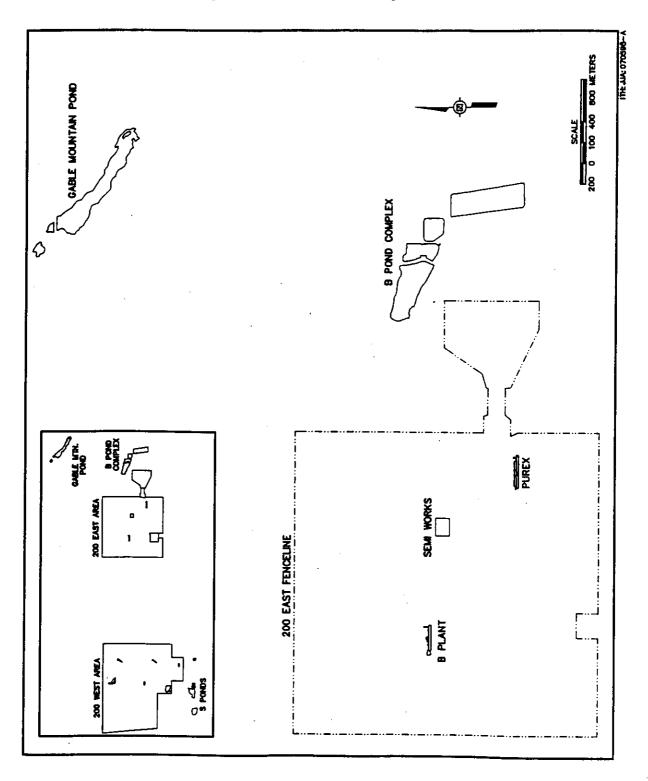
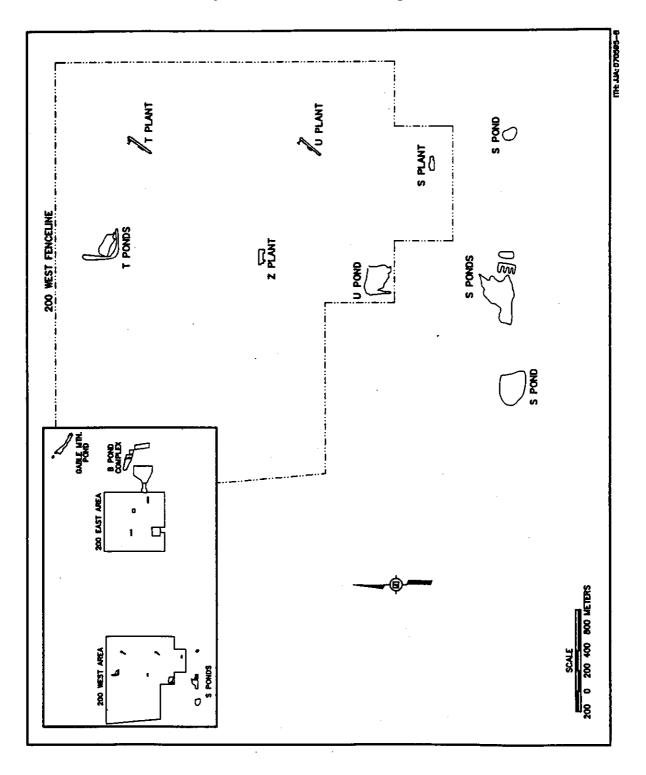


Figure 5-2. 200 West Processing Areas.



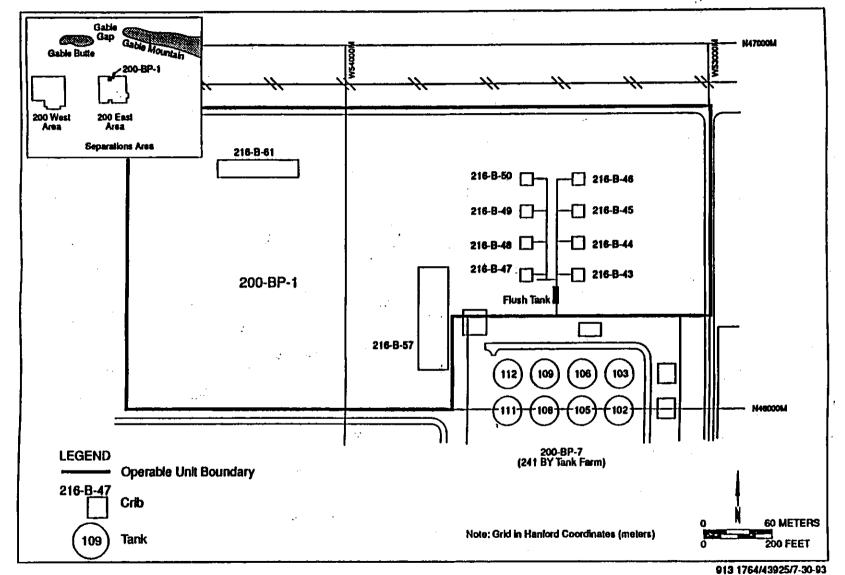


Figure 5-3. The BY-Cribs, 200-BP-1 Operable Unit.

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Figure 5-4. Estimated Annual Discharge (in liters) to Gable Mountain Pond (216-A-25), and B Pond System (216-B-3 Ponds), 200 East Area, 1945 to 1994.

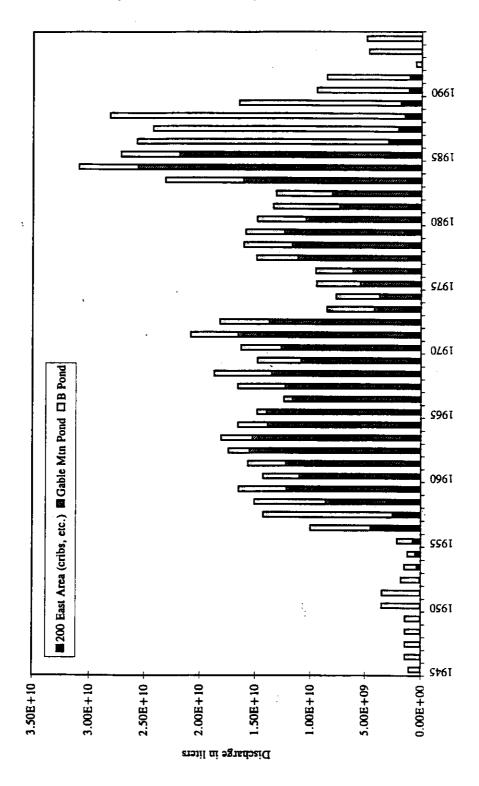


Figure 5-5. Ditches and Pipelines Used For Effluent Transport to the 216-B-3 Pond System.

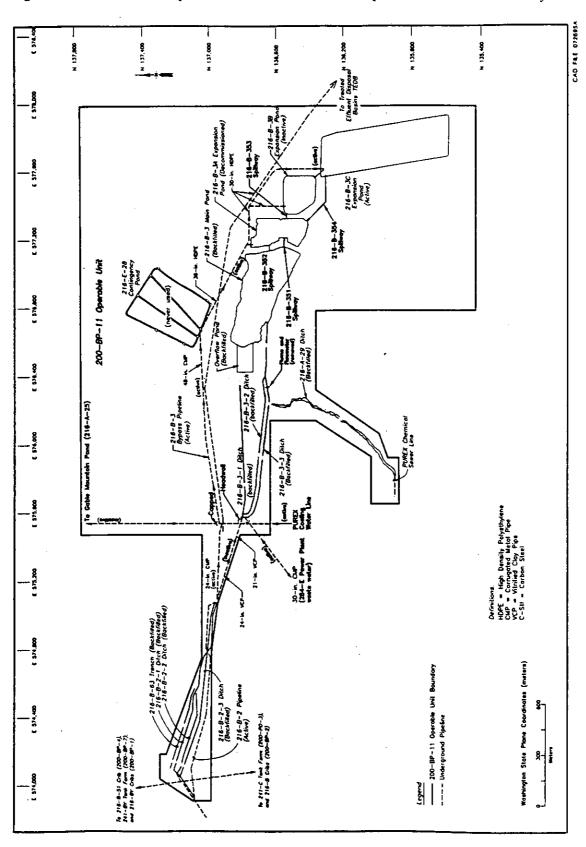


Figure 5-6. Estimated Annual Discharges (in liters) to S Ponds, T Pond, and U Pond, 200 West Area, 1944 to 1993.

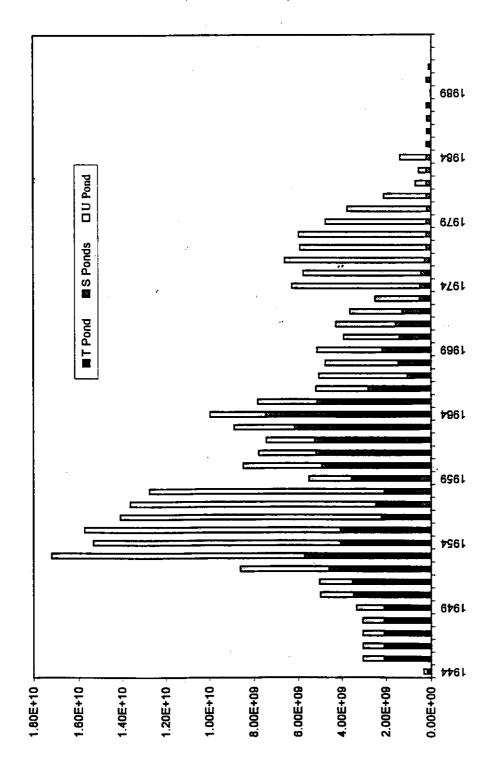
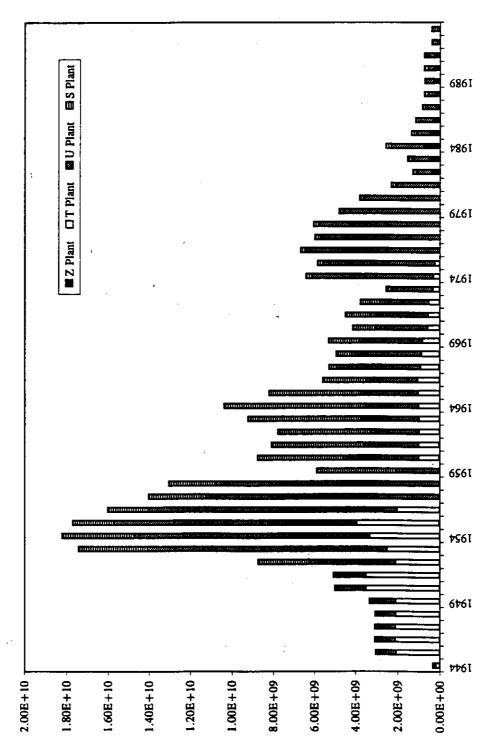
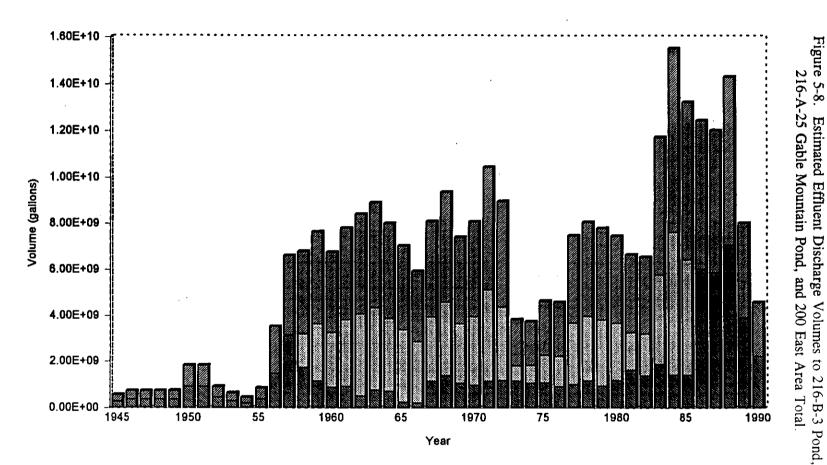


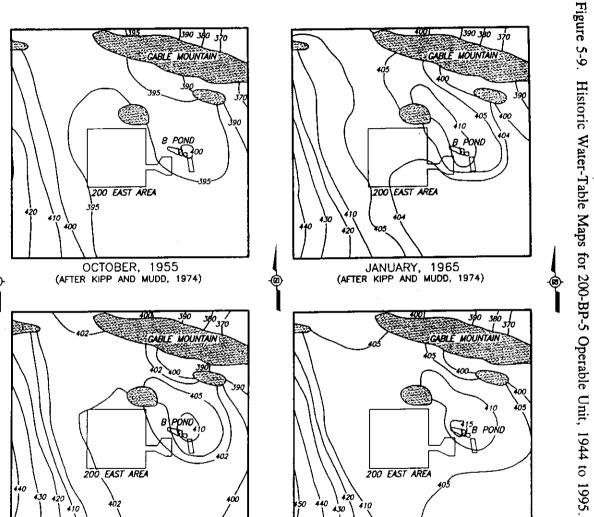
Figure 5-7. Estimated Annual Discharge (in liters) to S Plant, T Plant, U Plant, and Z Plant, 1944 to 1993.





■B POND ■GABLE MTN POND Ø2 EAST TOTAL

(2 Sheets)



200 EAST AREA

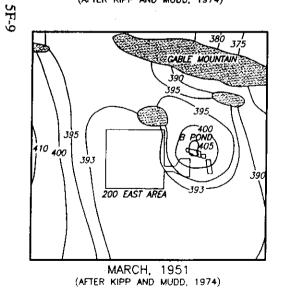
MAY, 1970 (AFTER KIPP AND MUDD, 1974)

430 410

390 390 370

380 GABLE MOUNTAIN B POND 200 EAST AREA JANUARY, 1944

(AFTER KIPP AND MUDD, 1974)



200 EAST AREA 440 130 420 \ 400

JUNE, 1960 (AFTER KIPP AND MUDD, 1974)

Figure 5-9. Historic Water-Table Maps for 200-BP-5 Operable Unit, 1944 to 1995. (2 Sheets)

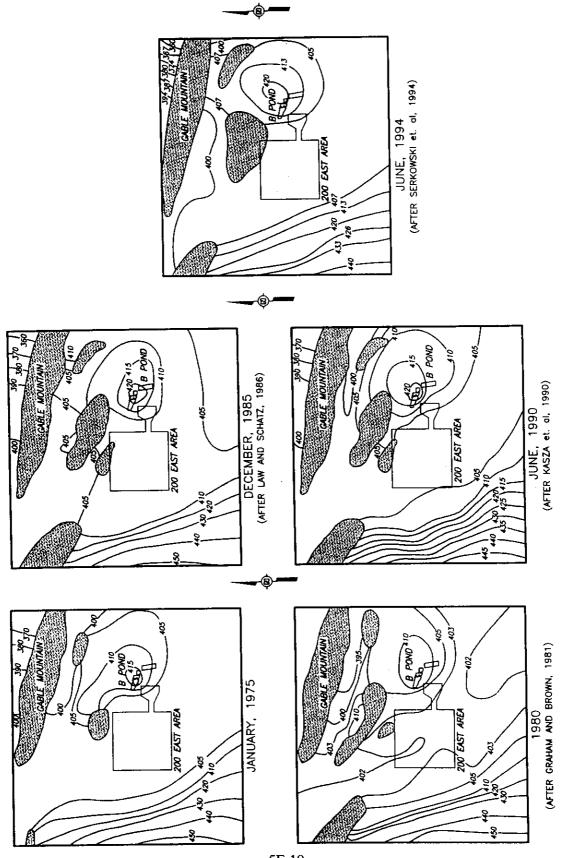
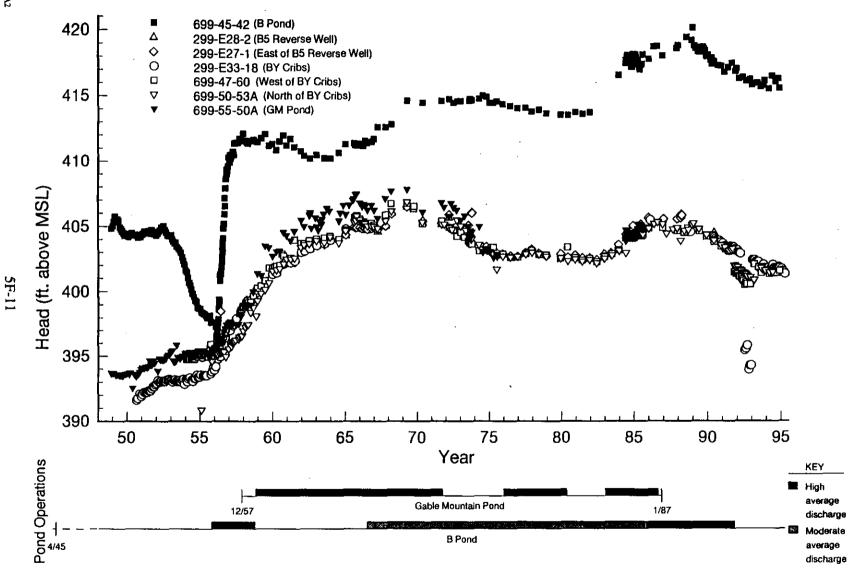


Figure 5-10. Historic Groundwater Elevation Changes for 200-BP-5 Operable Unit at Selected Wells, 1948 to 1995.



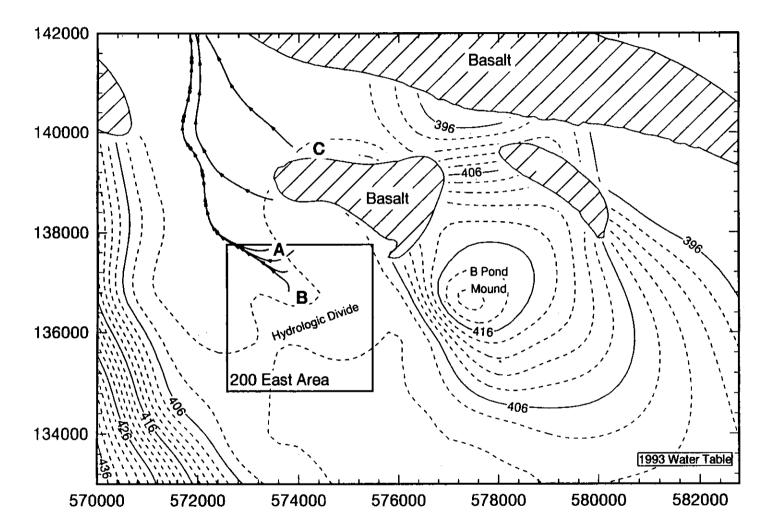


Figure 5-11. Streamline Analysis for 200 East Area, 1993.

Figure

5-12.

Historic

Gross

Beta

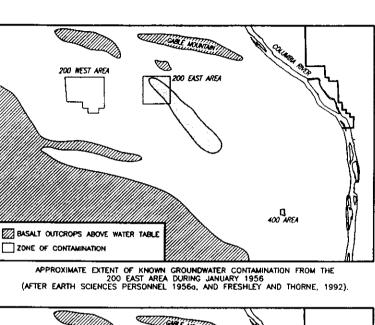
Distribution Maps

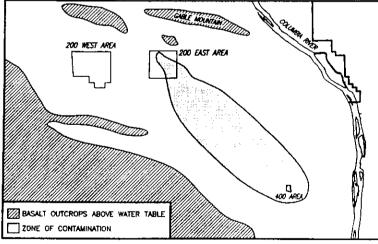
in 200 Areas,

1956 to

1992.

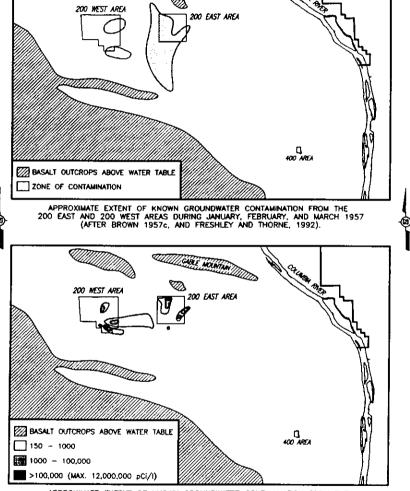
Sheets)





APPROXIMATE EXTENT OF KNOWN GROUNDWATER CONTAMINATION FROM THE 200 EAST AREA DURING APRIL 1956 (AFTER EARTH SCIENCES PERSONNEL 1956b, AND FRESHLEY AND THORNE, 1992).

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APPROXIMATE EXTENT OF KNOWN GROUNDWATER CONTAMINATION FROM THE 200 EAST AND 200 WEST AREAS DURING OCTOBER, NOVEMBER, AND DECEMBER 1958 (AFTER BIERSCHENK 1959», AND FRESHLEY AND THORNE, 1992).

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Figure 5-12.

Historic

Gross

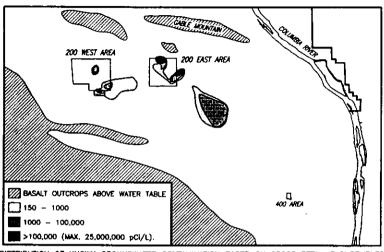
Beta

Distribution Maps

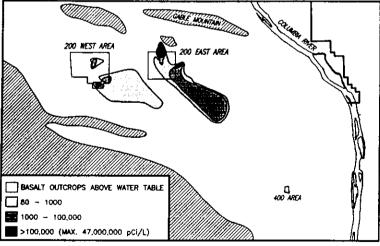
in 200 Areas,

1956 to 1992.

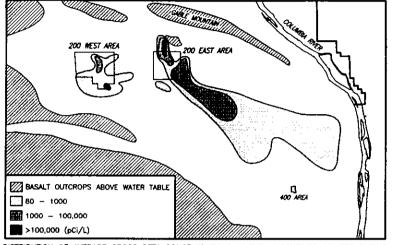
(4 Sheets)



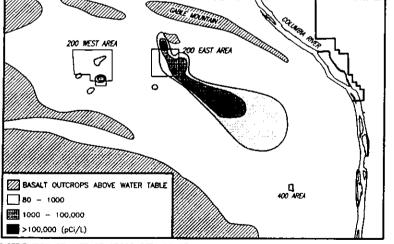
DISTRIBUTION OF KNOWN GROUNDWATER CONTAMINATION BASED ON GROSS BETA MEASUREMENTS FOR SEPTEMBER 1959 (AFTER HANEY 1959, AND FRESHLEY AND THORNE, 1992).



DISTRIBUTION OF KNOWN GROUNDWATER CONTAMINATION BASED ON GROSS BETA MEASUREMENTS FOR OCTOBER, NOVEMBER, AND DECEMBER 1959 (AFTER HANEY 1960g, AND FRESHLEY AND THORNE, 1992).



DISTRIBUTION OF AVERAGE GROSS BETA CONCENTRATIONS IN THE UNCONFINED AQUIFER DURING JULY THROUGH DECEMBER 1962 (AFTER BROWN 1963, AND FRESHLEY AND THORNE, 1992).



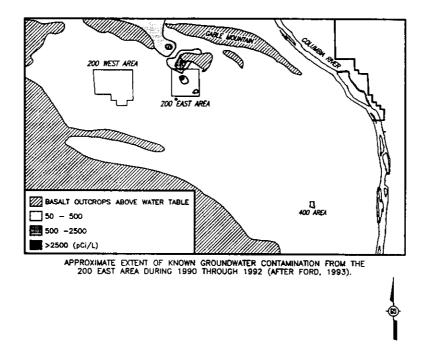
DISTRIBUTION OF AVERAGE GROSS BETA CONCENTRATIONS IN THE UNCONFINED AQUIFER DURING JANUARY THROUGH JUNE 1963 (AFTER HANEY 1963, AND FRESHLEY AND THORNE, 1992).

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2E-073095K

Figure 5-12. Historic Gross Beta Distribution Maps in 200 Areas, 1956 to 1992. (4 Sheets) MS608:20:3Z APPROXIMATE EXTENT OF KNOWN GROUNDWATER CONTAMINATION FROM THE 200 EAST AREA DURING JANUARY THROUGH DECEMBER 1985 (AFTER LAW AND SCHATZ, 1986). DISTRIBUTION OF AVERAGE GROSS BETA CONCENTRATIONS IN THE UNCONFINED AQUIFER DURING 1974 (RAYMOND et al. 1976, AND FRESHLEY AND THORNE, 1992). 100 APEA too AREA 200 EAST ARE BASALT OUTCROPS ABOVE WATER TABLE 200 MEST ARE ZONE OF CONTAMINATION DISTRIBUTION OF AVERACE GROSS BETA CONCENTRATIONS IN THE UNCONFINED AQUIFER DURING JANUARY THROUGH DECEMBER 1964 (ATER BROWN 1964, AND FRESHLEY AND THORNE, 1992). DISTRIBUTION OF AVERAGE GROSS BETA CONCENTRATIONS IN THE UNCONFINED AQUIFER DURING JULY THROUGH DECEMBER 1965 (AFTER ELIASON 1966a, AND FRESHLEY AND THORNE, 1992). 138Y 007 200 EAST APPEA BASALT OUTCROPS ABOVE WATER TABLE >100,000 (pci/l) FROM RIVER RECHARGE AND GROUND DISPOSAL >100,000 (pci/l)
FROM RIVER RECHARGE AND
GROUND DISPOSAL BASALT OUTCROPS ABOVE - 100,000 1000 - 100,000 80 - 1000

Figure 5-12. Historic Gross Beta Distribution Maps in 200 Areas, 1956 to 1992. (4 Sheets)



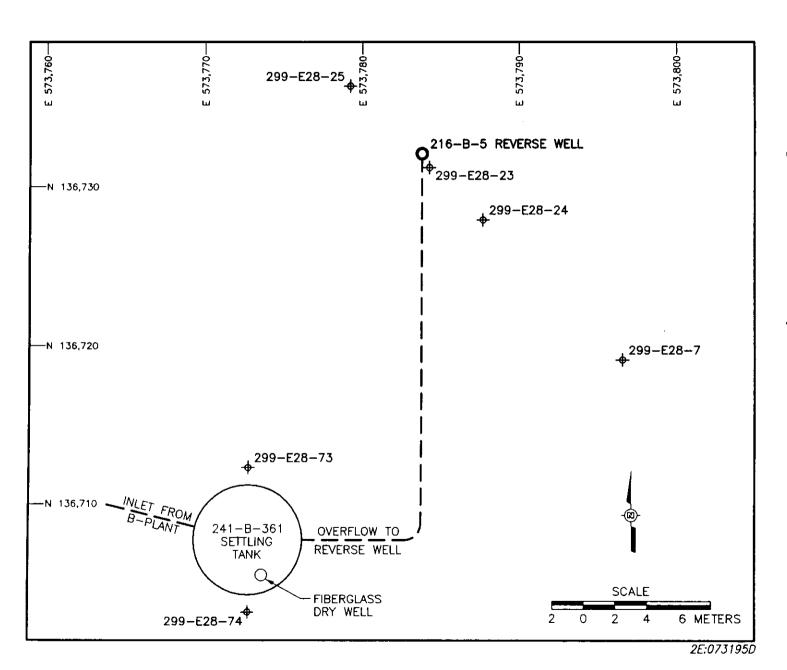


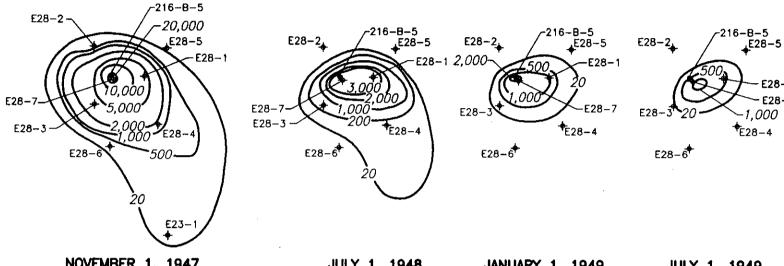
Figure 5-13. Well and Facility Locations, 216-B-5 Reverse Well.

5F-18

Estimated Radionuclide Distributions at 216-B-5 Reverse Well, 1947 to 1949

Figure 5-14.

FISSION PRODUCTS CONTAMINATION



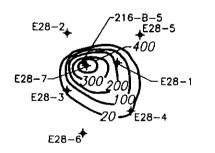
NOVEMBER 1, 1947

JULY 1, 1948

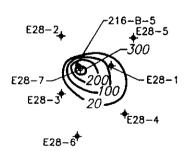
JANUARY 1, 1949

JULY 1, 1949

ALPHA CONTAMINATION



NOVEMBER 1, 1947

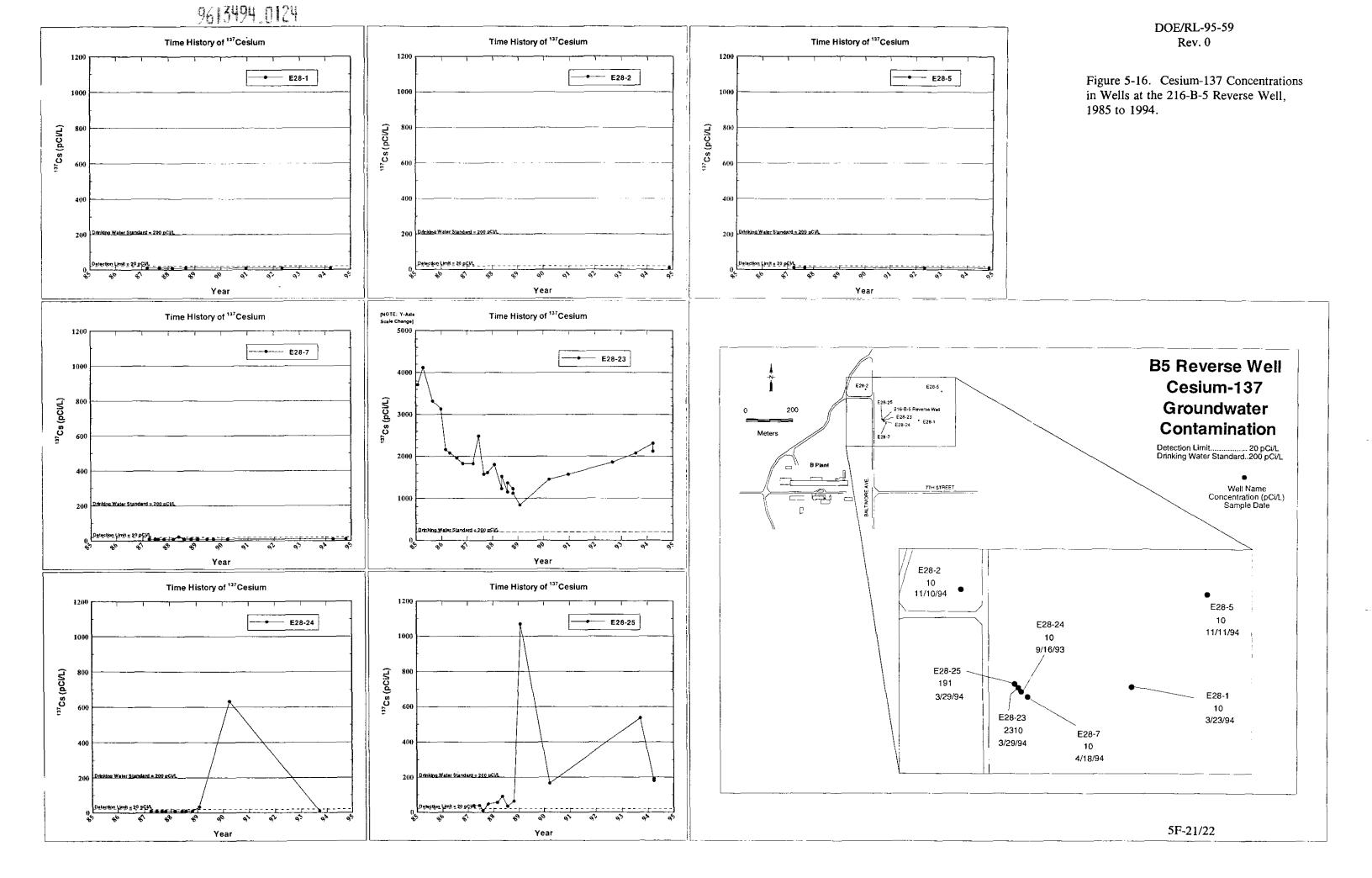


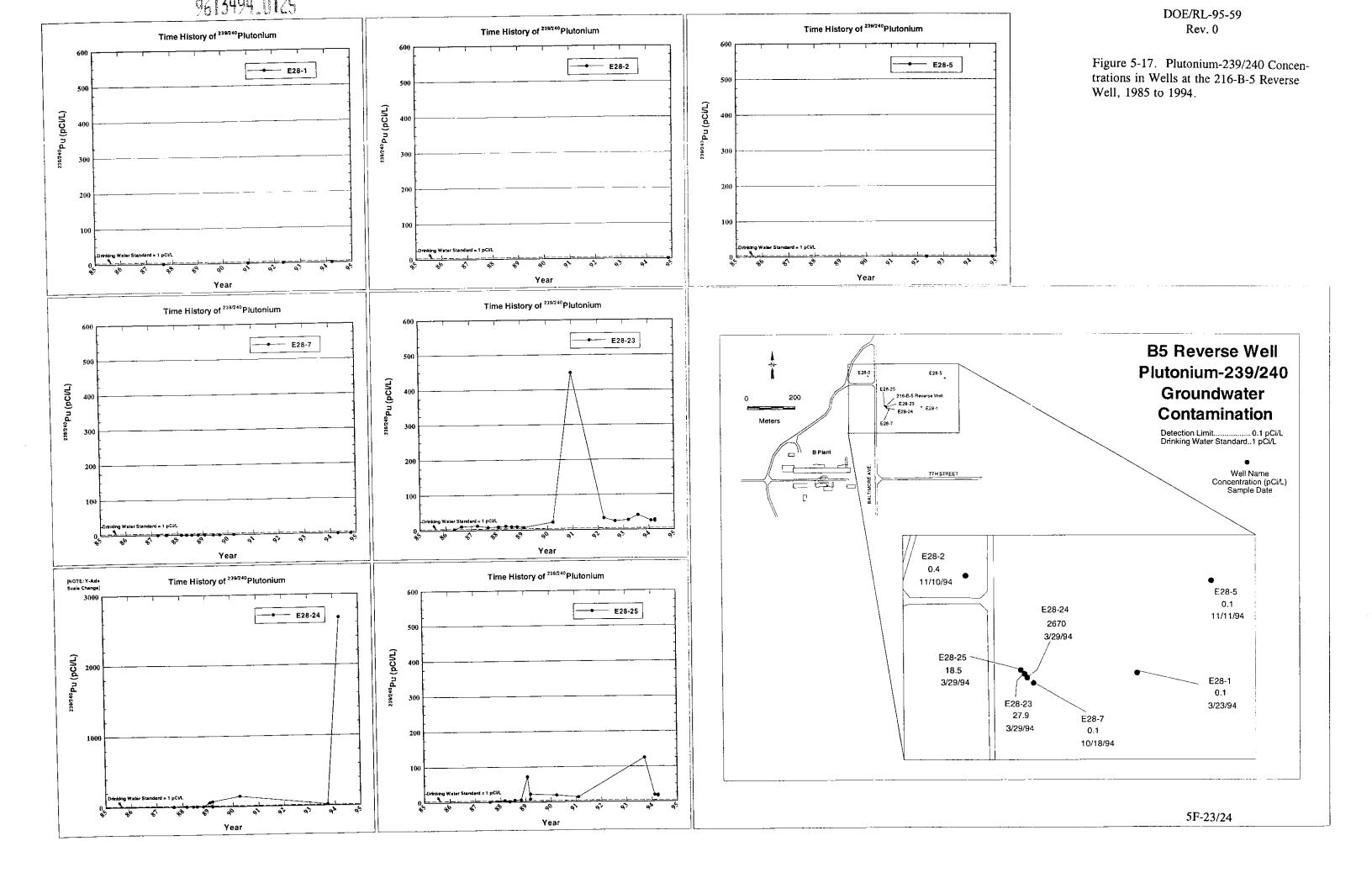
JULY 1, 1949

EXPLANATION

CONTOUR INTERVAL AS INDICATED ACTIVITY MEASURED IN pCi FISSION PRODUCTS PER LITER OF WATER, AND IN DIS/MIN/LITER FOR URANIUM. SIGNIFICANT LEVEL OF ACTIVITY CHOSEN AT 20 pCi/LITER AND 10 DIS/MIN/LITER FOR FISSION PRODUCTS AND URANIUM RESPECTIVELY.

WELLS TO WATER





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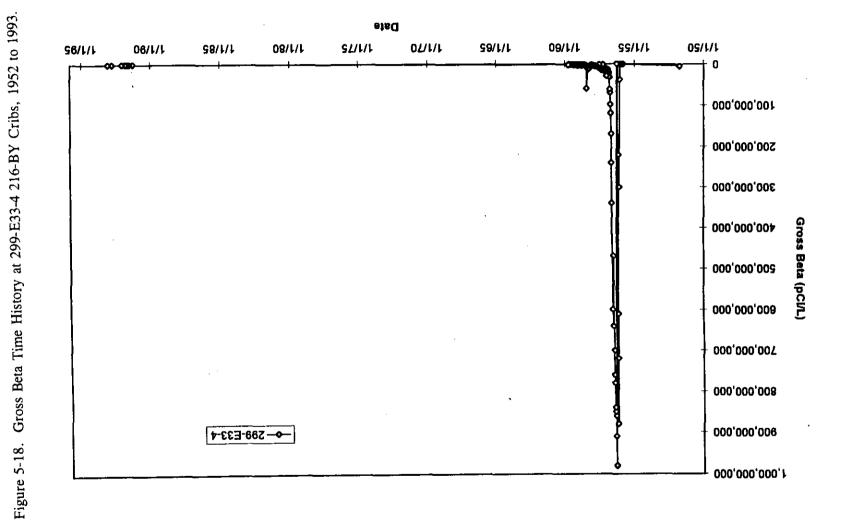
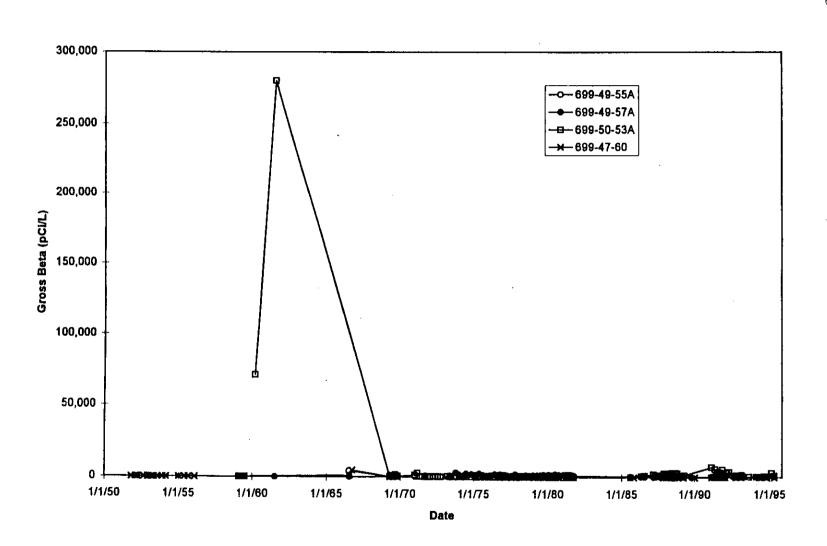
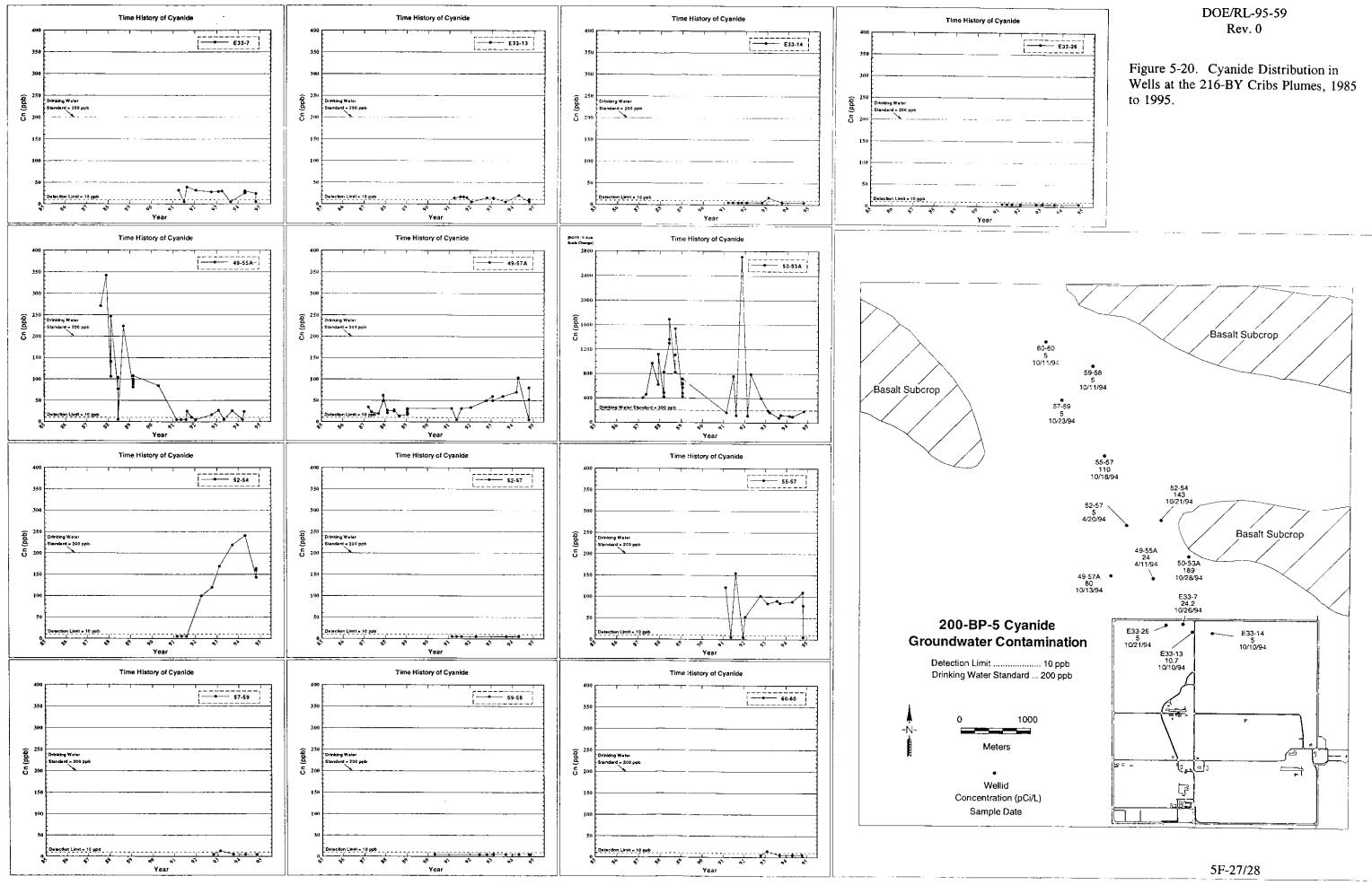
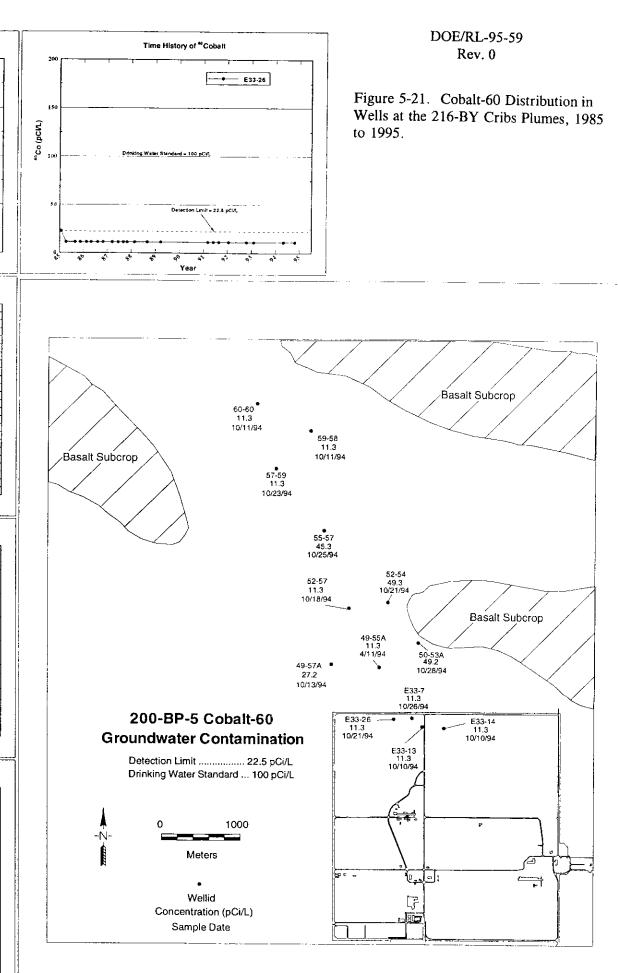


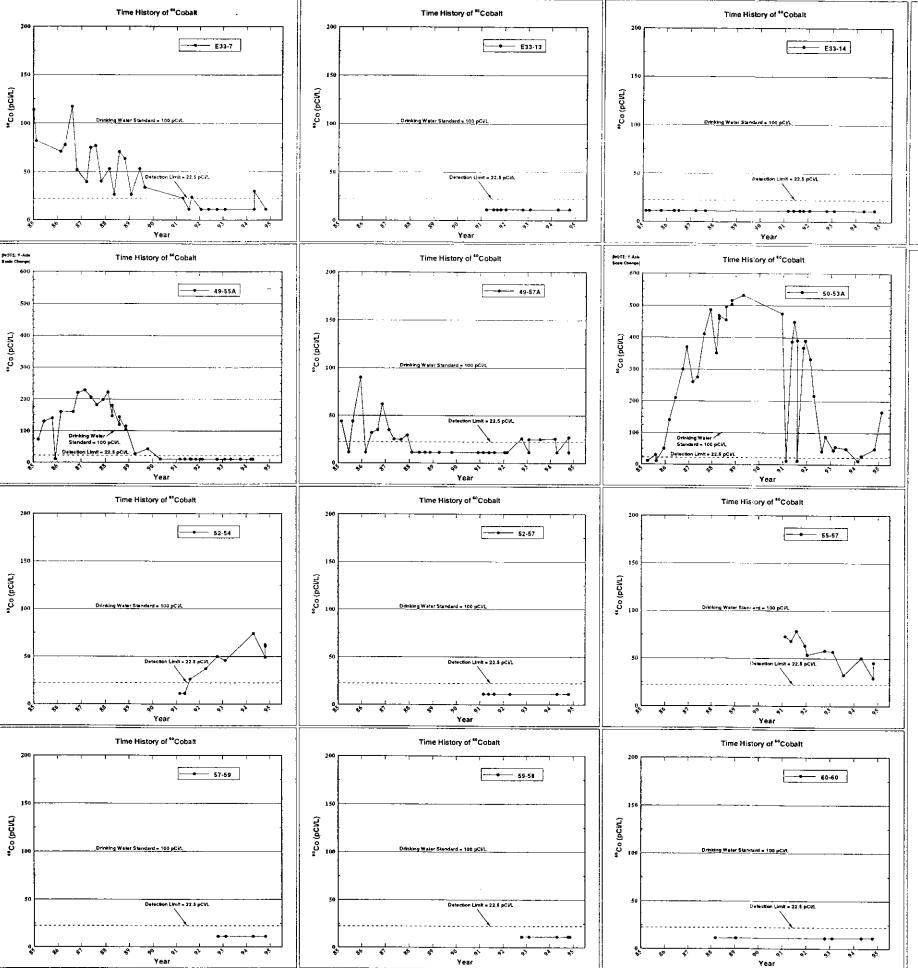
Figure 5-19. Gross Beta Time History for Wells within 216-BY Cribs Plumes, 1952 to 1995.

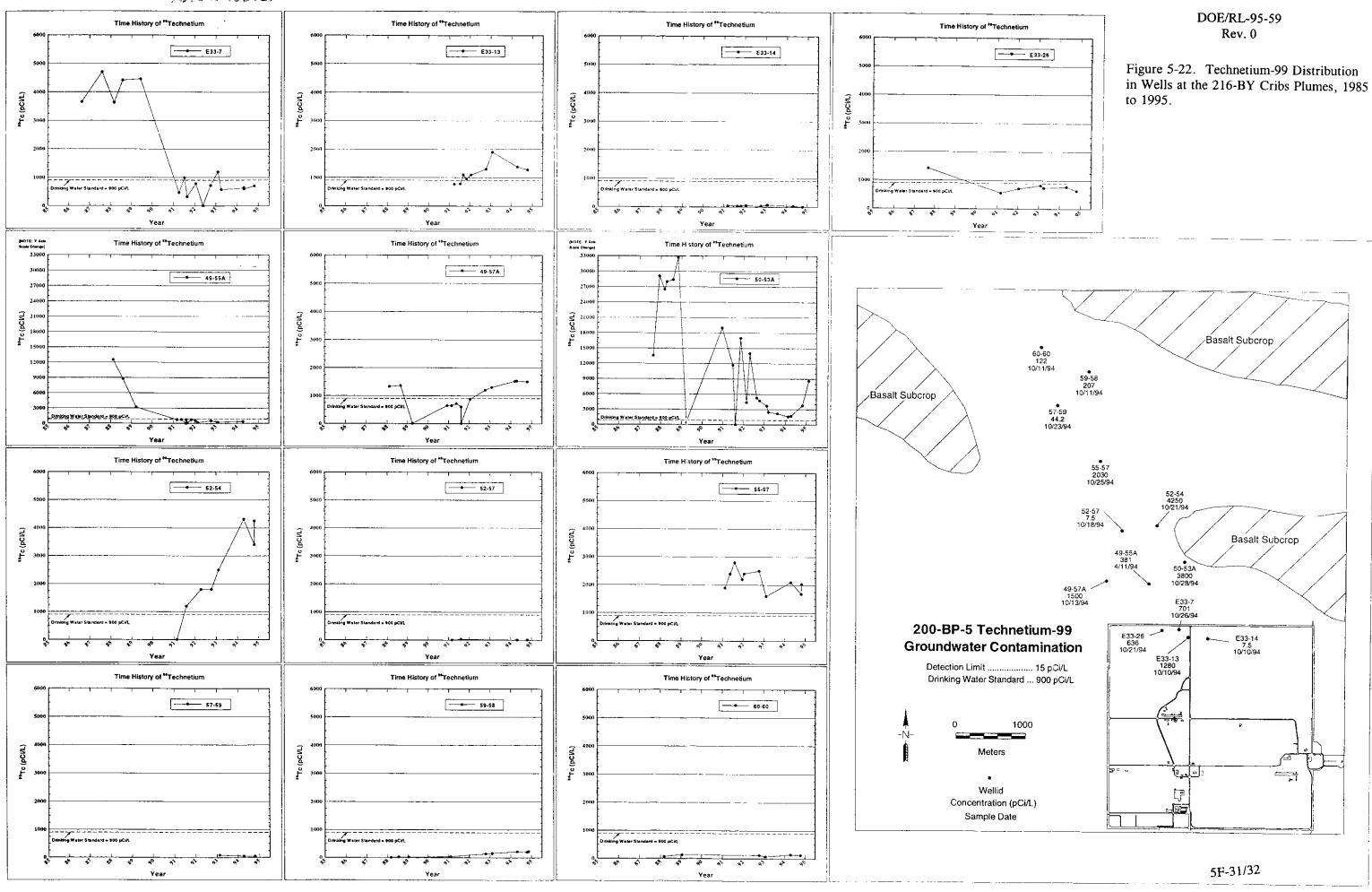






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Figure 5-23. Gross Beta Time History at Well 699-55-50A, 215-A-25 Gable Mountain Pond, 1952 to 1990.

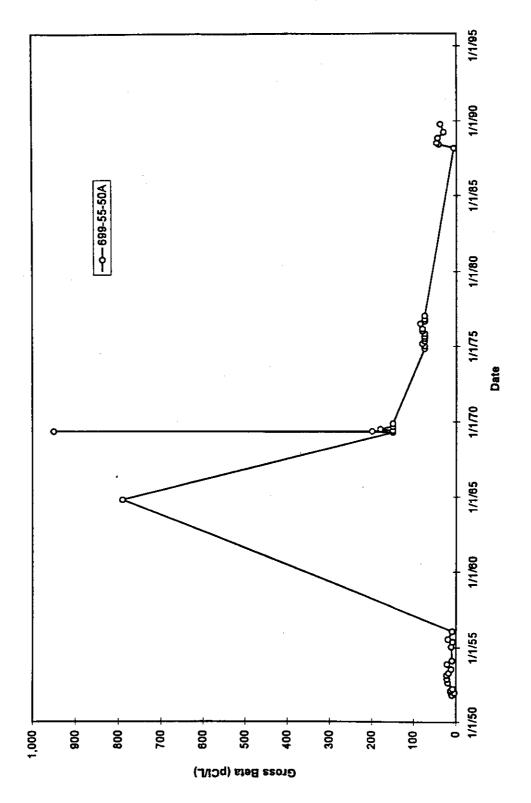
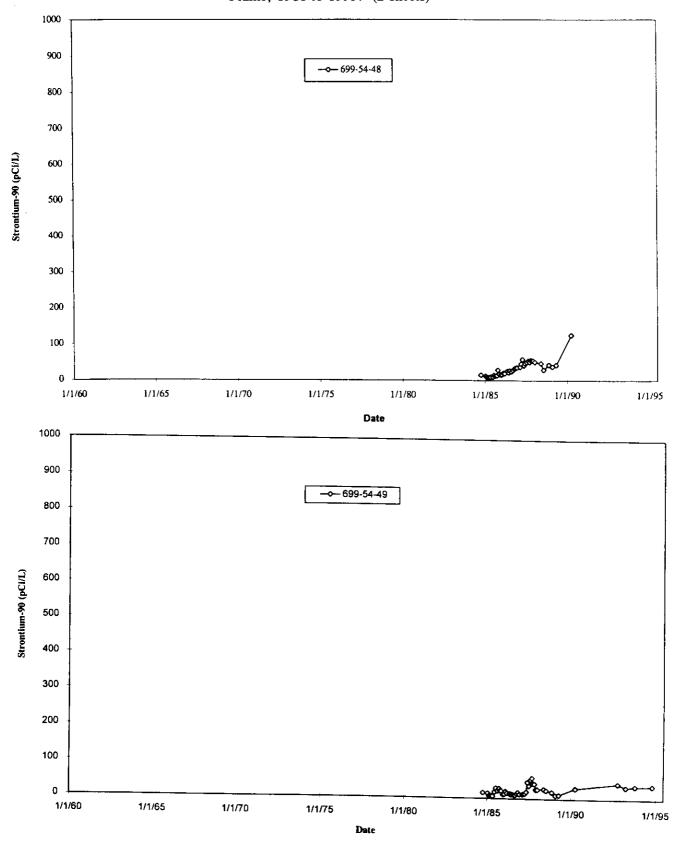
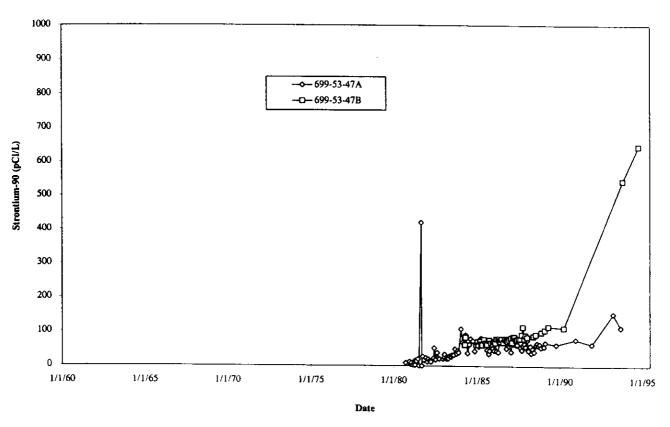


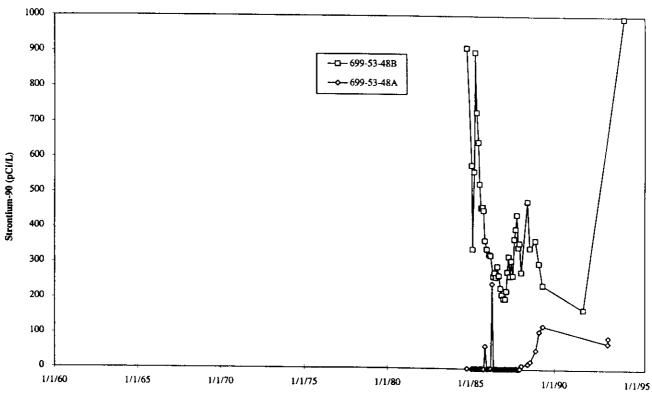
Figure 5-24. Strontium-90 Time History at Wells in the 216-A-25 Gable Mountain Pond Plume, 1980 to 1995. (2 sheets)



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Figure 5-24. Strontium-90 Time History at Wells in the 216-A-25 Gable Mountain Pond Plume, 1980 to 1995. (2 sheets)





Date 5F-35

Figure 5-25. Gable Mountain Pond Strontium-90 Plume: Data are from 1994.

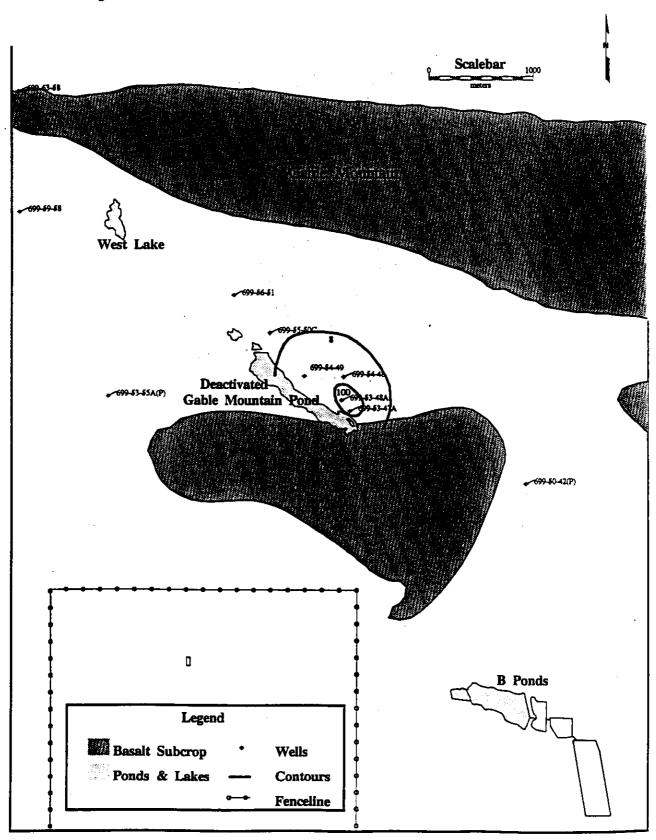


Figure 5-26. Site Map of the Deactivated 216-A-25 Gable Mountain Pond Showing Well Locations.

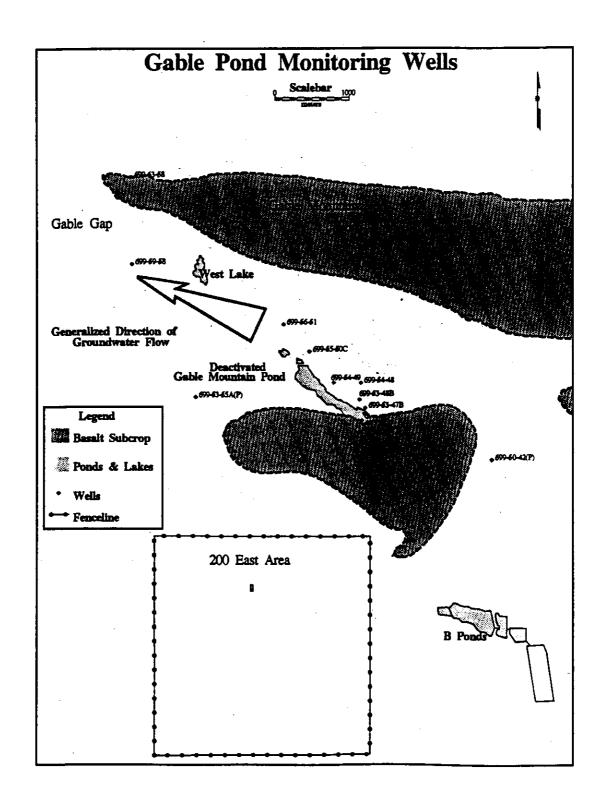


Figure 5-27. History of Strontium-90 Concentrations in Groundwater and Water-Table Elevations in Wells 699-53-47B and 699-53-48B, Gable Mountain Pond.

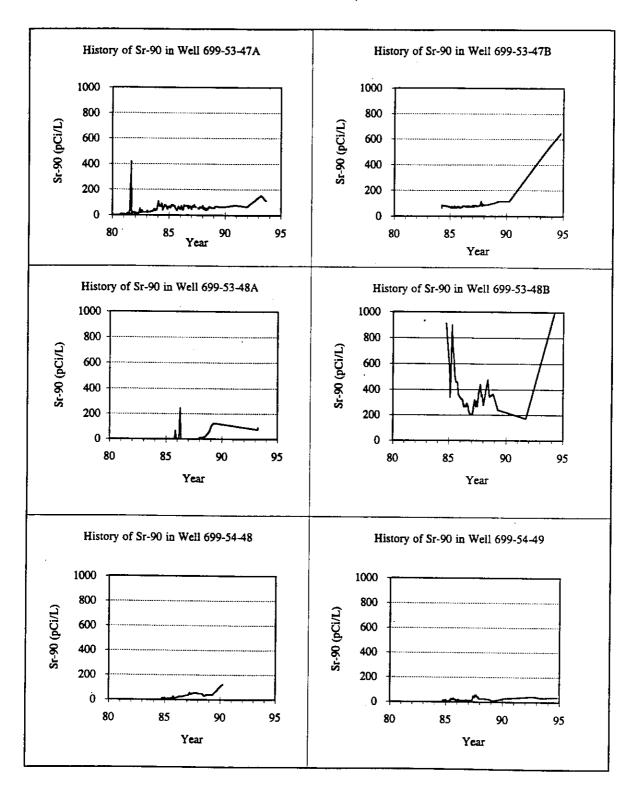


Figure 5-28. History of Strontium-90 Concentrations in Groundwater at Gable Mountain Pond.

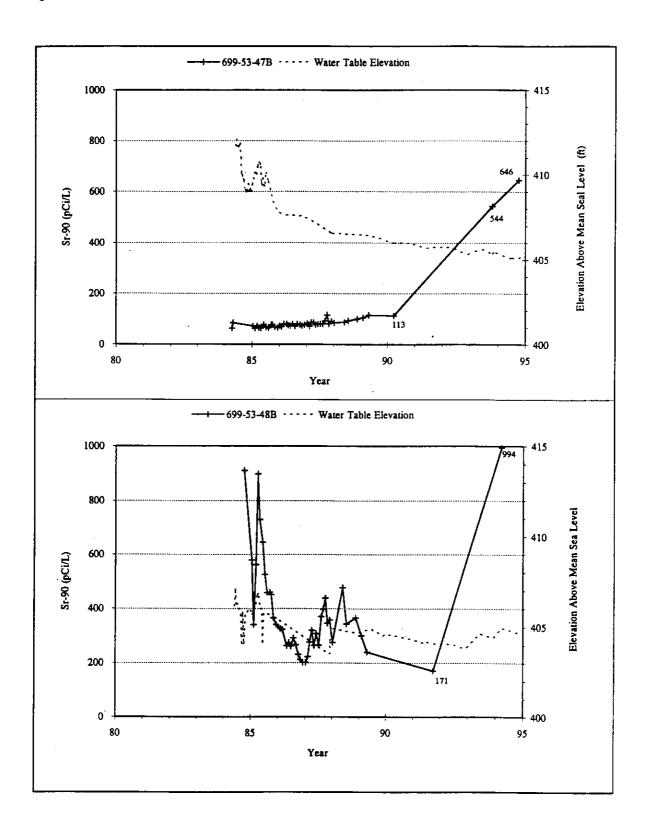


Figure 5-29. History of Cesium-137 Concentrations in Groundwater at Gable Mountain Pond.

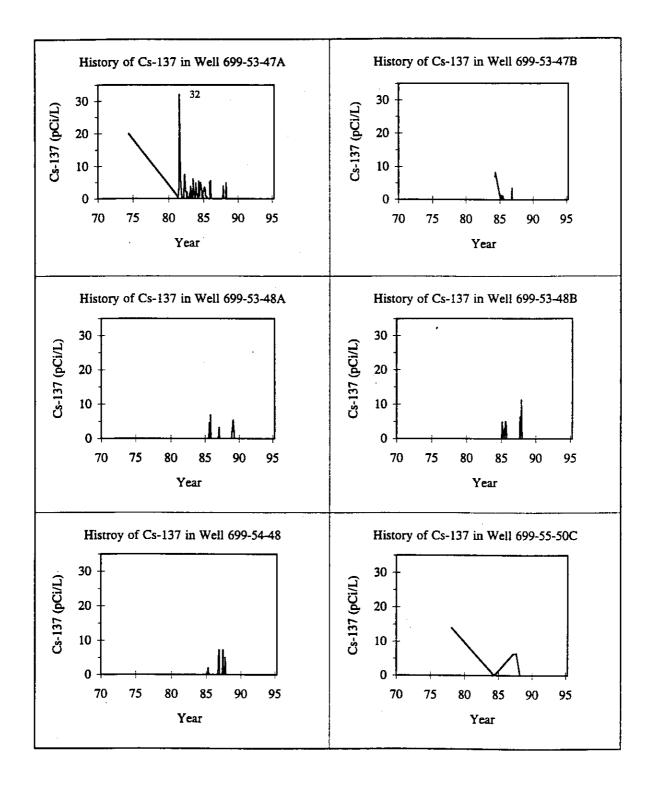


Figure 5-30. 216-BY Cribs Plume Incremental Lifetime Cancer Risk for Reasonable Maximum Exposure to Technetium-99 in Groundwater, Existing and Future Conditions.

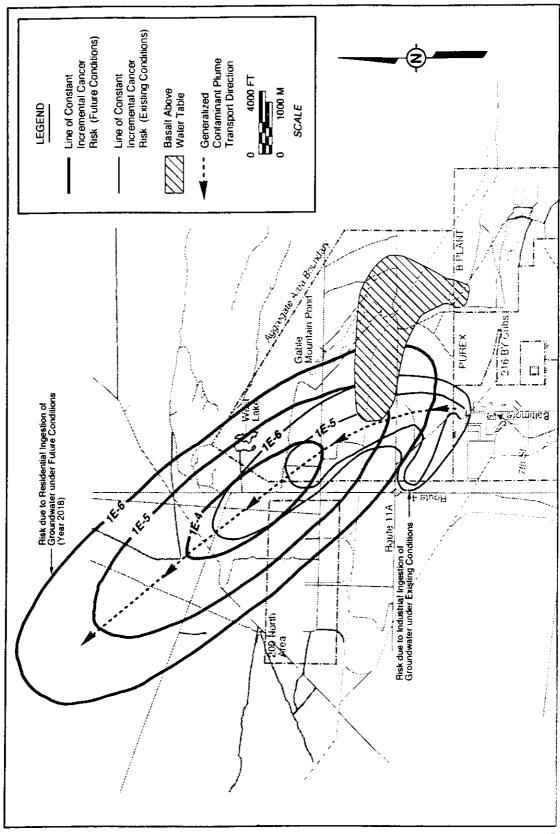


Figure 5-31. Maximum Exposure to Technetium-99 in Groundwater, Existing and Future Conditions. 215-B-5 Reverse Well Plume Incremental Lifetime Cancer Risk for Reasonable

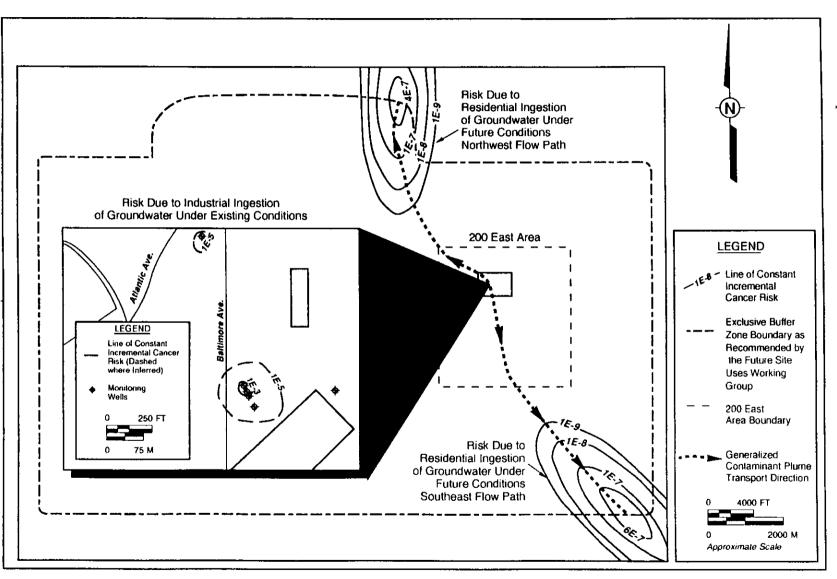
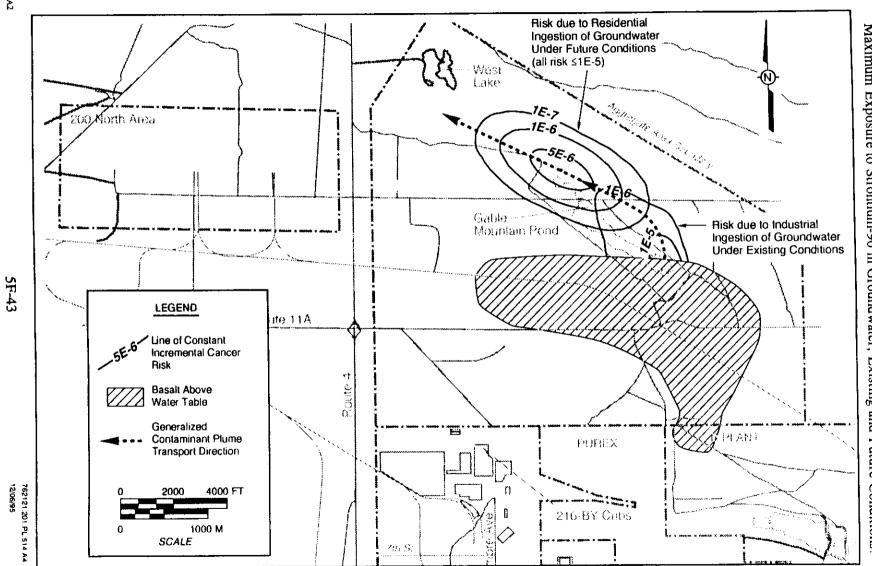


Figure 5-32. Maximum Exposure to Strontium-90 in Groundwater, Existing and Future Conditions Gable Mountain Pond Plume Incremental Lifetime Cancer Risk for Reasonable



5F-44

Waste Volume

Received (1)

3.38 E+07

5.48 E+07

8.44 E+07

 $3.06 E \pm 07$

October 1947 other liquid waste via Tank 5-6 in 221-B. April 1945 to 216-B-3 Pond Steam condensate and process cooling water from 221-B, cooling December 1994 water from Waste Encapsulation Storage Facility, water from October 1983 to 284-E Powerhouse, cooling water from 242-A, 244-CR, and 216-B-3A Pond 2.40 E+11 December 1994 244-AR, process, condenser, and air sampler vacuum pump cooling water from 202-A, chemical sewer effluent and fractionator 216-B-3B Pond June 1984 to 1985 condensate from 202-A. 1985 to Present 216-B-3C Pond Process cooling water from 202-A, contact condenser cooling water from 241-A-431, surface condenser cooling water from 241-A-401, December 1957 to 284-E Powerhouse water, cooling water and steam condensate from 3.07 E+11 216-A-25 Pond

B Plant cooling water.

Table 5-1. Summary of Waste Management Units Pertinent to 200-BP-5 Treatability Test.

B-5 Reverse Well in the 241-BY Tank Farms.

uranium recovery operations.

Tank Farm.

Source Description

Waste storage tank condensate from the in-tank solidification (ITS)

Waste storage tank condensate from the ITS BY-Cribs in the 241-BY

Supernatant overflow from the 216-B-361 Settling Tank waste via

244-AR Vault, steam condensate cooling water from 242-A, and

Tank 5-6 in 221-B and liquid waste from 224-B. Cell drainage and

Scavenged tributyl phosphate supernatant from 221-U during

Source: WHC (1991). Source: Maxfield (1979).

Waste Management

Unit

216-B-43 Crib through

216-B-5 Reverse Well

216-B-49 Crib

216-B-50 Crib

216-B-57 Crib

Years in Service

November 1954 to

December 1955

January 1965 to

January 1974

February 1968 to

June 1973

April 1945 to

1987

5T-1

5T-2

Waste Management

Quantity of Reported Radionuclides (Ci)^a

9.39 E+02

2.75 E+01

Unit Number and Type	²⁴¹ Am	⁶⁰ Co	⁹⁰ Sr	¹³⁷ Cs	Plutonium Total (grams)	²³⁸ Pu	²³⁹ Pu	²⁴⁰ Pu	
216-B-43 through 216-B-49 Crib	0.00 E+00	4.06 E-02 ^b	4.43 E+02	1.64 E+03	7.05 E+01	0.00 E+00	1.01 E+03 ^h	1.09 E+00b	
216-B-50 Crib	0.00 E+00	2.83 E-02 ^b	3.39 E+00	5.12 E+01	2.39 E-01	0.00 E+00	1.36 E-02 ^b	3.68 E-03 ^b	
216-B-57 Crib	0.00 E+00	1.47 E-02 ^b	1.83 E+00	2.26 E+02	1.87 E-01	0.00 E+00	1.06 E-02 ^b	2.87 E-03	
216-B-5 Reverse Well	0.00 E+00	0.00 E±00	2.55 E+01	2.92 E+01	4.27 E+03	0.00 E+00	2.44 E+02 ^b	6.57 E+01 ^b	
216-A-25 Pond	5.28 E-04		2.57 E+02	2.04 E+02	4.28 E+02				
216-B-3 Ponds ^c	3.96 E+00		1.01 E+02	9.35 E+01	2.50 E+02	2.60 E-03	7.99 E-01		
Waste Management Unit Number and Type	Quantity of Reported Radionuclides (Ci) ^a								
	²⁴¹ Pu	¹⁰⁶ Ru	Total Uranium	³ H	Alpha	Beta	²³⁸ U	²³⁵ U	
216-B-43 through 216-B-49 Crib	0.00 E+00	0.00 E+00	1.80 E-01	2.41 E+03 ^b	2.65 E+02	1.04 E+04	1.80 E-01 ^b	0.00 E+00	
216-B-50 Crib	0.00 E+00	0.00 E+00	9.50 E-05	9.00 E+01	1.47 E-02	1.05 E+02	1.00 E-04	0.00 E+00	
216 D 57 C-ib	 	0.00 5 .00	207 5 04	0.00 E ± 00	1.15 E-02	4.37 E+02	2.90 E-04	0.00 E + 00	
216-B-57 Crib	0.00 E+00	0.00 E+00	2.97 E-04	0.00 E+00	1.13 E-02	4.37 L 102	2.70 L 07	0.00 E 1 00	

2.13 E+02

NOTES:

216-A-25 Pond

216-B-3 Ponds^c

1.62 E-04

4.24 E+00

^{*}Values decayed through December 31, 1989 unless otherwise noted.

^bValues are decayed through April 1, 1986.

^cUnplanned releases UN-200-E-32, UN-200-E-34, and UN-200-E-138 contained approximately 21,000 Ci of activity.

⁻⁻ indicates no inventory data available.

Table 5-3. Chemical Waste Inventory Estimates.

Waste Management Unit Number and Type	Quantity of Reported Chemicals (kg) ^a									
	AlNO ₃	F	FeCN	H ₂ SO ₄	HNO ₃	K	Na	NaAl		
216-B-43 Crib through 216-B-49 Crib			2.27 E+04				2.65 E+06			
216-B-50 Crib				<u>.</u> _			5.00 E+02			
216-B-57 Crib										
216-B-5 Reverse Well	5.00 E+03	5.00 E+04				8.00 E+04				
216-A-25 Pond										
216-B-3 Ponds										
Waste Management	Quantity of Reported Chemicals (kg) ^a									
waste management			Quan	tity of Kepoi	rted Unemicals	(Kg)				
Unit Number and Type	NaCr ₂ O ₇	NH ₄ CO ₃	NH ₄ NO ₃	NO ₂	NO ₃	Oxalate	PO ₄	SO ₄		
Unit Number and	NaCr ₂ O ₇	NH ₄ CO ₃				<u> </u>	PO ₄ 3.32 E+05	SO ₄ 4.69 E+05		
Unit Number and Type 216-B-43 Crib through		NH ₄ CO ₃ 9.10 E+04	NH ₄ NO ₃	NO ₂	NO ₃	Oxalate	<u> </u>			
Unit Number and Type 216-B-43 Crib through 216-B-49 Crib			NH ₄ NO ₃	NO ₂	NO ₃ 5.69 E+06	Oxalate 	3.32 E+05	4.69 E+05		
Unit Number and Type 216-B-43 Crib through 216-B-49 Crib 216-B-50 Crib		9.10 E+04	NH ₄ NO ₃ 1.00 E+04	NO ₂	NO ₃ 5.69 E+06 1.50 E+03	Oxalate 	3.32 E+05	4.69 E+05		
Unit Number and Type 216-B-43 Crib through 216-B-49 Crib 216-B-50 Crib 216-B-57 Crib		9.10 E+04 1.20 E+04	NH ₄ NO ₃ 1.00 E+04	NO ₂	NO ₃ 5.69 E+06 1.50 E+03	Oxalate	3.32 E+05	4.69 E+05		

NOTES:

^aThe reported inventories are derived from the Waste Information Data System (WHC 1991) and Stenner et al. (1988). Not all sites have reported inventories, and the inventories do not necessarily list all of the contaminants disposed of at a site.

⁻⁻ indicates no inventory data available.

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6.0 216-BY CRIBS TREATMENT SYSTEM

This chapter describes a proposed system scale-up for the plume at the BY-Cribs site and includes cost estimates for system design, fabrication, operation, and well drilling. Because of similar treatment rate capabilities and system design, the N-Springs pump-and-treat system can be used as a model for system design and construction costs. A significant fraction of the cost is in locating significant quantities of highly contaminated groundwater. Operational costs are presented on a per-year basis and totaled for 5 years. Cost escalation is not included.

6.1 CONTAMINANT EXTRACTION SYSTEM CONCEPTUAL DESIGN

The interim remedial approach described here for the BY-Cribs Plume is intended to prevent further northward migration of the contaminant plume and eliminate the highest concentration core of the contaminant plume within a reasonable period of time (Figure 4-7). Based on the level of uncertainty in the contaminant conceptual model and the parameters established in Section 6.2 for system operations, a better understanding of the local aquifer properties and contaminant plume geometry is required to optimize the contaminant extraction system design and operation. A focused subsurface investigation program is necessary to refine the aquifer hydrology, geology, and contaminant trend data necessary to design an efficient and inexpensive extraction network.

The groundwater flow pattern for the BY-Cribs plume is restricted on the east due to the basalt subcrop located east of wells 699-50-53A and 699-52-54. Geologic logs of the local wells and regional geologic mapping indicate that the aquifer thickens to the west and north of the subcrop. Aquifer thickness in the area of the ⁹⁹Tc contaminant plume ranges from 0 m to less than 3 m (0 ft to < 10 ft) thick. Section 5.3.2.3 proposes the possibility of a late-Pleistocene flood-related erosional channel incised into the basalt underlying the aquifer. The erosional window defined by well 699-53-55 cluster and well 699-55-55 (Section 1.4.3) substantiates the possibility of such erosional features. If present, any erosional channel would result in a local thickening of the aquifer.

For greatest efficiency, the contaminant extraction well network should be located in the area where the greatest aquifer thickness coincides with the highest contaminant concentration. If an erosional channel is present, its location may be the optimum site for the extraction well network. The focused subsurface investigation would be composed of a seismic reflection survey to determine the profile of the bottom of the aquifer and locate any preferential flowpaths in the area of the contaminant plume (Section 5.3.2.3) and a limited groundwater sampling program using the sonic-push technique to access the aquifer (Section 4.4.1) and determine both aquifer thickness and contaminant concentrations. The limited geophysical investigation and sonic-push probe activities would produce a refined plume thickness and contaminant concentration model.

The subsurface investigation would be followed by the construction of at least five new groundwater extraction wells. In addition to providing sites to extract contaminated groundwater, the new wells would provide confirmation of the geohydrologic properties of the aquifer and also provide aquifer sediments for laboratory analyses related to contaminant geochemistry and bench-scale treatment tests. Aquifer pump tests would be conducted at each extraction well before the construction of the succeeding well to confirm that sufficient groundwater production is available from the well, to ensure that unanticipated changes in the aquifer are mitigated by the appropriate location of the next well, and to update the capture-zone model.

A preliminary capture-zone analysis for the By-Cribs plume was conducted utilizing known aquifer properties. Due to the recognized thin aquifer and flat gradient, it is expected that well spacing on the order of 250 to 300 m (820 to 984 ft) between wells would be sufficient to capture the highest concentration portion of the plume and prevent northward migration. The analysis indicates that five extraction wells located in the thicker aquifer section along the north-south axis of the plume and producing 38 L/min (10 gal/min) each would be necessary to effect plume containment and support contaminant mass reduction (Figure 6-1). According to a preliminary capture-zone analysis, groundwater pumping from each of the extraction wells shown in Figure 6-1 for a period of 5 years at a rate of 38 L/min (10 gal/min) would be sufficient to remediate the highest concentration core of the BY-Cribs plume.

The groundwater extraction wells would be constructed as 10-cm- (4-in.) diameter wells to reduce the installation cost. The wells would penetrate at least 6 m (20 ft) into the underlying basalt to enlarge their cones of depression and to capture any contaminants migrating within the fractured portions of the basalt.

The ion-exchange plant would be located near the center of the extraction well network north of well 699-52-54. One 15-cm- (6-in.) diameter injection well located to the east of the extraction well network and north of the basalt subcrop would be used for the injection of processed water. The appropriate location of the injection well would establish a groundwater gradient to assist in moving the plume towards the extraction wells. The approximate locations of the extraction well network, the injection well, and the configuration of the contamination capture zone are shown in Figure 6-1.

The estimated costs of the focused subsurface investigation, the groundwater extraction well network, and the injection well are presented in Table 6-1.

6.2 TREATMENT SYSTEM DESIGN PARAMETERS FOR THE 216-BY CRIB PLUME

This section discusses the design basis for an ion-exchange treatment system. Assumptions are as follows:

- 99Tc is the primary target contaminant of concern and is assumed to be entering the system at an average of 10,100 pCi/L. It is expected that 60Co and cyanide will also be removed by the resin but nitrate will not.
- Round-the-clock operations will extract and treat groundwater at rates of 189 L/min (50 gal/min) using the ion-exchange resin Dowex 21K. An operational life of 5 years is assumed of the system. An 80% service availability will be maintained with time out for scheduled maintenance, resin changeouts, etc.
- Fully automatic, failsafe, largely unmanned operation will be the norm. Daily/shift-wise examination of the system by operators and sampling for system performance by roving sampling crew.
- The system shall conform to all applicable environmental regulations. System will remove ⁹⁹Tc to below DWSs of 900 pCi/L.

- One large holding tank will be installed on either side of the treatment system to contain influent and effluent waters. Each tank will have a capacity of 49,210 L (13,000 gal).
- Operating design will be three resin vessels in series, plus a backup vessel, each column sized to provide 5-minute residence time. This will ensure no breakthrough of radionuclides while saturating the lead-column resin. No GAC treatment for organics is required.
- Equipment redundancies for mechanical operating components (pumps, filter systems, etc.) are required.
- No pre-treatment for removal of groundwater constituents is assumed.
- Laboratory analyses will be required for assessing system performance. No real-time, online techniques are available to quantify target radionuclide at historical concentrations. Regular sampling will be required until saturation of the lead column approaches. Continuous monitoring will not be required.
- Resin backwashing to remove fines will be performed using the offline vessel. Resin removal and addition will be accommodated through an adequately sized port.
- System winterization will be required. This is assumed to be implemented using gravity draining of the lines. The treatment system will need to be protected from the environment.
- Permanent power and utilities will be provided to process plant and extraction wells.
- Road construction will be required for access to the plant and to any wells used in plume treatment.

6.3 SYSTEM DESIGN

The N-Springs pump-and-treat system, used to remove ⁹⁰Sr at the N-Springs site, is used as a basis for conceptual design and cost estimate for the scaled-up BY-Cribs ⁹⁹Tc plume treatment system. The N-Springs treatment system functional and operational requirements have been presented in Frain and Jackson (1995), and a design description has been presented in DeSemple et al. (1995). The system design is included in this document by reference to the 100-N Area, N-Springs Pump and Treat Piping and Instrumentation Diagrams, 0100N-DD-J001, 0100N-DD-J002, and 0100N-DD-J003. Conceptually, the system design is similar to that shown in Appendix A. Water pumped from the well(s) is sent to a large holding tank near the treatment system. That water is pumped through a pump/filtration system to the treatment system. A treatment train of three resin columns in series is specified to contain .093 m³ (33 ft³) each of DOWEX 21K resin. A fourth column is held in reserve for resin changeout/loading. The system is plumbed such that any sequence of resin columns can be connected/valved to provide three-tank series operation. Treated water is sent to a second large holding tank for post-treatment storage. Treated water will be sent to the injection well(s) by a booster pump/filtration system. Sampling ports are provided. All equipment except the extraction and injection wells and large storage tanks are placed inside a weatherproof enclosure.

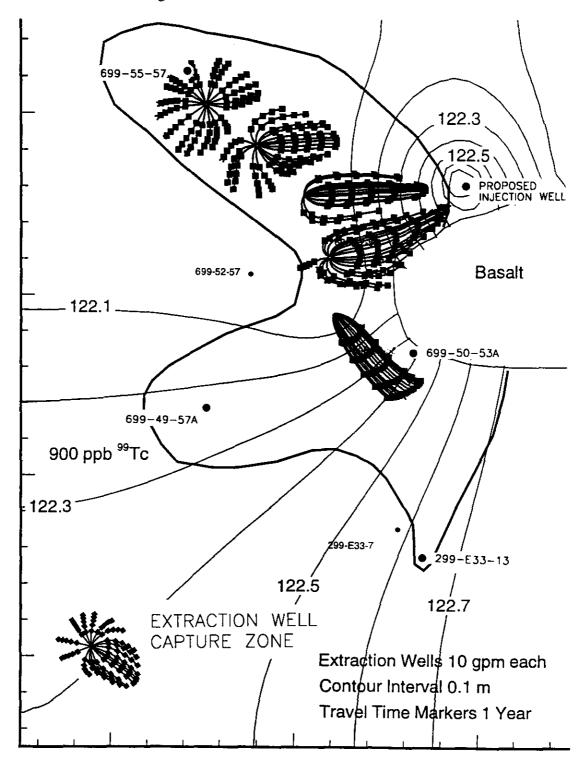
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6.4 SYSTEM COSTS

Cost data provided from the N-Springs pump-and-treat project are actual constructed costs. The ability to build a system from the existing plans eliminates some of the costs associated with designing the system from scratch. Labor requirements for operation assume roving operator support on a daily or shift-wise basis. Engineering support is assumed for day shift only. Costs for a BY-Cribs system are directly factored from N-Springs costs. Cost data are provided in Table 6-1.

6-4

Figure 6-1. Preliminary Capture-Zone Analysis.



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Table 6-1. 216-BY Cribs Treatment System Estimated Cost. (3 Sheets) (all costs are approximate)

	N-Spring Actual	BY-Cribs Site Estimate
System Design and Construction		
Project Support		
Direct Support	157,378	157,378
Indirect Support	297,940	297,940
Subtotal	455,318	455,318
Well Design ^a	270,781	250,000
Subsurface Investigations		
High-Resolution Seismic Reflection Survey (To include subcontractor labor/equipment/analysis/interpretation for surface study and necessary cross-hole seismic velocity testing/analysis/interpretation.)		500,000
Sonic Drilling Probe Holes		150,000
Subtotal		650,000
Well Installation		850,000
Indirect	201,295	
Installation	841,112	
Evaluation/Characterization	167,866	
Subtotal (To include costs of drilling, materials, aquifer testing and evaluation, geological logging/sampling/reporting, geophysical logging, etc.)	1,210,273	850,000
Design		
Skid Design	88,241	N/A ^b
Basic Operating Plant Design	458,133	150,000 ^h
Subtotal	546,374	150,000
Skid Procurement	281,739	281,739
Construction	992,736	992,736
	180,571	180,571
Subtotal	1,173,307	1,173,307
Construction Management	37,533	37,533
	87,705	87,705
Subtotal	125,238	125,238
Waste Management	7,537	7,537
Health and Safety	41,547	41,547

Table 6-1. 216-BY Cribs Treatment System Estimated Cost. (3 Sheets) (all costs are approximate)

(all costs are approximate)	N-Spring	BY-Cribs Site
	Actual	Estimate
Acceptance Setup/Testing	114,881	114,881
	211,029	211,029
Subtotal	325,910	325,910
Performance Monitoring		
Data Quality Objective/Performance Monthly Plan	42,408	42,408
Water Level and Groundwater Sampling	299,432	299,432
Groundwater Modeling	18,584	18,584
Performance Evaluation	5,367	5,367
Treatability Technical Evaluation	16,173	16,173
Subtotal	381,964	381,964
Site Maintenance	39,712	39,712
Total	4,859,700	4,732,272
System Operations		
Labor		
Craft (72 h/wk, includes operators, maintenance, health physics technicians)		206,000
Technical/Administrative (72 h/wk, includes engineering, geosciences, project management, samplers, clerical, etc.)		262,000
Field Supervisory (24 h/wk)		87,500
Subtotal (per year)		555,500
Subtotal (for 5 years)		2,777,500
Sampling		
Performance Monitoring (5 well samples and 1 post-treatment sample, 7 times/2 wk for 99Tc; 35 samples/2 wk, \$200/sample)		182,000
System Monitoring (6 post-treatment stage samples, 7 times/2 wk for ⁹⁹ Tc)		218,400
Offsite Laboratory Sampling (6 samples/mo, ⁹⁹ Tc, same sample points as system monitoring, \$200/sample)		14,400
Resin Characterization/Disposal (21 samples/yr analyze for ⁹⁹ Tc, ⁶⁰ Co, nitrate, metals)		105,000
Miscellaneous (⁶⁰ Co, cyanide, nitrate, interfering groundwater anions and cations, 12 times/yr, ⁹⁰ Sr quarterly)		100,000
Subtotal (per year)		619,800
Subtotal (for 5 years)		3,099,000

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Table 6-1. 216-BY Cribs Treatment System Estimated Cost. (3 Sheets) (all costs are approximate)

	N-Spring Actual	BY-Cribs Site Estimate
Materials		
Resin replacement (21 times/yr x 34 ft³, \$200/ft³)		143,000
Miscellaneous (maintenance replacement parts, materials, etc.)		45,000
Subtotal (per year)		188,000
Subtotal (for 5 years)		940,000
Waste Disposal		
Resin Removal and Replacement		
Craft Labor (decontaminating and decommissioning, health physics technical, etc., 4 craft at 16 h, 21 times/yr)		74,000
Field Supervision (16 h, 21 times/yr)		23,500
Materials (drums, burial boxes, miscellaneous materials, \$3,000/event, 21 events)		635,000
Disposal to the Environmental Restoration Disposal Facility		NO CHARGE
Subtotal (per year)		732,500
Subtotal (for 5 years)		3,662,500
Total Operations per year		2,095,800
Total Operations for 5 years		10,479,000
Total System Design/Installation		4,732,272
Total System Cost for 5 years		15,211,272

^aSelection of well locations based on subsurface investigation and groundwater modeling, sizing injection and extraction wells and extraction pumps; planning support facilities for wells including transfer line design, booster pump system design, well instrumentation, and power system lines and general site layout.

^bCosts not calculated or are reduced to reflect use of existing design.

N/A = not applicable.

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7.0 CONCLUSIONS AND RECOMMENDATIONS

The Risk-Based Decision Analysis for the 200-BP-5 Groundwater Operable Unit (BHI 1995c), summarized in Section 5.4, indicates that there is no significant future risk to human health associated with the ¹³⁷Cs, ^{239/240}Pu, and ⁹⁰Sr contamination at the B-5 Reverse Well plume. The treatment system for the B-5 Reverse Well performed satisfactorily for removal of all three contaminants. Aquifer pumping easily provided substantial quantities of groundwater containing significant concentrations of ¹³⁷Cs and ⁹⁰Sr. However, the concentrations of ^{239/240}Pu in the extracted groundwater were small relative to the quantities of ^{239/240}Pu discharged to the ground, confirming that the soil adsorbs plutonium to the extent that the groundwater is not excessively impacted. The future risks are low at this plume because both the ¹³⁷Cs and ⁹⁰Sr decay to negligible levels long before the plumes migrate off the Central Plateau and the ^{239/240}Pu is essentially immobile. For this reason, it is recommended that the treatability test/pilot-scale pump-and-treat study be discontinued at the B-5 Reverse Well plume.

The plume beneath the 216-A-25 Gable Mountain Pond has already migrated off the Central Plateau, and the maximum risk associated with the ⁹⁰Sr in this plume exceeds an ILCR of 1 x 10⁻⁵. Transport calculations show that this maximum risk will be reduced to the 10⁻⁵ residential ILCR level after the plume has traveled approximately 1,372 m (4,500 ft), which will take approximately 127 years.

The RBDA document also indicates that, at the BY-Cribs plume, the future risks associated with the ⁹⁹Tc in this plume are predicted to be above MTCA standards. The ion-exchange treatment system has been shown to work satisfactorily in removing elevated concentrations of ⁶⁰Co and ⁹⁹Tc at the BY-Cribs site. However, the demonstrated poor groundwater extraction rates at this site emphasize the large degree of uncertainty about plume geometry and aquifer characteristics. For this reason, it is recommended that the treatability test/pilot-scale pump-and-treat study be discontinued at the BY-Cribs plume.

A limited program of groundwater monitoring is necessary to assess the continued migration of plume contaminants at the 216-B-5 Reverse Well, the 699-505-3A site, and the 216-A-25 Gable Mountain Pond site. The monitoring program should be designed to determine the geometry and migration of the ⁶⁰Co, ⁹⁰Sr, ⁹⁹Tc, ¹³⁷Cs, ^{239/240}Pu, nitrate, and cyanide plumes. In addition, wells not associated with a plume but that have shown increased concentrations in target contaminants should be sampled regularly to determine if the plume is moving in an unexpected direction. An annual monitoring program at wells within and adjacent to the plume should be sampled for only the target contaminants. Other plumes that may be in the 200-BP-5 Operable Unit boundaries (e.g., ¹²⁹I, ³H) are either at low concentrations, are not treatable with current technology, or are sampled by other programs. Regardless of which Hanford organization performs the sampling, the analytical results must be evaluated to determine plume changes and to evaluate and approve future sampling.

7.1 TREATABILITY TESTING

The 200-BP-5 Operable Unit treatability test began with the initial definition of success as meeting or exceeding 90% removal of radionuclides in the influent groundwater. At a later date, the requirement that each radionuclide be treated to concentrations less than its MCL was applied to ensure no out-of-plume contamination of the aquifer. In most cases, this requirement was easily met; but for

⁹⁰Sr, the new definition for success required a 99.8% plus system removal efficiency to achieve the MCL, which was found to be more difficult to meet.

The treatability test results show that ⁶⁰Co and ⁹⁹Tc at the BY-Cribs plume and ¹³⁷Cs and ^{239/240}Pu at the B-5 Reverse Well could be removed to below MCL levels by ion exchange. For ⁶⁰Co and ^{239/240}Pu, the data show that the treatment systems either closely approached the 1.2 pCi/L MCL for ^{239/240}Pu or exceeded the 90% removal efficiency for ⁶⁰Co.

The initial pilot test run at the B-5 Reverse Well showed that the ⁹⁰Sr MCL could not be effectively met, although greater than 90% removal occurred over about 500 BV throughput. Increasing contact time in the second run significantly improved ⁹⁰Sr removal efficiency such that removal efficiency could be maintained above 90% for about 1,850 BV; the second pilot run did not achieve the ⁹⁰Sr MCL in the effluent.

The mini-column testing showed higher ⁹⁰Sr removal efficiencies relative to the pilot plant. Two mini-column tests run at approximately 5.5 minutes per column residence times indicated that ⁹⁰Sr system removal efficiencies of greater than 90% were achieved for 3,500 BV. For mini-column tests operated at 12 minutes per column empty-bed contact times, removal efficiencies greater than 90% were achieved for almost 2,900 BV. Additionally, ⁹⁰Sr was removed to below the 8-pCi/L MCL for the first 1,000 to 1,250 BV in the two 12-minute per column tests. Based on analysis of the kinetic rate constants calculated for these data, an extrapolation indicates that a three-column clinoptilolite system with about 18-minute contact time in each column should be sufficient to meet the 8-pCi/L MCL for several thousand bed volumes. However, high removal efficiencies require higher capital investment for larger adsorbent beds than do lower removal efficiencies.

Resin/adsorbent saturations of 90% to 100% for the radionuclides were not achieved for any of the materials in the pilot-scale systems. Full saturation test results were only achieved at the BY-Cribs mini-column tests for 99Tc, where 90% saturation was achieved at 5,200 BV.

Bench-scale mini-column tests were useful in shortening the time needed to get results and in the ability to test alternate resins. However, because the mini-column tests outperform the pilot-scale system in removal of radionuclides, some conservatism is needed to project the smaller system's results to that of a larger system. Tests capable of meeting the demonstrated need for long runs to reach saturation are practical only in a field-based bench-scale test. This type of test would be required to determine the ultimate adsorbent consumption rate. The treatability test data are adequate for developing a conservative design of a treatment system.

The test objectives included development of recommendations for a scale-up of the pilot test systems to full-scale IRMs. A design has been proposed for a treatment system at the BY-Cribs plume for the target contaminant ⁹⁹Tc. The design is based on the pilot-scale treatment resin, DOWEX 21K, in a system modeled after the 189-L/min (50-gal/min) N-Springs pump-and-treat system. However, it would first be necessary to characterize the ⁹⁹Tc contaminant plume prior to installing a treatment system.

7.2 GROUNDWATER REMEDIATION

While there are no drivers for remediation of any of the 200-BP-5 plumes on the accelerated IRM pathway, it is recommended that monitoring of the groundwater plumes at the BY-Cribs be continued and monitoring of both the B-5 Reverse Well and 216-A-25 Gable Mountain Pond plumes be added to the project scope. These monitoring activities should be conducted on at least an annual basis and should be increased in frequency if significant concentration changes or plume movement are observed. Existing well coverage may not be adequate for all sites or for potential changes in plume migration direction. In the future, additional wells should be considered if such changes are noted or groundwater modeling indicates gaps in the groundwater monitoring network.

The RBDA for the BY-Cribs plume shows that the future risks associated with the 99Tc in this plume exceed the risk level allowable by MTCA. The risk analysis predicts that at the year 2018, the point of maximum concentration (based on a historical 95% UCL concentration of 27,548 pCi/L of 99Tc) will have moved about 2,682 m (8,800 ft) downgradient and the maximum concentration of 9 Tc will be about 7,843 pCi/L (2.2 x 10⁻⁴ residential cancer risk). The risk analysis further predicts that the plume will reach the river in about 75 years and the residential risk will still exceed the allowable MTCA risk standard. Because there is no effective means of remediating this plume, it is recommended that this plume be removed from the accelerated IRM pathway and all further work be deferred for the time being. If an IRM or other remediation activities were to be initiated at some point in the future, a substantial aquifer characterization effort would be required to resolve the many technical uncertainties regarding the contaminant conceptual model and hydrogeology of this site. Such characterization work would include drilling new wells across the extent of the plume to determine its areal extent and aquifer properties. New wells should be located using results from a seismic refraction survey, which would be used to map the top of basalt, and from direct drilling to those locations where topographic lows in the basalt may indicate thicker aquifer conditions. Aquifer production properties need to be tested as well. Undisturbed sediment and basalt samples and depthspecific groundwater samples need to be obtained to permit geochemical and hydrological analyses.

The RBDA indicates that the B-5 Reverse Well plumes will not produce an unacceptable risk to offsite groundwater users. Therefore, it is recommended that the plumes be removed from the accelerated IRM pathway and that the future course of action include only groundwater monitoring to track plume movement and verify modeling results. Existing groundwater monitoring wells located south and southeast of the B-5 Reverse Well should be remediated and added to the monitoring network should any future site groundwater modeling activities indicate the potential for southerly groundwater flow.

The 216-A-25 Gable Mountain Pond does not require additional characterization at this time. The site has received large quantities of contaminants, but most appear to be well-bound to the vadose zone and will decay to negligible levels. Because of the expected low migration potential for the site contaminants, predicted at 1,372 m (4,500 ft) in 127 years for ⁹⁰Sr, by the risk analysis, only annual groundwater monitoring is required. Increases in groundwater ⁹⁰Sr concentrations observed over the last 5 years may require additional study if the trend continues.

7.3 LESSONS LEARNED

A number of problems were encountered during the course of developing the two pump-and-treat systems at the 200-BP-5 Operable Unit. Resolutions, consisting of either fixes to the problems or adjustments to the project, were ultimately found for the problems. A discussion of what was learned from these problems and responses is the subject of this section.

7.3.1 Groundwater

The most significant difficulty encountered in the 200-BP-5 Operable Unit was the lack of groundwater at the BY-Cribs site. Although the data were available during the course of treatability test plan preparation in 1993, in the form of semiannual and annual groundwater elevation reports (i.e., Kasza et al. 1990), the decline in groundwater levels was not recognized until early 1994 and was confirmed with the well suitability testing discussed in Section 4.1.

In an effort to better assess aquifer thickness and plume distribution at the BY-Cribs site, a drilling program for new wells was proposed and implemented several months before start of the treatability testing. Plans to drill holes blindly in the presumed vicinity of the plume were rejected as being too much risk for the cost. The use of a push technique (Section 4.4.1) using a Water Development Company sonic drill rig and standard cone penetrometer rods was implemented. Although only three boreholes out of eight were completed successfully into the aquifer and another three were stopped short of the aquifer, the data were collected rapidly and efficiently. Problems with the cone penetrometer rods were not overcome during the course of testing but helped to direct modifications after completion of the BY-Cribs drilling. The technique offers considerable benefits for relatively limited costs.

The use of high-resolution seismic reflection surveys to map the top of basalt was discussed during the course of field work and needs to be investigated further. The technique offers the ability to get away from blind drilling by characterizing broad areas of the subsurface with surface-based geophysical techniques. The cost of subsurface characterization on a per-square-foot-basis for several miles of survey lines would be equal to that of several boreholes but would significantly increase the potential for locating thicker parts of the aquifer and could proportionately decrease the chances of dry holes. Formation seismic velocity data have been gathered that would significantly improve the technique's accuracy.

7.3.2 Laboratory Testing

Laboratory testing was recognized as an important phase of the treatability test development program during preparation of the treatability test plan. The value of mini-column tests was recognized by their ease and efficiency of testing treatment alternatives. The systems were able to deliver saturation data for some of the contaminants. For others, alternate adsorbents, longer residence times, and pretreatment alternatives were examined. Some of this data could have been obtained during laboratory testing, but the volume of water required would have limited laboratory test effectiveness. Field mini-column testing provided an effective alternate to making constant changes in pilot-scale system operation.

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8.0 REFERENCES

- Ames, L.L., Jr., 1959, Zeolite Extraction of Cesium from Aqueous Solutions, HW-62607, Hanford Atomic Products Operation, General Electric Corporation, Richland, Washington.
- Ames, L.L. and R.J. Serne, 1991, Compilation of Data to Estimate Groundwater Migration Potential for Constituents in Active Liquid Discharges at the Hanford Site, PNL-7660, Pacific Northwest Laboratory, Richland, Washington.
- Anderson, J.D., 1990, A History of the 200 Area Tank Farms, WHC-MR-0132, Westinghouse Hanford Company, Richland, Washington.
- Baker, S.M., J.L. Devary, R.P. Elmore, E.F. Lorang, A.J. Rossi, and M.D. Freshly, 1988, U1/U2 Uranium Plume Characterization, Remedial Action Review and Recommendation for Future Action, WHC-EP-0133, Westinghouse Hanford Company, Richland, Washington.
- Barney, G.S., 1988, Adsorption of Plutonium on Hanford Sediments from Shallow Groundwaters Measurements of Distribution Coefficients, (internal memo 12714-PSL88-168 to J.L. Gieger, December 27, 1988), Westinghouse Hanford Company, Richland, Washington.
- Barney, G.S., K.J. Lueck, and J.W. Green, 1992, Removal of Plutonium from Low-Level Process Wastewaters by Adsorption, "Environmental Remediation, Removing Organics and Metal Ion Pollutants," ACS Symposium Series 509, American Chemical Society, Washington D.C., pp. 34-46.
- BHI, 1994, Environmental Investigation Procedures, BHI-EE-01, Bechtel Hanford, Inc., Richland, Washington.
- BHI, 1995a, 216-B-3 Pond Interim Stabilization, Final Report, BHI-00219, Rev. 00, Bechtel Hanford, Inc., Richland, Washington.
- BHI, 1995b, Risk-Based Decision Analysis for Groundwater Operable Units, BHI-00161, Rev. 01, Bechtel Hanford, Inc., Richland, Washington.
- BHI, 1995c, Risk-Based Decision Analysis for the 200-BP-5 Groundwater Operable Unit, BHI-00416, Rev. 00, Bechtel Hanford, Inc., Richland, Washington.
- BHI, 1995d, Workplan and Procedures for Bench-Scale Testing for the 200-BP-5, Unit #2 Groundwater Operable Unit, BHI-OP-00042, Rev. 00, Bechtel Hanford, Inc., Richland, Washington.
- BHI, 1995e, Workplan and Procedures for Bench-Scale Testing of Potential Absorbents for the 200-BP-5 Groundwater Operable Unit, BHI-OP-00015, Rev. 00, Bechtel Hanford, Inc., Richland, Washington.
- Bierschenk, W.H., 1957, The Effect of Ground-Water Mounds on the PUREX Operation, HW-49728, General Electric Company, Hanford Atomic Products Operations, Richland, Washington.

- Bierschenk, W.H., ed. 1959a, Chemical Effluents Technology Waste Disposal Investigations, July, August, September, 1958, HW-58023-RD, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Bierschenk, W.H., ed. 1959b, Chemical Effluents Technology Waste Disposal Investigations, October, November, December, 1957, HW-54848, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Brown, D.J., 1957, Chemical Effluents Technology Waste Disposal Investigations, October, November, December 1956, HW-53225, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Brown, D.J., 1963, Chemical Effluents Technology Waste Disposal Investigations, July December 1962, HW-76120-RD, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Brown, D.J., 1964, Chemical Effluents Technology Waste Disposal Investigations, January December 1964, HW-84549, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Brown, D.J. and J.R. Raymond, 1962, Hanford Experience in the Radiological Monitoring of Ground Water, HW-SA-2566, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Brown, R.E. and H.G. Ruppert, 1950, *The Underground Disposal of Liquid Wastes at the Hanford Works, Washington*, HW-17088, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Brown, M.J., R.L. P'Pool, and S.P. Thomas, 1990, Westinghouse Hanford Company Effluent Discharges and Solid Waste Management Report for Calendar Year 1989: 200/600 Area, WHC-EP-0141-2, Westinghouse Hanford Company, Richland, Washington.
- Connelly, M.P., J.V. Borghese, C.D. Delaney, B.H. Ford, J.W. Lindberg, and S.J. Trent, 1992, Hydrogeologic Model for the 200 East Groundwater Aggregate Area, WHC-SD-EN-TI-019, Westinghouse Hanford Company, Richland, Washington.
- Coony, R.M. and S.P. Thomas, 1989, Westinghouse Hanford Company Effluent Discharges and Solid Waste Management Report for Calendar Year 1988: 200/600 Area, WHC-EP-0141-2, Westinghouse Hanford Company, Richland, Washington.
- Cushing C.E. and D.G. Watson, 1974, Aquatic Studies of Gable Mountain Pond, BNWL-1884, Pacific Northwest Laboratory, Richland, Washington.
- Deju, R.A., 1974, *The Hanford Field Testing Program*, ARH-00004, Atlantic Richfield Hanford Company, Richland, Washington.
- Del Cul, G.D., W.D. Bostick, D.R. Trotter, and P.E. Osborne, 1992, "Technetium-99 Removal from Process Solutions and Contaminated Groundwater," In: Separation Science and Technology, 28(1-3), pp. 551-564.

9613494 0146 DOE/RL-95-59 Rev. 0

- Delaney, C.D., K.A. Lindsey, and S.P. Reidel, 1991, Geology and Hydrology of the Hanford Site: A Standardized Text for Use in Westinghouse Hanford Company Documents and Reports, WHC-SD-ER-TI-003, Rev. 0, Westinghouse Hanford Corporation, Richland, Washington.
- Delegard, C.H., R.L. Weiss, R.T. Kimura, A.G. Law, and R.C. Routson, 1986, Characterization and Anion Exchange Removal of Uranium from Hanford Groundwater, RHO-RE-SA-116P, Rockwell Hanford Operations, Richland, Washington.
- DeSemple, D.L., J.R. Hoyle, and E.A. Nelson, 1995, N Springs Pump-and-Treat System Design Description, BHI-00335, Bechtel Hanford, Inc., Richland, Washington.
- DOE, 1988, Consultation Draft Site Characterization Plan, DOE/RW-0164, Vols. 1-9, Office of Civilian Waste management, U.S. Department of Energy, Washington, D.C.
- DOE-RL, 1990, Remedial Investigation/Feasibility Study Work Plan for the 200-BP-1 Operable Unit, Hanford Site, Richland, Washington, DOE/RL-88-32, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1991, *Hanford Past-Practice Strategy*, DOE/RL-91-40, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1992, Hanford Site Waste Management Units Reports, DOE/RL-88-30, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1993a, 200 East Groundwater Aggregate Area Management Study Report, DOE/RL-92-19, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1993b, 200 West Groundwater Aggregate Area Management Study Report, DOE/RL-92-16, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1993c, B Plant Source Aggregate Area Management Study Report, DOE/RL-92-05, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1993d, *Phase I Remedial Investigation Report for the 200-BP-1 Operable Unit*, DOE/RL-92-70, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1993e, *PUREX Source Aggregate Area Management Study Report*, DOE/RL-92-04, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1994, State Waste Discharge Permit Application for Industrial Discharge to Land, 200 East Area, W-252 Streams, DOE/RL-93-61, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1995a, Treatability Test Report for the 200-UP-1 Operable Unit Hanford Site, DOE/RL-95-02, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

- DOE-RL, 1995b, Hanford Site Risk Assessment Methodology, DOE/RL-91-45, Rev. 3, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- DOE-RL, 1995c, *Pilot-Scale Treatability Test Plan for the 200-BP-5 Operable Unit*, DOE/RL-93-98, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- Earth Sciences Personnel, 1956, Earth Sciences' Waste Disposal Monitoring Activities Summary, April, 1956, HW-44354, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Ecology, EPA, and DOE, 1994, Hanford Federal Facility Agreement and Consent Order, 2 vols., as amended, Washington State Department of Ecology, U.S. Environmental Protection Agency, and U.S. Department of Energy, Olympia, Washington.
- Eliason, J.R., 1966, Earth Sciences Waste Disposal Investigations, July December, 1965, BNWL-CC-574, Pacific Northwest Laboratory, Richland, Washington.
- EPA, 1976, National Interim Primary Drinking Water Regulations, EPA-570/9-76-003, U.S. Environmental Protection Agency, Office of Water Supply, Washington, D.C.
- EPA, 1990, Assessment of Technologies for the Remediation of Radioactively Contaminated Superfund Sites, EPA/540/2-90/001, U.S. Environmental Protection Agency, Washington, D.C.
- EPA, 1992, Guide for Conducting Treatability Studies Under CERCLA, Final, EPA/540/R-92/071a, U.S. Environmental Protection Agency, Washington, D.C.
- Ford, B.H., 1993, Groundwater Field Characterization Report for the 200 Aggregate Area Management Study, WHC-SD-EN-TI-020, Westinghouse Hanford Company, Richland, Washington.
- Frain, J.M. and G.J. Jackson, 1995, N Springs Groundwater Pump-and-Treat System Functional and Operational Requirements, BHI-00030, Bechtel Hanford, Inc., Richland, Washington.
- Freeze, R.A. and J.A. Cherry, 1979, *Groundwater*, Prentice-Hall, Inc., Englewood Cliffs, New Jersey, p. 318.
- Freshley, M.D. and P.D. Thorne, 1992, Ground-Water Contribution to Dose from Past Hanford Operations, PNWD-1974 HEDR, Pacific Northwest Laboratory, Richland, Washington.
- Freundlich, 1926, Colloid and Capillary Chemistry, Methuen and Company, London.
- Graham, M.J. and D.A. Brown, 1981, Results for Separations Area Groundwater Monitoring Network for 1980, RHO-LD-0165, Rockwell Hanford Operations, Richland, Washington.
- Grove, D.B., 1971, Part II: An Analysis of the Flow Field of a Discharging-Recharging Pair of Wells, USGS-474-99, United States Geological Survey, Denver, Colorado.

- Haney, W.A., ed., 1959, Chemical Effluents Technology Waste Disposal Investigations, July, August, September, 1959, HW-62692-RD, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Haney, W.A., 1960, Chemical Effluents Technology Waste Disposal Investigations, October, November, December 1959, HW-64094-RD, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Haney, W.A., ed., 1963, Chemical Effluents Technology Waste Disposal Investigations, January-June, 1963, HW-78951, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Hayward, W.M., 1989, Gable Mountain Pond Interim Stabilization Report, WHC-SD-DD-TI-036, Westinghouse Hanford Company, Richland, Washington.
- Hodges, F.N., 1995, In Preparation, Hanford B-5 Reverse Well: Controls on Radionuclide Distribution, WHC-EP-0863, Westinghouse Hanford Company, Richland, Washington.
- Jelinek, R.T. and T.J. Sorg, 1988, "Operating a Small Full-Scale Ion Exchange System for Uranium Removal," In: *Journal American Water Works Association*, Volume 80, No. 7, pp. 79-83.
- Jungfleisch, F.M., 1984, Preliminary Estimation of the Waste Inventories in Hanford Tanks through 1980, SD-WM-TI-057, Rockwell Hanford Operations, Richland, Washington.
- Kasza, G.L., 1993, Potential for Groundwater Contamination from High Density Wastes Disposed at the BY Cribs, 200-BP-1 CERCLA Operable Unit, WHC-SD-EN-TA-003, Westinghouse Hanford Company, Richland, Washington.
- Kasza, G.L., S.F. Harris, and M.J. Hartman, 1990, Ground Water Maps of the Hanford Site, WHC-EP-0394-1, Westinghouse Hanford Company, Richland, Washington.
- Kipp, K.L. and R.D. Mudd, 1974, Selected Water Table Contour Maps and Well Hydrographs for the Hanford Reservation, 1944-1973, BNWL-B-360, Pacific Northwest Laboratory Water and Land Resources Department, Richland, Washington.
- Krauskopf, K.B., 1979, Introduction to Geochemistry, 2nd Ed., McGraw-Hill Book Company.
- Langmuir, 1918, "The Adsorption of Gases on Plane Surfaces of Glass, Mica, and Platinum,"
 J. American Chemical Society, Vol. 40, No. 9, p. 1,361.
- Last, G.V., B.N. Bjornstad, D.W. Wallace, D.R. Newcomer, J.A. Schranke, M.A. Chamness, C.S. Cline, S.P. Airhart, and J.W. Wilbur, 1989, *Hydrogeology of the 200 Areas Low-Level Burial Grounds-An Interim Report*, PNL-6820, Pacific Northwest Laboratory, Richland, Washington.
- Law, A.G. and R.M. Allen, 1984, Results of Separations Area Ground-Water Monitoring Network for 1983, RHO-RE-SR-84-24P, Rockwell Hanford Operations, Richland, Washington.

DOE/RL-95-59 Rev. 0

- Law, A.G. and A.L. Schatz, 1986, Results of Separations Area Ground-Water Monitoring Network for 1985, RHO-RE-SR-86-24P, Rockwell Hanford Operations, Richland, Washington.
- Law, A.G., A.L. Schatz, M.R. Fuchs, and K.L. Dillon, 1986, Results of Separations Area Ground-Water Monitoring for 1984, RHO-RE-SR-85-24P, Rockwell Hanford Operations, Richland, Washington.
- Ledgerwood, R.K., 1993, Summaries of Well construction Data and Field Observations for Existing 200-East Resource Protection Wells, WHC-SD-ER-TI-007, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Lindsey, K.A., B.N. Bjornstad, J.W. Lindberg, and K. M. Hoffmann, 1992, *Geologic Setting of the 200 East Area: An Update*, WHC-SD-EN-TI-012, Westinghouse Hanford Company, Richland, Washington.
- Maxfield, H.L., 1979, *Handbook 200 AREA Waste Sites*, RHO-CD-673, Rockwell Hanford Operations, Richland, Washington.
- Mercer, B.W., Jr., 1960, The Removal of Cesium and Strontium from Condensate Wastes with Clinoptilolite, HW-66276, Hanford Atomic Products Operations, General Electric Corporation, Richland, Washington.
- Myers, C.W., S.M. Price, J.A. Caggiano, M.P. Cochran, J.W. Czimer, N.J. Davidson, R.C. Edwards, K.R. Fecht, G.E. Holmes, M.G. Jones, J.R. Kunk, R.D. Landon, R.K. Ledgerwood, J.T. Lillie, P.E. Long, T.H. Mitchell, E.H. Price, S.P. Reidel, and A.M. Tallman, 1979, Geologic Studies of the Columbia Plateau: A Status Report, RHO-BWI-ST-4, Rockwell Hanford Operations, Richland, Washington.
- Newcomer, D.R., M.A. Chamness, D.L. McAlister, R.J. Brockman, A.W. Pearson, and S.T. Teel, 1992, *Unconfined Aquifer Hydrologic Test Data Package for the 200 East Area*, WHC-SD-EN-DP-030, Rev. 0, Westinghouse Hanford Company, Richland, Washington.
- Parker, H.M., 1956, Radiological Sciences Department Quarterly Progress Report, Research and Development Activities, January-March, 1956, HW-42403, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Parker, H.M., and C.C. Gamertsfelder, 1945, Weekly H. I. Reports on 200 Area and Environs for 1-5-45 Through 2-13-46 (#1 Through 58), HW-7-1115, E. I. duPont de Nemours and Company, Hanford Engineer Works, Richland, Washington.
- Price, K.R., 1986, Environmental Monitoring at Hanford for 1985, PNL-5817, Pacific Northwest Laboratory, Richland, Washington.
- Raymond, J.R., D.A. Myers, J.J. Fix, V.L. McGhan, and P.M. Schrotke, 1976, Environmental Monitoring Report on Radiological Status of the Ground Water Beneath the Hanford Site, January December, 1974, BNWL-1970, Pacific Northwest Laboratory, Richland, Washington.
- Reactor Handbook, 1961, Vol. II, Fuel Reprocessing, Interscience Publishers, New York, New York.

- Reidel, S.P., K.A. Lindsey, and K.R. Fecht, 1992, Field Trip Guide to the Hanford Site, WHC-MR-0391, Westinghouse Hanford Company, Richland, Washington.
- Rhodes, D.W., K.R. Holtzinger, and J.R. McHenry, 1954, Adsorption of Radioactive Isotopes by Soil from a Bismuth Phosphate Waste, HW-32978, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.
- Robinson, S.M., 1990, Strategy for Management of Oak Ridge National Laboratory (ORNL) Process Wastewater, Draft, Martin Marietta Energy Systems.
- Serkowski, J.A. and W.A. Jordan, 1989, *Operational Groundwater Monitoring at the Hanford Site --* 1988, WHC-EP-0260, Westinghouse Hanford Company, Richland, Washington.
- Serkowski, J.A., W.A. Jordan, and M.J. Hartman, *Groundwater Maps of the Hanford Site*, *June 1994*, WHC-EP-0394-9, Westinghouse Hanford Company, Richland, Washington.
- Serne, R.J. and M.I. Wood, 1990, Hanford Waste-Form Release and Sediment Interaction Status Report with Rationale and Recommendations for Additional Studies, PNL-7297, Pacific Northwest Laboratory, Richland, Washington.
- Smith, R.M., 1980, 216-B-5 Reverse Well Characterization Study, RHO-ST-37, Rockwell Hanford Operations, Richland, Washington.
- Smith, D.L., 1993, Safety Evaluation for the Interim Stabilization of the 216-B-3 Pond, the 216-B-3-3 Ditch, and the 216-B-3A Lobe, WHC-SD-EN-TI-082, Westinghouse Hanford Company, Richland, Washington.
- Sorg, T.J., 1989, Removal of Uranium from Drinking Water by Conventional Treatment Methods, EPA/600/D-89/020, Drinking Water Research Division, Risk Reduction Engineering Laboratory, U.S. Environmental Protection Agency, Cincinnati, Ohio.
- Sorg, T.J., 1991, Radionuclide Removal, EPA/600/D-91/262, Drinking Water Research Division, Risk Reduction Engineering Laboratory, U.S. Environmental Protection Agency, Cincinnati, Ohio.
- Stedwell, M.J., 1954, Evaluation of Nickel Ferrocyanide Scavenging of Uranium Recovery Plant Solvent Extraction Waste, HW-31428, General Electric Company, Richland, Washington.
- Stenner, R. D., K. H. Cramer, K. A. Higley, S. J. Jette, D. A. Lamar, T. J. McLaughlin, D. R. Sherwood, and N. C. VanHouten, 1988, *Inactive Waste Sites at Hanford*, PNL-6456, Pacific Northwest Laboratory, Richland, Washington.
- Strait, S.R. and B.A. Moore, 1982, Geohydrology of the Rattlesnake Ridge Interbed in the Gable Mountain Pond Area, RHO-ST-38, Rockwell Hanford Operations, Energy Systems Group, Richland, Washington.
- Thomas, C.W., D.L. Reid, and E.A. Treibs, 1956, Cobalt-60 in Ground Water and Separations Plant Waste Streams, HW-42612, General Electric Company, Hanford Atomic Products Operation, Richland, Washington.

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DOE/RL-95-59 Rev. 0

- Walton, W.C., 1989, Analytical Groundwater Modeling, Flow and Contaminant Migration, Lewis Publishers, Inc., Chelsea, Michigan.
- WHC, 1988, Environmental Investigations and Site Characterization Manual, WHC-CM-7-7, Westinghouse Hanford Company, Richland, Washington.
- WHC, 1991, Waste Information Data System, (WIDS), Westinghouse Hanford Company, Richland, Washington.
- WHC, 1994, Treatment Tests for Ex Situ Removal of Chromate, Nitrate, and Uranium (VI) from Hanford (100-HR-3) Groundwater Final Report, WHC-SD-ER-DTR-001, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

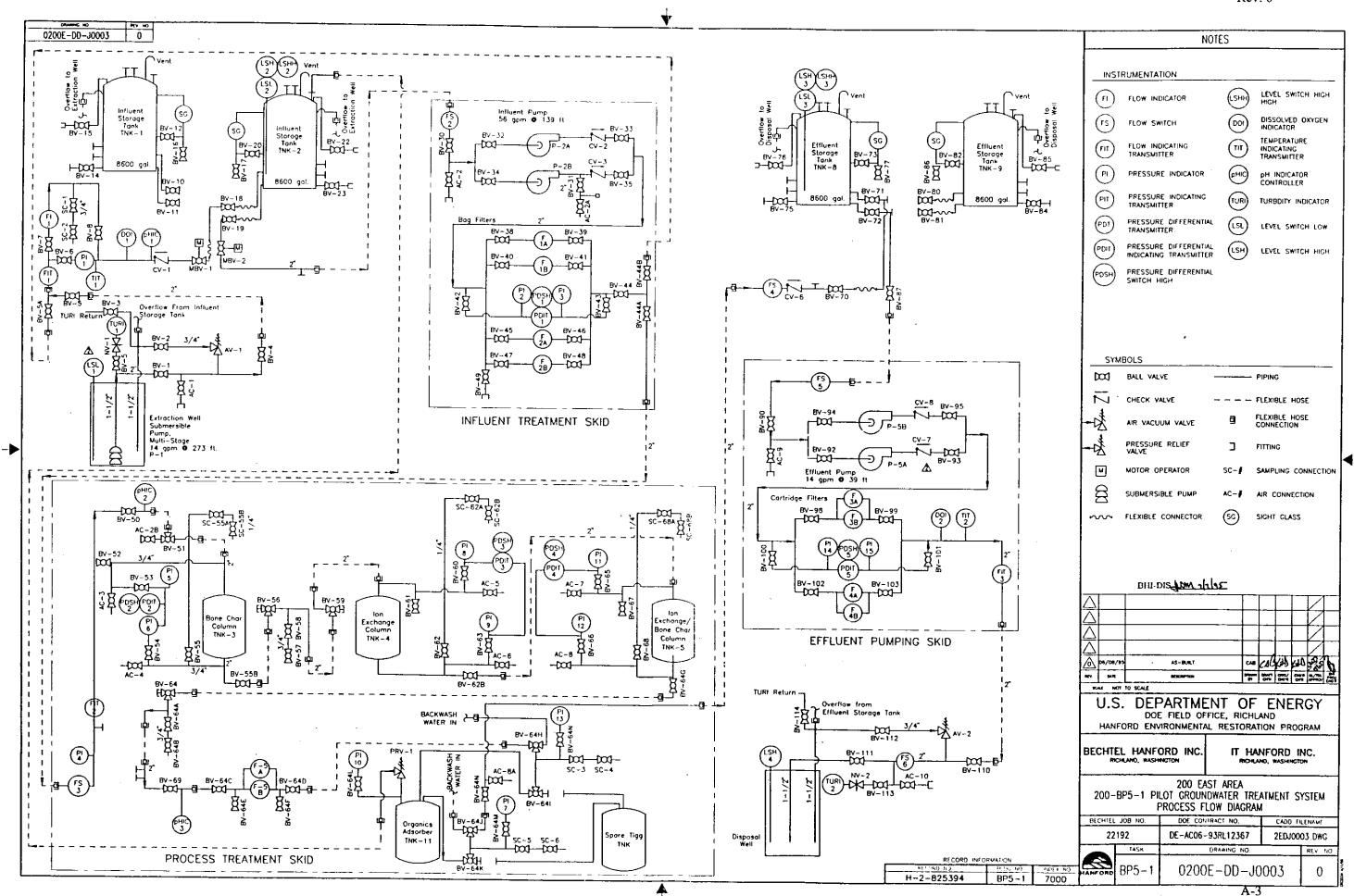
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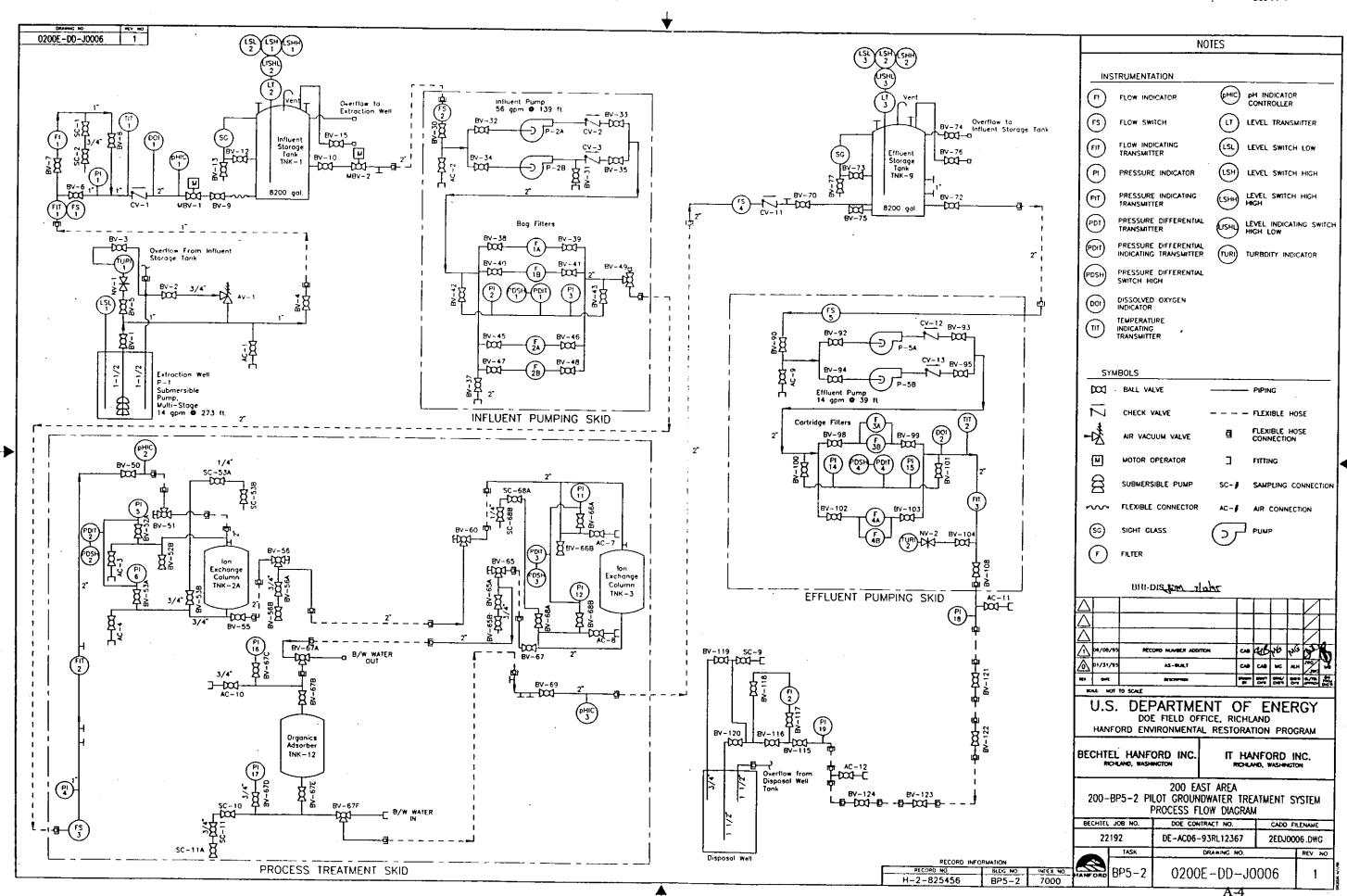
APPENDIX A

PROCESS FLOW DIAGRAMS

RL95-59,W51/A2 A-1

RL95-59.W51/A2 A-2





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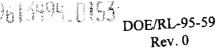
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APPENDIX B

DATA SHEETS

RL95-59.W51/A2 B-1

RL95-59.W51/A2 B-2





TECHNICAL DATA SHEET

CATION EXCHANGE CAPACITY......1.85 meg/gm. (This number may vary as C.E.C. values are relative to procedure)

Clinophlolite (based on X-ray diffraction in weight percent 90+/-5

Chemical Analysis (In weight percent)		Major Exchangeable Cations Rb+ Li+ K+ Cs+ NH4+ Na+
so ₂	69.1	Ag ⁺ Cd ⁺² Pb ⁺² Zn ⁺² Ba ⁺²
Al ₂ O	11.9	st ⁺² Cu ⁺² Hg ⁺² Mg ⁺² Fe ⁺³
C	J	Co ⁺³ A ⁺³ C ⁺³
MgO	A	Selectivity for the above cations is a function of
No ₂ O	3.5	hydrated molecular size and relative concentration.
K ₂ O	38	
Fe ₂ O ₃	J	Primary Adsorbing Gases
MnO	4.9	CO CO2 SO2 H2S NH3 HCHO Ar O2
		N2 H2O He H2 Kr Xe CH2CH
		Freon Formaldehyde Merc:aptans

Molecular Ratio Si/Al	4.9
Bulk Density (solid dried)	87lbs/ft ³ (1390kgs/m ³⁾ 51-60 lbs/ft. ³ (817-962 kgs./mi ³)
Bulk Density(aggregate dried)	51-60 lbs/ft.3 (817-962 kgs./mt ³)
. Hardness-Mohs Scale	5.1
Pore size (diameter)	4.0A
Pare Valume	15%
Specific Surface Area	1357 yd ² /oz (40m i²/g m.)
Alkali Stability (pH)	7-13
Acid Stability (pH)	0-7
Thermal Stability	1202f (650C)
Crushing Strength	1202f (650C) 2500lbs./in ² (176kgs./cm ²)

Information herein is accurate to the best of our knowledge. Suggestions are made without warranty or guarantee of results. Before using, user should determine the suitability of the product for his intended use and user assumes the risk and liability in connection therewith.

615 DEER ROAD . CHERRY HILL, NJ 08034 . (609) 354-1152

TELEX: 6503023149 • FAX: (609) 354-6165

RESINTECH SIR-600 has a high ion exchange capacity that it is highly specific for Cesium, Ammonia and Strontium, when in the presence of large amounts of divalent ions such as Calcium and Magnesium. It has a unique crystalline structure that physically restricts its reactivity towards certain ions thus increasing its selectivity towards others.

FEATURES & BENEFITS

HIGHLY SELECTIVE - Order of Selectivity:
 Cs>>K>NH₄>Na>Sr>Ca>Mg

ResinTech SIR-600 is a naturally occurring zeolite that is specially purified and backwashed free of humus, fines and other foreign matter.

ResinTech SIR-600 is supplied as moist granular particles with a size range essentially the same as for standard ion exchange materials used in water and radwaste treatment.

→ MAXIMUM OXIDATION & RADIATION STABILITY

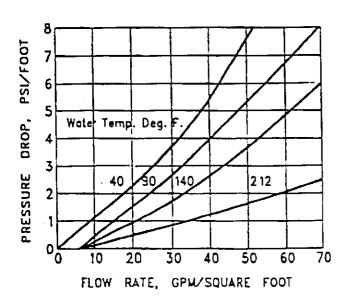
ResinTech SIR-600 will give greatly increased life where resin degradation due to oxidative effects such as from ionizing radiation or secondary thermal effects from radioactive decay are anticipated such as in high temperature or highly radioactive environments and waste treatment recovery operations.

* SUPERIOR PHYSICAL STABILITY

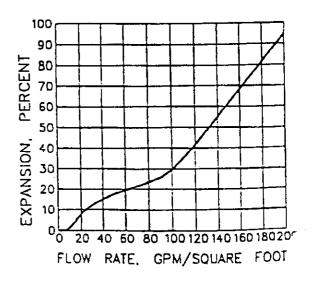
its low shrink and swell characteristics, as it changes ionic form, insures superb resistance to osmotic shock.

HYDRAULIC PROPERTIES

PRESSURE DROP.-The pressure drop of ResinTech SIR-600 treating a water at various temperatures and flow rates is shown below.



BACKWASH—The resin bed should be backwashed at a rate that expands the bed 25 to 75 percent to classify the bed and remove fines generated during the loading process. The expansion charecteristics at 60 Deg. F are shown below.



B-4









A-50 ION EXCHANGER SERVES

Description

UOP A-50 lon Exchanger Scries are an alkali metal alumino-silicate of the Type A crystal structure supplied in the Na+ (sodium) form.

Chamical Formula

 $Na_2O_3 \cdot Al_2O_3 \cdot 2 SiO_2 \cdot 4.5 H_2O_3$

Shipping Information

A-50 Ion Exchanger Series are available in powder and 20×50 mesh form. Samples may be obtained on request.

Typical Application Areas

A-50 Ion Exchanger Series products are generally highly effective cation exchangers for the removal of small ions from dilute solutions.

Typical Properties

	4 5 (A-51
Particle Size	Powder (Fite IG µm)	Mesh (20×50)
Pore Openings	3 2A into a-cago	4 2 Å into a-cage
, .	2A inte ficare	2 2A into B-cage
Bulk Density		
As shipped	26 lbs/cu.fr	42 lbs/cu.ft.
	420 kg/cu.m.	670 kg/cu.m.
Hydrated, packed	J	50 lbs/cu.ft.
BS&D Density		800 kg/cu.m.
(Hydrated)		42 lbs/cu.ft.
(II)UIZUXI)		670 kg/cu.m.
H ₂ O content	18-22wt,%	070 kg/cu.m.
As shipped		1-10 wt. %
Hydrated		16-20 wt. %
Ion Exchange		
Capacity	6.5 – 7.0 meg/gm	5.0 - 5.5 meq/gm
• •	(anhydrous)	(anhydrous)

Regeneration

Typically, regeneration may be accomplished by the use of NaCl solution as the regenerant fluid.

Additional Information

Bulletins discussing additional applications of UOP Molecular Sieves are available. Call or write your nearest UOP Molecular Sieve Sales Office.

Sales Offices

UOP Molecular Sleve Adsorbents 25 E. Algonquin Hoad Des Plaines, 1L 60017-5017 (708) 391-2000

EASTERN RECION Molecular Slave Adsorbents 307 Fellowship Drive Suite 207 Mt. Laurel, NJ 08054

MIDWESTERN REGION Molecular Slave Adsorborts 10 South Riverside Plaza, Suite 2200 Chicago, 11, 60606.3882 (312) 454-2230

Molecular Sieve Adsorbents 25111 Country Club Blvd. Suite 265 N. Olmsted, OH 44070 (216) 734-0086 SOUTHWESTERN REGION Molecular Sieve Adjordments 13105 Northwest Freeway Suite 800 Houston, TX 77040 (713) 744-2840

PACIFIC REGION Molecular Sieve Adsorbents 12389 Lewis Street, Suite 201 Garden Grove, CA 92640 (714) 750-1551

CANADA UOP Canada, Inc. Suite 1860, 444-5th Ava., S.W. Calgary, Alberta T2P 2T8, Canada (403) 294-1757 UOP Canada, Inc. 120 Eglinton Ave. East Suite 304 Toronto, Ontario, M4P 1E2, Canada (416) 488-4752

EUROPE UOP M.S. S.p.A. Via Durini 27 20122 Milano. Italy 39-2-76004773

UOP Molecular Sieve Admirhents P.O. Box 9848, Fi. 25 Dubai International Trade Centre Dubai, United Arab Emirates (971-4) 313-841 FAR EAST UOP K.K. Singapore Representative Office Molecular Sleve Adsorbents 101 Thomson Road 11-05 United Square Singapore 1130 (65) 253-1652

UNION SHOWA K.K. Molecular Sleve Adsorbonts Sanwa Building 27-17 Hamamatsu-cho, 1 Chome Minato-ku, Tokyo 105, Japan (81) 3-3432-7553







1E-95 UN EXCHANGE

Description

UOP IE-95 Ion Exchanger is an alkali me a alumino silicate primarily of the chabazite structure type supplied in the mixed ionic (Na+, Mg++, Ca++) form.

Chamical Formula

(Na,O, MgO, CaO) - Al₂O₃ - 4-6 SiO₂ • H₂O

Shipping Information

IE-95 Ion Exchanger is available in 20×50 mesh form. Samples may be obtained on request.

Regeneration

Typically, regeneration may be accomplished by the use of NaCl solution of the regenerant fluid.

Typical Proparate

Particul Simi Mesh (20×50)

Pore Opining: 3.7×4.2 Å and 3.

HS&D Density (hydrated)

H₂O Content
As shipped
Hydrated
Ion Exchange Capacity

1-10 wt. % 12-17 wt. % 2.0-2.5 meq/g (anhydrous)

38 lbs/cu.m.

610 kg/cu.m.

Typical Application Areas

UOP IE-95 Ion Exchanger products have been found to be effective for recovery and storage o radio-nuclides and for cation exchange from ac solutions.

Additional Information

Bulletins discussing additional applications of UOP Molecular Sieves are available. Call or write your near UOP Molecular Sieve Sales Office.

Sales Offices

Wolecular Sieve Adsorbents 25 E. Algonquin Road Des Plaines, 1L 60017-5017 (708) 301-9000

EASTERN RECION
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ROHM AND HAAS COMPANY

PHILADELPHIA, PENNSYLVANIA 19105

FLUID PROCESS CHEMICALS



WINNI I I I I I I I I I

AMBERLITE IRG

Amberlite IRC-718 is a crosslinked macroreticular cation exchange selectivity for the heavy metal cations over the more common adhali or a sodium, potassium, calcium and magnesium. This selections is active to be chelating functionality chemically bound to the resin matrix. This is each several cation active sites which coordinate with the heavy metal ion and make bound than the more common ions.

IMPORTANT FEATURES OF AMBERCITE IRC 2718

HIGHLY SELECTIVE FOR HEAVY METAL IONS—Amberlite IRC-718 resin, by means of its unique chelating functionality, is selective for heavy metal cations in the presence of alkali or alkaline earth cations.

STABLE OVER THE ENTIRE PH RANGE—Amberlite IRC-718 has proven itself in both the field and the laboratory to be stable over the entire pH range, and functionally active over a wide pH range.

EXCELLENT KINETICS—Because of its unique structure, Amberlite IRC-718 exhibits excellent kinetic behavior,

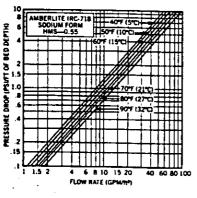
resulting in low leakage of metal and a scing cycle. The elution of Amberlite IRC of some non-accomplished typically with a strong acid at pH to less Elution is very efficient and a high concentration of ions is obtained in the eluate. (After acid elution the resin must be returned to the appropriate form; e.g., to the sodium form using sodium hydroxide).

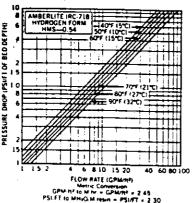
HIGHLY RESISTANT TO PHYSICAL BREAKDOWN—The cross-linked macroreticular structure of Amberlite IRC-718 prevents the excessive swelling and contracting that fragments gelular chelating resins.

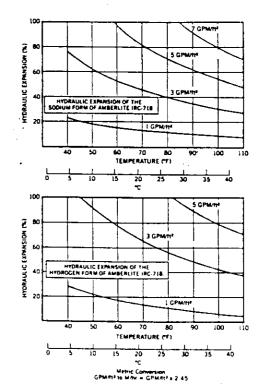
HYDRAULIC CHARACTERISTICS:

PRESSURE DROP—The approximate pressure drop for each foot of bed depth in normal downflow operation at various water temperatures and rates of flow are shown in the graphs below.

BACKWASH CHARACTERISTICS—After each operational cycle, Amberlite IRC-718 should be backwashed for approximately ten minutes to purge the bed of any insoluble material that may have collected on top of the resin. The bed should be expanded a minimum of 50% during backwash.







SUGGESTED: OPERATING: CONDITIONS

pH Range 2.0 to 14.0

Minimum Bed Depth 24 inches (0.61 m)

Maximum Temperature 160°F (71°C) in all forms (In some forms the resin is

stable at higher temperatures. Contact the manufacturer for specific in-

formation)

Regenerant Concentration*

4 to 15% HCl or H₂SO₄

Regeneration Flow Rate

0.25 to 0.50 gpm/ft³ (2.0 to

4.0 l/hr/l)

Neutralization

1 to 15% sodium

hydroxide

Service Flow Rate

1.0 to 5.0 gpm/ft³ (8.0 to

40.1 l/hr/l)

Cation Exchange Capacity

1.0 meg/ml. wet resin

SELECTIVITIESS'

The relative selectivity of Amberlite IRC-718 in the sodium form for various cations in aqueous solution at pH 4 is shown below. Solutions containing 0.075 M metal ions were passed through Amberlite IRC-718 at a flow rate of 1 gpm/ft³. The quantity of metal in the influent to the column was twice that of the capacity equivalent in the Amberlite IRC-718. All values are relative to calcium.

Metal Ion	K M
Hg ⁺⁺	2800
Hg ⁺⁺ Cu ⁺⁺	2300
Pb++	1200
Ni ⁺⁺	57
$\mathbf{Z}\mathbf{n}^{++}$	17
Cd++	15
Co++	6.7
Fe ⁺⁺	4.0
Mn++	1.2
Ca ⁺⁺	1.0

The selectivity of Amberlite IRC-718 was also investigated in an ammoniacal stream (pH = 9) containing 200 g/l (NH₄)₂SO₄. A flow rate of 1 gpm/ft³ and six times the resin equivalent of each metal ion was used. The results are given below.

Metal Ion	К ^М
Co++ Ni++	83
Ni ⁺⁺	30
Cd++	14
Cu ⁺⁺	10
Zn++	3
Ca++	$\overline{1}$

Rev. 0

TCAE CHARACTERISTICS

PHYSICAL FORM—Attrition resistant, tan spherical particles supplied in the fully hydrated sodium form.

Shipping Weight

42 lbs/ft3 (Na+ form) (672g/l)

Screen Grading (wet)

16 to 50 mesh (U.S. Standard Screen)

Moisture Content

69%* (Na + form)

Swelling

40% H⁺ → Na⁺

* Approximate

APPLICATIONS

RECOVERY OF HEAVY METALS FROM HYDROMETALLURGICAL LEACH STREAMS—Because of the ability of Amberlite IRC-718 to function over a wide pH range, this resin can effectively recover heavy metal cations that have been leached from ores by either acids or bases. Elution of this resin is extremely efficient, yielding a concentrated solution of the desired metal ion.

PURIFICATION OF PROCESS STREAMS CONTAINING TRACE TRANS-ITION METALS—Amberlite IRC-718 can often effectively remove trace heavy metal cations that are present in process streams as catalysts or process aides, without altering the composition of the stream. This resin can also remove the products of corrosion encountered in process streams. The high selectivity of Amberlite IRC-718 for transition metal cations, as well as its activity over wide pH range make it a good choice for a variety c specialized processing applications.

ELECTROPLATING INDUSTRY—The high selectivity of Amberlite IRC-718 for heavy metals will also prove useful in the removal and recovery of metal ions from plating baths, dragout, and rinse waters in the plating industry. The macroreticular structure of this resin insures its stability in the aggressive solutions of a plating bath.

chlor-alkali industry—Amberlite IRC-718 effectively removes hardness, to less than 0.1 ppm, from brine fed to chlor-alkali electrolysis cells.

SAFE HANDLING INFORMATION—A Material Safety Data Sheet is available for Amberlite IRC-718. To obtain a copy contact your Rohm and Haas representative.

CAUTION: Acidic and basic regenerant solutions are corrosive and should be handled in a manner that will prevent eye and skin contact.

Nitric acid and other strong oxidizing agents can cause explosive-type reactions when mixed with ion exchange resins. Proper design of process equipment to prevent rapid buildup of pressure is necessary if use of an oxidizing agent such as nitric acid is contemplated. Before using strong oxidizing agents in contact with ion exchange resins, consult sources knowledgeable in the handling of these materials.

Amberiate is a trademark of Rohm and Haas Company, or of its subsidiaries or affiliates Company's policy is to register its trademarks where products designated thereby are marby the Company, its subsidiaries or affiliates.

These suggestions and data are based on information we believe to be reliable. They are olfered in good faith, but without guarantee, as conditions and methods of use of our products are beyond our control. We recommend that the prospective user determine the suitability of our materials and suggestions before adopting them on a commercial scale.

Suggestions for uses of our products or the inclusion of descriptive material from patents and the citation of specific patents in this publication should not be understood as recommending the use of our products in violation of any patent or as permission or license to use any patents of the Rohm and Haas Company.

^{*} See Safe Handling Information Section

TECHNICAL BULLETIN

DURASIL®

230

Ion Exchange Media

Description

DURASIL® 230 media is an inorganic oxide based cation exchanger with a high specificity for radioactive cesium (Cs) and a low specificity for such nonradioactive species as sodium (Na) and lithium (Li).

Application

DURASIL® 230 media was designed to selectively remove Cs radioisotopes from radwaste streams, and has been used at commercial nuclear power plants and DOE sites. It is sluiceable, bridges retention screens, and can be used in systems designed for organic resins.

Characteristics

DURASIL® 230 media has the following characteristics:

- High capacity for Cs
- Excellent mechanical stability
- · Invulnerable to attack by oxidants such as chromates and nitrates
- Non-flammable
- Does not shrink or swell
- Resists oil fouling
- · Virtually immune to radiation damage and biological
- · No effect on pH of waste stream
- · Low pressure drop

Properties

Base Material Inorganic Oxide Bulk Density (dry) 40 lbs/cu.ft. (0.65 g/ml) Maximum Swelling (%) Particle Size 20-50 mesh (840µ-290µ) Ion Exchange Mechanism Sodium Exchange Functional Group Acidic Ionic Form Shipped Sodium Exchange Capacity 2.0 meg/mi (3.1 meg/g) 3 - 10 pH Range Operating Temperature 32-140°F (0-60° C) Storage Temperature 32-140°F (0-60° C) Shipped Wet

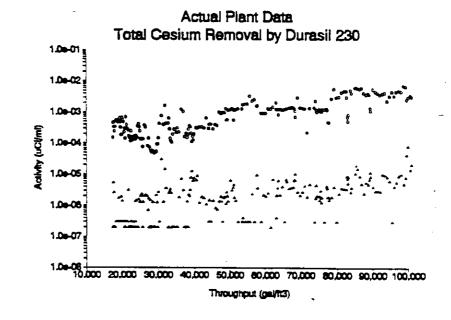
Operating Performance

The selectivity of **DURASIL®** 230 media results in high media capacity and large DFs for Cs. The graph below shows actual operating performance of **DURASIL®** 230 media in the treatment of a radioactive wastewater stream. The total volume radwaste processed was 2,120,100 gallons with an average conductivity of 163 µmho/cm. Other chemistry data is presented below.

Average Cs Influent Activity
O 1.9E-03 µCi/ml

Average Cs Effluent Activity 5.7E-06 µCi/ml

Average
Decontamination Factor = 247

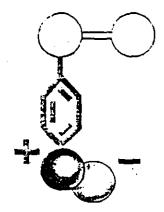


GTS DURATEK, INC. 8955 Guilford Road Suite 200 Columbia, MD 21046 (410) 312-5100 (301) 621-8211 (FAX)

GTS Duratek, Inc. reserves the right to alter characteristics of the Durasil[®] and Dura C[™] media. This technical bulletin is subject to revision without notice.

DURASIL[®] is a registered trademark of GTS Duratek, Inc. DURA C[™] is a trademark of GTS Duratek, Inc.





Properties of DOWEX Anion Exchange Resins — Tailored for Uranium Recovery

Strongly basic anion exchange resins, particularly gel Type 1, are extremely versatile products. For this reason, they are widely used in various fixed bed and moving bed equipment configurations for uranium recovery. The DOWEX 21K anion exchange resins have optimum levels of crosslinkage in the polymer matrix. This results in high porosity to afford efficient adsorption and elution of uranyl complexes.

TABLE

DOWEX 21 K Resin Properties (Typical)

Physical Form
Sphericity
Total Exchange Capacity
(wet) Chloride Form
Water Retention Capacity

Chloride Form
Color Throw APHA no.

Classified Density

Hard Golden Brown Colored Spheres 90% Whole Bead (minimum) 1.2 meg/ml (minimum)

50-58%

20 (maximum) 43 lb/cu ft Performance is positively affected by other chemical properties of the family of DOWEX 21 K resins, as well. These include:

- High available total exchange capacity that provides useful operating capacity.
- Appropriate moisture content, a property which impacts the kinetics of uranyl complex exchange in fixed bed and fluidized bed designs.
- Resistance to osmotic shock due to moderate changes in ionic form from regenerated to exhausted state.
- Resistance to organic fouling which affords more cycles before necessary resin replacement.

DOWEX 21 K resins have superior physical strength with low attrition rates. In processes where movement of ion exchange resins is necessary, this high bead strength minimizes increases in pressure drop due to bead fragmentation, and also lessens attrition losses. Typical properties are shown in Table 1.

Screen Size — Important To Productivity

In addition to the necessary chemical properties above, particle size (screen size distribution) is important to maximize productivity. Particle size optimization is required because of the differences in uranium mine streams, leach liquors, and the varying designs of the contacting equipment used.

DOWEX 21K, 16-20 mesh, resin is manufactured for use in resin-in-process (RIP) circuits, high flow rate fluidized bed processes, and high flow rate fixed bed columns. The large beads have higher terminal settling velocity, higher surface area per bead, and good skeletal density. Thus, they offer more resistance to fluidization in upflow systems. Concerns about pressure drop

per square foot of bed depth are minimized in high flow systems using DOWEX 21 K, 16-20, mesh resin.

DOWEX 21 K. 20-40 mesh, resin facilitates a faster mass transfer of uranium complexes in conventional fixed bed downflow systems. The smaller beads are durable, with the good porosity and enhanced surface area necessary for exchange to take place.

A few systems now in operation require a compromise between the 16-20 mesh and the 20-40 mesh products. DOWEX 21 K. 16-30 mesh, resin is an intermediate product that allows for acceptable loading and elution in these designs.

Figure 1 shows a comparison of pressure drop versus flow rate for the three DOWEX 21 K resin products. Some operations require upflow in a fluidized bed and, in most cases, a backwash is employed to classify the resin bed or to remove particulates from the resin. The bed expansion characteristics of the resins are important for each of these cases. Figure 2 compares the various sized DOWEX 21 K resin products with respect to fluidization versus flow rate. Screen size specifications for DOWEX 21 K resins are listed in Table 2.

TABLE 2

Screen Size Specifications of DOWEX 21 K Resins

16-20 Mesh Resin

Through 20 Mesh (840 microns)	10%	maximum
Through 25 Mesh (710 microns)	2%	maximum
16-30 Mesh Resin		
Through 16 Mesh on 30 Mesh (1,190 microns to 590 microns)	95%	minimum
Through 30 Mesh (590 microns)	3%	maximum
20-40 Mesh Resin		
On 16 Mesh (1,190 microns)	2%	maximum

20% maximum

5% maximum

On 20 Mesh (840 microns)

Through 40 Mesh (420 microns)

Photomicrographs of DOWEX 21K Resins

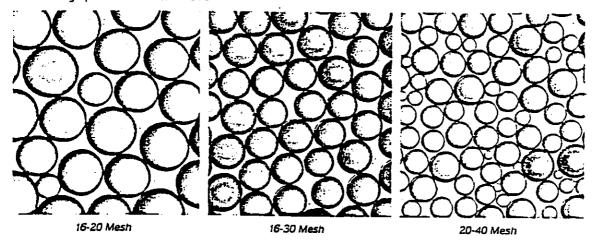
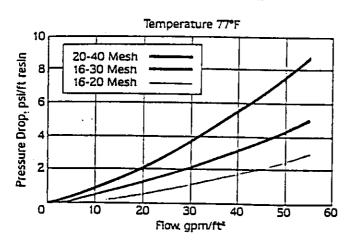


FIGURE 1

FIGURE 2

Pressure Drop vs. Flow Rate for DOWEX 21K Resin, CI Form



Backwash Expansion vs. Flow Rate with DOWEX 21 K Resin, CI Form

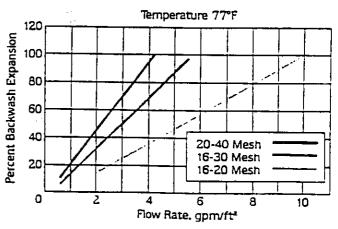


TABLE 3

0 - 14

Suggested Operating Conditions for DOWEX 21K Resin

pH Range

Maximum Temperature

Hydroxyl Form 140°F or 60°C Chloride Form 220°F or 100°C Minimum Bed Depth 30 inches

Service Flow Rate 2-4 gpm/cu ft (16 to 32 Bed

Volumes/hr)

Regenerants Dependent on the Particular

Application

Regenerant Level Dependent on Capacity Desired
Regenerant Flow Rate 0.25-1.0 gpm/cu ft (2.0 to 8.0

Bed Volumes/hr)

Regenerant Temperature

Rinse Water Requirement Displacement Rinse Rate Backwash Flow Rate Ambient Approximately 50 gal/cu ft

Equivalent to Regeneration Rate Sufficient to Produce at Least 50% Expansion in Bed Volume

Operating Conditions

DOWEX 21 K resin is shipped in the chloride form. When put in service, the chloride ion is replaced — or exchanged — with other anions. Ion exchange resins change in size due to ionic interaction. Consequently, they shrink or swell, depending on the particular species and liquid composition. The shrink or swell between the regenerated and exhausted forms will cause a shift in size distribution of the beads. This, in turn, causes a change in pressure drop characteristics. Table 3 summanzes suggested operating conditions.

NOTE: Each facility should evaluate and optimize its procedures to assure the most efficient uranium recovery.

Loading and Elution Behavior

The economics of a uranium recovery operation will be influenced to varying degrees by the loading and elution characteristics of the ion exchange resins. Generally, large volumes of low concentration feed liquor are passed through a bed of resin for uranium uptake. Upon elution, a smaller volume of more highly concentrated effluent is obtained for further treatment and recovery.

Minimization of capital and reduction of operating costs are related to the choice of resin for the system. The resin should have a preference and a selectivity for uranyl complexes, and a loaded resin must be elutable. These are in addition to the properties discussed above. Performance will also be affected by the proper density, size, and other factors related to fluid-

ization and pressure drop. Other considerations, such as uranium concentration, temperature, competing ions, impurity levels, and flow rates will also have an influence on the system's performance. In sulfate leaching systems, most of which operate in the pH 1.5 to 2.0 range, it has been found that uranium loading decreases as bisulfate levels increase.

As manufactured, an ion exchange resin contains a fixed number of functional groups or sites that are available for use. When uranium is being recovered from either acid or alkaline streams, four exchange sites on the resin must be used to recover one uranyltricarbonate, or one uranyltrisulfate complex. This is shown in equations 1 and 2, where R represents one functional group attached to the polymer backbone.

EQUATION 1 — CARBONATE LEACH SYSTEM
$$2R_2CO_3 + UO_2(CO_3)_3^{4-} \stackrel{}{\longleftarrow} R_4UO_2(CO_3)_3 + 2CO_3^{2-}$$
 Equation 2 — Sulfate leach system
$$2R_2SO_4 + UO_2(SO_4)_3^{4-} \stackrel{}{\longleftarrow} R_4UO_2(SO_4)_3 + 2SO_4^{2-}$$

For a resin with a total exchange capacity of 1.2 milliequivalents/milliliter, the theoretical capacity for uranyltrisulfate would be less than 1.2 meq/ml, depending upon equipment configuration and barren requirements.

Dow recognized that a high theoretical capacity resin may not give the highest operating capacity in an actual system. Therefore, a resin with a higher porosity was developed. The higher porosity increased the rate of uranyl complex exchange and thereby increased the operating loading. This change not only improved the loading efficiency, but also improved the sharpness of elution.

In acid systems, the ultimate loading capacity of a DOWEX 21 K resin is dependent upon the pregnant liquor temperature, ionic species present in solution, operating conditions, and resin properties. Generally, heating the pregnant liquor to 35-40°C will increase the loading significantly. However, as the temperature of the liquor is raised above 40°C, the improvement in resin loading begins to diminish.

While foulants are a major cause of lower resin loadings, the particular foulants vary from mill to mill. However, one ionic species common to all acid leach operations is the bisulfate anion (HSO $_4$). Bisulfate ions readily form at pH values less than 3, and they compete with uranyl-trisulfate species for resin sites.

In a similar way, uranium can also be removed with DOWEX 21 K resin in alkaline carbonate leach systems. Competition of chloride or carbonate with uranyl complexes for resin sites can result in lower operating capacities. As carbonate increases, the $\rm U_3O_8$ loading declines, as shown in Figure 3. The total loading for the 40 g/l Na₂CO₃ liquor was 34.5 g U₃O₈/l, and for the 60 g/l Na₂CO₃ liquor the loading was 20.7 g U₃O₈/l.

A typical loading curve for an acid leach is shown in Figure 4. The rapid rise in uranium concentration from initial uranium breakthrough to the final effluent concentration reflects the efficient utilization of DOWEX 21K ion exchange resin.

To minimize the cost of the chemicals used in the extraction and precipitation of uranium products, it is desirable to have a highly concentrated uranium feed solution. This objective is reached when the uranium elutes from the resin in a sharp, narrow band. Figure 5 shows several elution curves for different conditions in a carbonate leach system. The increase in NaCl concentration results in a sharper, concentrated eluent.

Commonly used eluents include sodium chloride, sodium carbonate, ammonium carbonate, or some mixture of these chemicals.

Additional data on acid or carbonate leach systems for loading and elution is available from The Dow Chemical Company upon request.

FIGURE 3

Column Loading from Low Uranium (0.1 g U₃0₈/l)

High Carbonate Leach Liquors

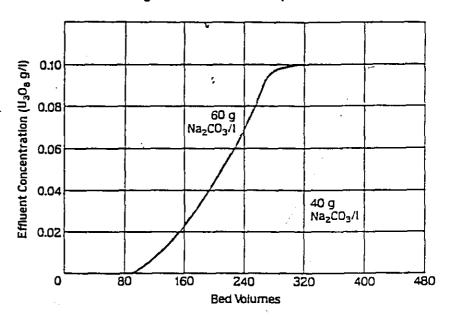
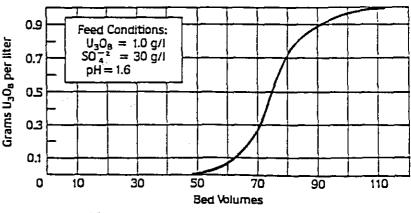


FIGURE 4

Typical Breakthrough Profile —

DOWEX 21 K, 20-40 Mesh, Resin



B-15



TYPICAL PROPERTIES REILLEXTM crosslinked poly-4-vinylpyridines

	Reillex [™] 402	Reillex [™] 425	Reillex [™] HP	Reillex TM HPQ
Appearance	granular powder	spherical beads	spherical beads	spherical beads
Weak Base Capacity	8.8 eq/kg dry	5.5 eq/kg dry 1.6 eq/l wet	5.5 eq/kg dry 1.6 eq/l wet	1.3 eq/kg dry 0.3 eq/l wet
Strong Base Capacity	none	none	none	3.3 eq/kg dry 0.9 eq/l wet
Water (%)	5-7	50-60	50-60	55-65
DVB (%)	2	25	25	25
Particle Size (U.S. Mesh)	-60/+200	-18/+50	-30/+60	-30/+60
Bulk Density	. <u>-</u>	39.5	38.5	41.7
Average Pore Size (A)	none	450	1000	1000

This information is for guidance in the use of this product only and, while the facts are believed to be reliable, suitability of this product for a particular application should be determined by their user. Comments concerning the possible use of this product in commercial applications are not intended as a recommendation to infringe on any existing patents. Every possible effort has been made to insure the accuracy of the information based on research done at Reilly Industries, Inc., but no guarantee is implied or should be inferred.

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TIGG Type MRM Adsorbent

Typical Physical Properties:		
Mesh Size, Tyler Sieve, 90% Min.	8x24	
Total Surface Area, m ² /g	100	
Pore Volume (Within Particle), cc/g	0.225	
Acid Soluble Ash, Wt.% Max.	3	
Moisture, Wt.% Max.	5	
Bulk Density, lbs./ft ³	40	

TIGG Type MRM is a granular adsorbent used for applications involving removal of inorganic and organic materials from potable and process water. Specific applications include removal of chlorine, fluoride, and heavy metals (including lead, cadmium, mercury, zinc, iron, copper, manganese, aluminium, arsenic, silver, strontium, cobait, vanadium, and chromium). Furthermore, TIGG Type MRM has applications for removing radionuclides (including radioisotopes of elements above and ruthenium, plutonium, and americium). Frequently, economical applications of TiGG Type MRM are capable of achieveing non-detectable effluent quality.

Most applications are initially plioted to determine long-term economics and to optimize operating conditions. Pilot design data is then used to select the most appropriate TIGG CANSORB vessel size to apply the adsorbent. TIGG Type MRM is backwashable, if the application requires this step.

Another grade, TiGG Type MRM - AG is a specially prepared aquarium grade material useful for similar applications in fisheries and fish hatcheries. This material is usually supplied in a 5x8 Tyler Sieve size.

While TIGG Type MRM is ususally supplied in our standard preengineered CANSORB liquid phase modular adsorbers, it can be supplied separately as makeup when spent beds in preexisting adsorbers are replaced. Please contact us to order pliot or commercial adsorbers containing TIGG Type MRM, or for ordering replacement adsorbert.

RL95-59.W51/A2 B-18

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APPENDIX C

DATA TABLES, DATABASE, AND SECONDARY CONTAMINANTS

RL95-59,W51/A2 C-1

RL95-59.W51/A2 C-2

Table C-1. Strontium-90 Data, 216-B-5 Reverse Well Pilot-Scale System, Run 1.

Mari aç		19	ell (Influent)		Bone Char	Muent		Lead Clino Colu	ımn Effluent		Polishing Col Char/Clin	umu (Bone ::	GA	C Column	System .
Date	Bed Volumes	5	Sr (pCI/L)	<	Sr (pCI/L)	Removal Efficiency	<	Sr (pCI/L)	Removal Efficiency	<	Sr (pCL/L)	Removal Efficiency	<	Sr (pCI/L)	Removal Efficiency
07-Nov-94	36.03		4730				Т		1				Т	40.1	99.2%
29-Nov-94	62.77		3950				1			1			Г	15.9	99.6%
01-Dec-94	93.05		3840		1990	48.2%	1						<	21.9	99.4%
18-Jan-95	199.85	_	_ 4410		2000	54.6%					l		1	10.5	99.8%
19-Jan-95	236.18		4430		3100	30.0%	1		1]		1	76.7	98.3%
20-Jan-95	262.68		4340		3350	22.8%			1		l		1	11.1	99.7%
23-Jan-95	301.94		4010	ı	3800	5.2%]	1.	I		-	10.5	99.7%
24-Jan-95	351.02		3210	ı	2470	23.1%	Ι.		1		1	·		40.8	98.7%
25-Jan-95	435.44		4050	ı	4030	0.5%								94.3	97.7%
26-Jan-95	515.66		-34. 9	ı	3280		I			1	[1	48.i	
27-Jan-95	595.87		4370	I				553	87.3%	1				151	96.5%
30-Jan-95	718.82		4390		3800	13.4%	<u>!</u>	937	75.3%	1_					
31-Jan-95	819.08		4570		4190	8.3%	<u> </u>	976	76.7%						
01-Feb-95	908.26		3620	<u> </u>	2920	19.3%	l	1120	61.6%						
02-Feb-95	1010.04		4060		4090			1190	70.9%						
03-Feb-95	1048.44		3040	Į .	1160	61.8%	ļ	3440		.	L		<u> </u>		
06-Feb-95	1174.77		4500	<u> </u>	Lost		I	848	81.2%	.			I		
07-Feb-95	1395.31		4030	l	3730	7.4%		1980	46.9%	ــــــ	l		<u> </u>		
08-Feb-95	1595.54		3880	<u> </u>	did not sample		1	nsufficient samp		_	1480				61.9%
09-Feb-95 10-Feb-95	1814.04 2018.16		3660	I	3910		<u> </u>	did not sample	<u>. </u>		4070				
15-Feb-95	2134.12	_	3230 3780				<u> </u>	2000	38.1%	ļ	1240	38.0%	 		61.6%
16-Feb-95	2232.25						<u> </u>	1650	56.3%	<u> </u>	995	39.7%	L		73.7%
17-Feb-95	2232.23		3750 3940				 	1940	48.3%	1_	1270	34.5%	L		66.1%
21-Feb-95	2390.06			<u> </u> _			 	1740	55.8%	┡	485	72.1%	<u> </u>		87.7%
22-Feb-95	2506.70	_	3630 3580	<u> — </u>			 ¦	1800	50.4%	1_	1150	36.1%	<u> </u>		68.3%
23-Feb-95	2588.68	-	3800	!				1840 1960	48.6%	!	1290	29.9%	 	<u></u> .	64.0%
24-Feb-95	2689.71	_	4130	! —				2020	48.4%	·}	1310	33.2%	<u> </u>		65.5%
27-Feb-95	2780.03		4320			— · · · · — — —	 	2040	51.1% 52.8%		1380	31.7%	-		66.6%
28-Feb-95	2880,29		4080	-			-	2300	43.6%	ł	1250 3090	38.7%			71.1%
01-Mar-95	2968.20	-	4050	! —∣				1910	52.8%	┢	1280	33.00			24.3%
02-Mar-95	3028.36	-	4070		-			2000	50.9%	1	1360	33.0% 32.0%			68.4%
03-Mar-95	3129.85		3810				H	2160	43.3%	 	1200	32.0% 44.4%	<u> </u>		66.6%
06-Mar-95	3213.96		1355 · ·					1860	43.376	 	1150	38.2%	<u> </u>		68.5%
07-Mar-95	3284.15	\neg	4050	[[155	96.2%	┢	1140	30.2 %	-		
08-Mar-95	3391.10		3250	-				1970	39.4%		1360	31.0%			71.9% 58.2%
09-Mar-95	3465.07	1	3900					2300	41.0%	1—1	1600	30.4%			59.0%
10-Mar-95	3548.62	—i	3890					2140	45.0%		1540	28.0%	<u> </u>		59.0%
13-Mar-95	3666.10	··	3890				[—·	1980	49.1%	1—	1320	33.3%			
14-Mar-95	3762,16	-1	3940				<u> </u>	2210	43.9%	 	1540	30.3%	-		66.1%
15-Mar-95	3869,12	— İ	-3.45					2210	73.7/0	1	1250	43.4%	-		60.9%
16-Mar-95	3950.72	T	3790	-	·		—i	2020	46.7%		1640	18.8%	-		56.7%
17-Mar-95	4033.17	_	3520					2140	39.2%	Н	1660	22.4%			52.8%
(3.3 ×	The Sagrage and Section					DUPLICATES	۳	2.70	37.270		1000	££.7 <i>R</i>			
09-Mar-95	3465.07 1	_				~ V= LACELED	1	2320	· · ·		· ·			· ·	<u>i Alle</u>
17-Mar-95	4033.17	-1					<u></u> —I	2320	,,_		1660				
						OFFSETS 1	480	RATORY RESU	A TC	ш	1660				
09-Mar-95	3465.07	_	т	•		OFFSILE L	ABUI		LIS	, —					
09-Mar-95	3465.07		i	<u>—</u> [2300							
U7-1712E-73	3403.07		i.			inn 1 an		2200			<u> </u>				
1202-1-						VERAGE VALU	ES								
AVERAGE	VALUES	1	3841	L	3188		1	1787		1 I	1482	7		47	

Table C-2. Cesium-137 Data, 216-B-5 Reverse Well Pilot-Scale, Run 1.

Physical Street Physical Street Chair Occided Stree							T					
156.00 1000		d Volumes		_	Removal		Removal		900	emoval		41.1
Column C		26 03	1040		ratio areas		change.	-	3	1	108	201.90
1976 1170		30.00	001					<u> </u>			15.0	08 78
199.28 1470 140 152.4. 140 153.4. 140 153.4. 140 140 153.4. 140		20.20	100	023	60 26	:			+	İ	+	08 3%
199.18 1900 1210 123.8 1900 1210 1210 123.8 124.18 1210 1210 123.8 124.18 1210 1210 123.8 122.1	-	25.05	0771	0/5	55.5 %			+	1		4	70.4.70
186. 1900 19		199.85		- 140	77.4%			:			2 3	25.5%
1906 1900		236.18	1500	1210	19.3%				- :		76.7	% V. 4%
State 1400 1176		262.68	1390	1240	10.8%						17.1	99.2%
186.00 1190		301.94	1430	1300	6.1%				 - -		10.5	99.3%
1,000		351.02	1360	1350	0.7%				•	:	40.8	97.0%
1865.57 1870 1870 1870 1875 26.57 18.57 18.57 18.50	+	435 44	0913	1440							94 3	93.0%
1985.37 1400 1400 1408 < 80.7 95.8%	+	27 213	02.01	7 - 7						:	48	07.4%
186,547 1400	+	212.00	0/81	:	i 1		2000		:			N
1985 1490 1450	+	595.87	1460				%8.56			İ	15	89.1%
187.98	\dashv	718.82	1420	1400	1.4%	262	81.3%					
988.26 1900.04 6 id 4 99.9% 100.044 1420 1420 < 35 99.9% 100.044 1410 1260 10 6% < 78 99.9% 1306.34 1410 1420 10 6% < 78 96.2% 1306.34 1410 1420 10 6% < 75 96.2% < 83.3 1306.34 1410 1410 3.0% < 85 < 87.3 3.5% 200.84 130 1420 3.0% < 87.2 8.3% < 87.4 3.5% 200.86 130 1420 3.0% < 87.7 9.6% < 87.4 2.7% 200.86 130 3.0% < 87.1 9.6% < 87.4 2.7% 200.87 130 130 < 87.1 9.6% < 87.5 < 87.6 < 87.6 200.87 130 10.0 < 87.7 9.6% < 87.6 < 87.6 < 87.6 < 87.6 < 87.6 <	-	819.08	1490	1650			96.5%					
100.044 14.00 14.00 16.05 < 78.6 94.5%	_	908.26	1360	1480		-	95.9%					
1984.44 1410 1260 10 6 % < 78 6 94.4%	H	1010.04	1420	1420		ļ.	95.9%					
1174.77 1570 1400 C 59 96.2% C 58.3 1410 1420 1420 C 55 96.2% C 58.3 1410 1420 1420 C 57.3 59.2% C 58.3 58.4 C 57.3 59.2% C 58.3 58.4 C 57.3 59.2% C 59.4 3.5% C 59.4	+	1048 44	1410	1360	10.6%	1	94 4%		-			
1395.34 1410 1430 6 55 95.2% 55.3 3 1810 143	+	1174 777	0251				26.790	1	-			
1395-34 1310 1430	+	4105 34	0/2	1,430			20.7.00	;	-		+	
1399.348 1300 1420 30% 6 95 8 95 8 95 8 95 8 95 8 95 8 95 8 95	+	1275.51	01410	0000		_	20.7%	1				80 30
218.14 1310 310%	+	35.55	1380	1470					3			25.6%
1313.12 1410 < 895 95 57.4 3.5% 222.23 1330 < 871 95.8% < 871 3.5% 223.23.3 1330 < 871 95.8% < 874 1000% 229.06.0 1310 < 871 95.8% < 874 27% 229.0.0 1310 < 871 95.8% < 874 27% 250.0 1310 < 875 87.8% < 875 0.0% 250.0 1300 < 875 87.8% < 88.5 0.0% 250.0 1300 < 87.9 87.8% < 88.5 0.0% 250.0 1300 < 87.9 87.5% < 88.5 0.0% 250.0 1300 < 87.9 87.5% < 87.9 3.5% 250.0 1300 < 87.9 87.5% < 87.9 3.5% 250.0 1300 < 87.9 87.5% < 87.9 3.5% 251.36 1300 < 87.9 87.5% < 87.9 3.5%		1814.04	1350	1310	3.0%						-	95.7%
215A.12 1350 57.3 95.8% < 99.1 100.0% 2286.31 1300 < 77.1 55.6% < 90 100.0% 2286.70 1310 < 87.1 55.6% < 87.4 2.7% 2286.70 1310 < 87.7 87.6 87.6 87.4 2.7% 228.18.6 1300 < 87.7 87.6 87.6 87.4 2.7% 228.18.6 1300 < 87.7 87.6 87.6 87.7 1.7% 228.18.6 1300 < 87.7 87.6 87.7 87.7 1.7% 228.1.6 1400 < 87.7 87.7 87.7 87.7 87.7 87.7 3173.45 170 87.7 87.7 87.7 87.7 87.7 87.7 3173.45 170 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7 87.7		2018.16	1410	-			95.8%			3.5%		95.9%
232.2.4 1330 437.2 95.7% 0 100.0% 2366.70 1300 73.1 95.7% 90.0% 2396.70 1310 79.0 95.5% 57.4 27% 2396.70 1310 58.7 95.5% 57.4 27% 2386.71 1300 58.7 95.5% 46.1 21.7% 2386.72 1300 55.9 95.5% 46.1 21.7% 238.73 1300 55.7% 55.7% 55.6 55.7% 266.30 1330 55.7% 55.6 55.8 <th< td=""><td></td><td>2134.12</td><td>1350</td><td></td><td></td><td></td><td>95.8%</td><td></td><td>_</td><td></td><td>-</td><td>95.6%</td></th<>		2134.12	1350				95.8%		_		-	95.6%
2386.70 1300 < 571.1 95.6% < 59.6 59.6 296.70 2396.70 577.4 2.7% 7.7% <td>_</td> <td>2232.25</td> <td>1330</td> <td></td> <td></td> <td>57.2</td> <td>95.7%</td> <td></td> <td></td> <td>00.00</td> <td>!</td> <td>100.0%</td>	_	2232.25	1330			57.2	95.7%			00.00	!	100.0%
2990.06 1310 5 99 95.5% 5.1% 2.7% 2506.70 1320 <td></td> <td>2285.21</td> <td>1300</td> <td></td> <td></td> <td></td> <td>95.6%</td> <td></td> <td></td> <td></td> <td></td> <td>95.4%</td>		2285.21	1300				95.6%					95.4%
2566.70 1320 < 88.6 95.6% < 58.6 0.0% 258.68 1320 < 87.7 95.6% < 88.6 0.0% 258.88 1300 < 87.7 95.6% < 46.1 21.7% 258.03 1400 < 85.9 95.6% < 88.2 2.6% 258.30 1300 < 65.9 95.6% < 59.6 2.6% 258.30 1300 < 67.7 95.6% < 59.6 2.6% 3028.36 1300 < 67.7 95.6% < 60.3 3.3% 313.96 1300 < 87.7 8.8 3.6% 313.96 1300 < 87.7 8.8 3.6% 313.96 1170 < 87.7 8.8 3.6% 354.16 1400 < 87.8 9.9 8.8 8.8 356.17 1400 < 87.4 8.8 8.8 8.8 356.17 120 < 87.4 8.8 <t< td=""><td>Ė</td><td>2390.06</td><td>1310</td><td></td><td></td><td></td><td>95.5%</td><td></td><td></td><td>2.7%</td><td></td><td>95.6%</td></t<>	Ė	2390.06	1310				95.5%			2.7%		95.6%
2588.68 1300 < 57.7 95.6% < 38.5 2780.03 1300 5.89 95.5% < 46.1 21.7% 2780.23 1400 5.99 96.9% < 38.2 2880.29 1370 6.12 95.5% < 5.96 3078.30 1330 6.12 95.5% < 5.96 3078.41 1330 6.17 95.5% < 5.96 313.95 1380 5.77 5.6% 334.13 180 5.7% 5.6%		2506.70	1320				95.6%			0.0%		95.6%
2689.71 1300 589 99.5% 46.1 21.7% 2800.03 1300 46.1 21.7% 2800.03 1200 61.2 99.2% < 5.6% 2968.20 1300 40.3 3.3% 3038.34 1300 57.7 5.6% 331.95 1300 57.7 56.3 3.3% 331.96 130 57.7 50.7% 346.10 1170 57.8 59.4% < 57.7 346.11 1100 59.4% 55.4% 57.7 346.10 1100 59.4% 50.4% 50.4% 346.10 100 <t< td=""><td></td><td>2588.68</td><td>1300</td><td></td><td></td><td></td><td>95.6%</td><td></td><td></td><td></td><td>-</td><td>95.5%</td></t<>		2588.68	1300				95.6%				-	95.5%
2780.03 1400 < 55.9 96.0% < 58.2 2.6% 2860.29 1300 5.7.9 3.3% 3028.36 1300 5.7.9 3.3% 3028.36 1300 3129.46 1300 6.0 3.5% 3.5% 3129.46 1300 6.1 95.2% 6.0 3.5% 339.10 1170 5.3 9.9 9.9 6.0 5.3 6.0 8.%	Н	2689.71	1300				95.5%		_	21.7%	_	96.5%
2880.29 1270 < 61.2 95.2% < 59.6 2.6% 3028.36 1330 59.9 95.2% < 57.9 33% 3129.36 1300 61 95.2% < 60 3.6% 3129.36 1380 53.7% < 60 3.6% 324.15 1380 53.7% < 60 8.% 324.15 1380 53.7% < 60 8.% 3465.07 1400 53.6 95.7% 58.8 3465.07 1400 59.4% 58.8 3465.07 1190 59.4% 50.4 3465.07 1210 DUPLICATES 57.6 56.4	\dashv	2780.03	1400				20.0%		-			95.8%
2968.20 1330 C 59.9 95.5% C 57.9 33% 3028.36 1300 C 57.1 95.6% C 60.3 3.3% 313.36 1380 C 6 61.7 95.7% C 60.0 8.7 324.15 1380 C 53.6 95.7% C 60.0 8.7 8.8		2880.29	1270				95.2%			2.6%		95.3%
3028.36 1300 < 57.1 95.6% < 60.3 3129.45 1270 61 95.2% < 60.3 324.13.96 1380 324.13.96 1380 3391.10 170 3465.07 1260		2968.20	1330				95.5%			3.3%		95.6%
1319.85 1270 C 61 55.2% C 58.8 3.6% C 58.1 3213.96 1380 C 58.7 55.7% C 600 C 600 3213.96 1380 C 53.6 69.7% C 53.8 6.8% C 600 336.10 1170 C 57.8 69.4% C 53.8 6.8% C 60.8 336.10 1400 C 57.8 69.4% C 58.8 4.4% C 60.8 336.10 1400 C 59.4 95.8% C 58.1 4.4% C 60.8 336.10 1400 C 59.4 95.8% C 60.4 C 60.4 336.11 1190 C 58.3 95.4% C 60.4 C 60.4 336.12 1190 C 57.4 95.8% C 60.4 C 60.4 4033.17 1210 DUPLICATES C 57.6 C 56.4 3465.07 C 4.8 C 60.4 C 60.4 3465.07 C 60.4 C 60.4 C 60.4 C 60.4 3465.07 C 60.4 C 60.4 C 60.4 C 60.4 3465.07 C 60.4 C 60.4 C 60.4 C 60.4 3465.07 C 60.4	3028.36	1300				95.6%		ξ.V.			95.4%	
3213.96 1380 < 5.83.7 95.7% < 60 3784.15 1380 53.6 96.1% < 50.77 348.10 1170 5.36 94.9% < 5.3.8 6.8% 346.61 1260 5.7% 58.6 356.10 1400 59.7% < 58.1 4.4% 356.10 1400 59.7% < 58.1 4.4% 356.12 1310 59.2% 55.8% < 60 5.1% 395.72 1280 59.5% < 56.2 5.1% 4033.17 1210 DUPLICATES 57.4 95.3% < 58.6 4033.17 1210 57.4 95.3% 58.6		3129.85	1270				95.2%			3.6%		95.4%
3264.15 1380 < 53.6 96.1% < 57.7 3391.10 1170 < 53.9 94.9% < 53.8 6.8% 3465.17 1260 < 67.7 87.8 95.4% < 58.6 < 68.8 3566.10 1400 < 67.8 99.7% < 58.1 < 44.7% 366.10 1400 < 60.8 95.7% < 56.2 51.% 366.10 1400 < 59.4 95.8% < 60.4 51.% 366.12 150 < 58.3 95.4% < 56.2 51.% 3950.72 1280 < 58.8 95.4% < 60.4 51.% 4033.17 1210 DUPLICATES 57.4 95.3% < 56.4 4633.17 1210 DUPLICATES 57.6 < 56.4 < 56.4 4633.17 1210 1210 4.8 < 56.4 < 56.4 4633.17 25.4 <td>_</td> <td>3213.96</td> <td>1380</td> <td></td> <td></td> <td></td> <td>95.7%</td> <td></td> <td></td> <td></td> <td></td> <td>95.7%</td>	_	3213.96	1380				95.7%					95.7%
3391.10 1170 599 94.9% < 55.8 6.8% 3465.07 1260 < 57.8 95.4% < 58.6 6.8% 3546.07 1400 < 6.00 59.7% < 58.1 4.4% 3569.12 1310 < 59.2 95.8% < 56.2 5.1% 3569.12 1300 < 58.3 95.8% < 56.2 5.1% 3569.12 1280 < 58.3 95.8% < 56.4 5.1% 3569.12 1280 < 58.8 95.4% < 57.1 2.1% 4033.17 1210 DUPLICATES 57.4 95.3% < 58.6 8.6 3465.07 1280 DUPLICATES 57.6 56.4 < 56.4 4033.17 4033.17 < 56.4 56.4 < 56.4 < 56.4 < 56.4 3465.07 2 2 2 2 2 < 56.4 < 56.4 < 56.4 3465.07		3284.15	1380				96.1%		1			95.8%
3465.07 1260 57.8 95.4% 58.6 3548.62 1400 6.6 59.7% 58.1 44% 3566.10 1400 59.2 95.7% 51.% 3569.12 1310 59.2 55.2 51.% 3569.12 1190 58.3 95.4% < 50.4 21.% 4033.17 1210 DUPLICATES 57.4 95.3% < 58.6 4033.17 1210 DUPLICATES 57.6 57.6 56.4 4033.17 0.0FFSITE LABORATORY RESULTS 56.4 3465.07 1 57.6 3465.07 1 3465.07 1	\dashv	3391.10	1170			59.9	94.9%			6.8%	_	95.2%
3546.62 1400 < 60.8 95.7% < 58.1 4.4% 3666.10 1400 59.4 95.8% < 50.0 4.4% 3763.16 1310 59.2 95.8% < 50.0 3950.72 1180 4033.17 1210 DVPLICATES 3465.07	_	3465.07	1260			-	95.4%		_		_	95.3%
3666.10 1400 59.4 95.8% < 60 3763.16 1310		3548.62	1400				95.7%		:	4.4%		95.9%
3763.16 1310	\dashv	3666.10	1400			_	95.8%			:		95.7%
3865.12 1190	\dashv	3762.16	1310			\rfloor	95.5%	\rfloor	-	5.1%		95.7%
3950.72 1280	_	3869.12	8				95.1%			2.1%	×*1	95.2%
4033.17 1210	\dashv	3950.72	1280				95.4%		4			95.3%
3465.07 DUPLICATES S7.6		4033.17	1210	_			95.3%		9		i	95.2%
3465.07					DUPLICAT	ES						A
3465.07	Н	3465.07										
3465.07 OFFSITE LABORATORY RESULTS AVERAGE VALUES	_	4033.17							4			
3465.07 < 11 < 11					OFFSITE	LABORATORY RE	SULTS					
3465.07 < 4.8 AVERAGE VALUES	\dashv	3465.07										
AVERAGE VALUES	-	3465.07					T	_				
					AVERAGE VA	١					٠	

Table C-3. Plutonium Data, 216-B-5 Reverse Well Pilot-Scale, Run 1.

		We	ll (Influe nt)	M, a	Bone Char I	(fluent	1	ead Clino Colu	mn Effluent	G/	AC Column	* System
Date	Bed Volumes	<	Pu (pCI/L)	٧	Pu (pCI/L)	Removal Efficiency	7	Pu (pCI/L)	Removal Efficiency	<	Pu (pCI/L)	Removal Efficiency
07-Nov-94	36.03	۷.	10	П						┪~	10	-9,
29-Nov-94	62.77	1	18.4							 	10	45.7%
01-Dec-94	93.05		30.1	<	20	33.6%				<	20	33.6%
18-Jan-95	199.85		32.5	<	10	69.2%				1	10	69.2%
19-Jan-95	236.18		33.5	<	10	70.1%				<	10	70.1%
20-Jan-95	262.68		44.6	<	2.26	94.9%				<	3.16	92.9%
23-Jan-95	301.94		26.9	<	2.18	91.9%				<	2.81	89.6%
24-Jan-95	351.02		29.6		3,47	88.3%			· · · · · · · · · · · · · · · · · · ·	<	2.68	90.9%
25-Jan-95	435.44		19.4		2.48	87.2%				<	2.42	87.5%
26-Jan-95	515.66		24.6	<	16.5	32.9%				<	3.24	86.8%
27-Jan-95	595.87		22.6				<	2.2	90.3%	<	2.63	88.4%
30-Jan-95	718.82		19.3	<	2.61	86.5%	<	2.48				87.2%
31-Jan-95	819.08		18.3	<	2.89	84.2%	<	2.23				87.8%
01-Feb-95	908.26		18.6	<	2.64	85.8%	7	12.2				34.4%
02-Feb-95	1010.04		16.7	<	2.13	87.2%	<	2.26		I	<u> </u>	86.5%
03-Feb-95	1048.44				P. A					 -	·	
06-Feb-95	1174.77		17.4	<	2.47	85.8%	<	2.39		\vdash		86.3%
07-Feb-95	1395.31		17.5	<	2.18	87.5%	<	5.76		<u> </u>		67.1%
08-Feb-95	1595.54		16.1	<	3.73	76.8%			· · · · · · · · · · · · · · · · · · ·	1	 -	
09-Feb-95	1814.04		15.5	<	4.77	69.2%						
10-Feb-95	2018.16		16.2				<	3.98	75.4%			75.4%
15-Feb-95	2134.12		I4.4				<	18.4				13,470
16-Feb-95	2232.25		· · · · · · · · · · · · · · · · · · ·									
17-Feb-95	2285.21		19.9	-			<	2.91	85.4%	<u> </u>		85.4%
21-Feb-95	2390.06	<	28				<	2.69	90.4%	l		90.4%
22-Feb-95	2506.70		15.3				<	2.11	86.2%			86.2%
23-Feb-95	2588.68		15.2					2.31	84.8%			84.8%
24-Feb-95	2689.71		16.6				<u> </u>	3.78	77.2%			77.2%
27-Feb-95	2780.03		17.4				<	2.55	85.3%	 	·	85.3%
'28-Feb-95	2880.29		18	1			<	2.78	84.6%			84.6%
01-Mar-95	2968.20	<	5.93				<u> </u>	6.17	04.070			04.070
02-Mar-95	3028.36		17.4	-			<	5.71	67.2%			67.2%
03-Mar-95	3129.85		17.6				7	5.61	68.1%			68.1%
06-Mar-95	3213.96	<	5.17	T			~	4.14	00.1 70			19.9%
07-Mar-95	3284.15		18.3	-			<u>`</u>	5.68	69.0%			69.0%
08-Mar-95	3391.10		18.4	÷	-		`	4.46	75.8%			
09-Mar-95	3465.07		15.9				-	6.6	58.5%		<u></u>	75.8%
10-Mar-95	3548.62		13.8	1			 	2.67	80.7%			58.5% 80.7%
13-Mar-95	3666.10		14.6				<	3.17	78.3%	┝┈		80.7% 78.3%
14-Mar-95	3762.16		16.4	-			<	3.32	79.8%	<u> </u>		79.8%
15-Mar-95	3869.12		16			·	<	2.62	83.6%			
16-Mar-95	3950.72		13.7				<	2.53	81.5%	\vdash	· · · ·	83.6%
17-Mar-95	4033.17	<	31.5				~	4.11	87.0%	 -		81.5%
. Ner'					DUPLICATES			7.11	. 07.070	<u> </u>		87.0%
09-Mar-95	3465.07			· · · ·		 -		5 62 1				
17-Mar-95	4033.17			-			<	5.52				
	4033.17				OFFGITE 2 4	DODATORY :	12022	TOC .		_		
09-Mar-95	3466.07 1					BORATORY R	ESU!	.15				
	3465.07				1.6		-					
09-Mar-95	3465.07				1.5							
								Mintion decreases	ama al deserv	15.71	g gere in mile	98
AVERAGE	VALUES		19.4	<	5.6	77.0%	<	4.4	79.4%		7.0	75.6%

		Well	l (Influent)		efore Bag Fülters		After Bag	Filters	Lea	d Clino Coli	ımn Effluent		Second Cline	o Column	System
Date	Bed Volumes	<	Sr (pCI/L)	<	Sr (pCI/L)	<	Sr (pCI/L)	Rem. Eff.	<	Sr (pCI/L)	Rem. Eff.	<	Sr (pCI/L)	Rem. Eff.	Rem. Eff.
11-Apr-95	94.38		4610				3840	16.7%		56	98.5%		48.6	13.2%	98.9%
13-Apr-95	227.83		4890		L]]			53.2	98.9%		28	47.4%	99.4%
17-Apr-95	354.15		4100		4380		4290			136	96.8%		78.6	42.2%	98.1%
19-Apr-95	424.68		4820		4790					289	94.0%		29.3	89.9%	99.4%
21-Apr-95	574.57		5380		5130					440	91.8%		47.7	89.2%	99.1%
24-Apr-95	697.90		5180		4840					298	94.2%		57	80.9%	98.9%
26-Apr-95	840.08		5690				4810	15.5%		387	92.0%		51.7	86.6%	99.1%
28-Apr-95	966.98		5440				4880	10.3%		631	87.1%		59.5	90.6%	98.9%
1-May-95	1015.85		4930				5300	_		537	89.9%		57.6	89.3%	98.8%
3-May-95	1145.78		5260				4970	5.5%		641	87.1%		67.3	89.5%	98.7%
5-May-95	1280.52		5340				4550	14.8%	1	828	81.8%		106	87.2%	98.0%
8-May-95	1322.71		5040				4800	4.8%		707	85.3%		82.5	88.3%	98.4%
10-May-95	1451.98		4440				5030		1	773	84.6%		101	86.9%	97.7%
12-May-95	1592.89		5250				4720	10.1%		1100	76.7%		128	88.4%	97.6%
15-May-95	1646.13		4710				5060			940	81.4%		128	86.4%	97.3%
17-May-95	1755.08		5210				5070	2.7%		934	81.6%		117	87.5%	97.8%
19-May-95	1850.51		4980				5100			998	80.4%		134	86.6%	97.3%
22-May-95	2146.95		6410				4670	27.1%		2230	52.2%		589	73.6%	90.8%
23-May-95	2410.22		4160							2060	50.5%		741	64.0%	82.2%
24-May-95	2689.39		4140							2240	45.9%		984	56.1%	76.2%
25-May-95	2964.66		4130				3990	3.4%	**********	2710	32.1%		1080	60.1%	73.8%
26-May-95	3198.00		3600				3740			2580	31.0%		1160	55.0%	67.8%
							DU	PLICATES							. 4.1
11-Apr-95	94.38		4740												
26-Apr-95	840.08						4950		1			i			
5-May-95	1280.52								1	785		1	į		·
12-May-95	1567.44									964		l			
19-May-95	1850.51				·					885		l	1		
24-May-95	2689.39								i	2390					
						01	FFSITE LAB	ORATORY	RESU						
11-Apr-95	94.38		4410												
5-May-95	1280.52		4690							784					
19-May-95	1850.51		4450				1		1	871					
	VERAGE VAL	LUE	S						:				·		
AVERAGE	VALUES		4896		4785		4676	11.1%		980	77.9%		267	74.5%	93.8%

Table C-5. Cesium-137 Data, 216-B-5 Reverse Well Pilot-Scale, Run 2.

		We	Well (Influent)		re Bag Pilters	Afte	r Bag Filters	Lead	Clino Column Effluent	Second Clino Column Effluent	
Date	Bed Volumes	<	Cs-137 (pCI/L)	<	Cs-137 (pCI/L)	<	Cs-137 (pCI/L)	<	Cs-137 (pCI/L)	<	Cs-137 (pCI/L)
11-Apr-95	94.38		1380				1000	<	59.1	<	56
13-Apr-95	227.83		1450					<	59.9	<	61
17-Apr-95	354.15		1400		626		280	<	59	<	58.9
19-Apr-95	424.68	`	1530		1360						
21-Apr-95	574.57		1490		1430				60.9	< ;	57.6
24-Apr-95	697.90	,	1620		1440			<	58		
26-Apr-95	840.08		1600				1390	<	60.2	<	59.3
28-Apr-95	966.98	·	1560		<u> </u>		1360	<	58.2	*	
01-May-95	1015.85	[` `	1480				1470	<	57.1		
03-May-95	1145.78	I	1760				1430	1		t	
05-May-95	1280.52		1420				1500	<	58.7		<u>-</u>
08-May-95	1322.71		1630							··· Ŧ	
10-May-95	1451.98	l	1570								
12-May-95	1592.89	l	1530		-		1590	1		< .	60.8
15-May-95	1646.13	l "	147							-	
17-May-95	1755.08	•	1540								
19-May-95	1850.51	l	1530				1500	<	57.6		
22-May-95	2146.95	·	1820				1720	<	58.2	< :	56
23-May-95	2410.22		1510								
24-May-95	2689.39		1420					<	60.2	:	
25-May-95	2964.66		1360	i							
26-May-95	3198.00		1310	,			1300	<	60.6	i	
					DUPLICATI	ES		_			
11-Apr-95	94.38	Ţ .	1440							<u> </u>	
26-Apr-95	840.08]					1460				
5-May-95	1280.52	1						<	59.4		
12-May-95	1567.44	1									
19-May-95	1850.51	1						<	58		
24-May-95	2689.39	1						<	69.9		
a, section	<u> </u>		Ol	FSITI	LABORATO	Y RI	ESULTS			'	
11-Арг-95	94.38		1510								
5-May-95	1280.52	†·	1550		,				6.76	-+	
19-May-95	1850.51	İ	1370					<	4.75		
	AVERAGE V	ALU			1,541 846					- 1	
AVERAGE			1457		1214		1322	<u> </u>	59.1	<	58.5

Table C-6. Technetium-99 Data, 216-BY Cribs Pilot-Scale System.

			Before Filter		After Filter		Lead Column	Effluent		Polishing Column	n Effluent	System
Date	Bed Volumes	<	Tc-99 (pCi/L)	<	Tc-99 (pCi/L)	٧	Tc-99 (pCi/L)	Removal Efficiency	<	Tc-99 (pCi/L)	Removal Efficiency	Removal Efficiency
26-Jan-95	251		4250		6600		118.0	98.2%				
1-Feb-95	306	l	5470		5090	<	60.9	98.8%	-			
3-Feb-95	380		5110		4720	<	67.9	98.6%				
7-Feb-95	544	1 .	6180		5910	<	79.4	98.7%				
8-Feb-95	582				6770	<	82.6	98.8%	<	82.3	0.4%	98.8%
9-Feb-95	619		8860		7030	<	134.0	98.1%	1			
15-Feb-95	703		10100		9230		317.0	96.6%				
17-Feb-95	750				9770		211.0	97.8%	1	352.0		96.4%
22-Feb-95	874		9280		8310	<	92.4	98.9%				
23-Feb-95	907		· · · · · · · · · · · · · · · · · · ·		9860	. <	102.0	99.0%		113.0		98.9%
27-Feb-95	1054		10200		9980		196.0	98.0%			·	
1-Mar-95	1118				9520	<	89.1	99.1%	1	160.0		98.3%
3-Mar-95	1187		9930		9640		177.0	98.2%	·			
6-Mar-95	1293				9310	<	82.6	99.1%	<	87.2		99.1%
10-Mar-95	1426		10900		10700	<	81.0	99.2%				
13-Mar-95	1533				9980		269.0	97.3%		442.0		95.6%
17-Mar-95	1674		10600		10700		112.0	99.0%			39.6%	99.4%
24-Mar-95	1797				10900	<	67.2	99.4%	<	67.7		-
30-Mar-95	1797		11200		10100		144.0	98.6%				
5-Apr-95	1898				9920		206.0	97.9%		200.0	2.9%	98.0%
10-Apr-95	2006		10500		9320	<	146.0	98.4%				
12-Apr-95	2080				10700	<	160.0	98.5%	<	158.0	1.3%	98.5%
19-Apr-95	2217		10800		9630		271.0	97.2%				
21-Apr-95	2283		10800				162.0		<	83.8	48.3%	
26-Apr-95	2372		8910		9030		188.0	97.9%				
28-Apr-95	2440	j			10000	<	166.0	98.3%	<	138.0	16.9%	98.6%
4-May-95	2549		8130		11300		271.0	97.6%				
16-May-95	2782	'	9830	- 	9760		700.0	92.8%				!
18-May-95	2821		6960		8430		342.0	95.9%	<	84.8	75.2%	99.0%
					Offs	te La	boratory Results					
4-May-95	2549	Ţ	10500	1			251.0					
18-May-95	2821				8590	ľ				1		

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Table C-7. Cobalt-60 Data, 216-BY Cribs Pilot-Scale System.

	1	Before Filter After Filter					Lead Column	E <i>ff</i> luent	ļ ;:·	Polishing Colum	System	
Date	Bed Volumes	<	Co-60 (pCi/L)	<	Co-60 (pCi/L)	<	Co-60 (pCi/L)	Removal Efficiency	<	Co-60 (pCi/L)	Removal Efficiency	Removal Efficiency
26-Jan-95	251		95.5		92.1	<	40.7	55.8%				
1-Feb-95	306	1	92.2	i	100.0	<	40.7	59.3%				
3-Feb-95	380]	133.0	1	56.3	<	36.6	35.0%			Ì	
7-Feb-95	544		158.0		124.0	<	42.1	66.0%				
8-Feb-95	582	l '			111.0	<	37.8	65.9%	<	38.9		65.0%
9-Feb-95	619	1	136.0	1	157.0	<	37.8	75.9%	1	•	t .	
15-Feb-95	703		142.0		146.0	i	ND					<u> </u>
17-Feb-95	750	1		1	172.0	<	40.7	76.3%	<	41.7		75.8%
22-Feb-95	874	[-	147.0		183.0	Ī	37.7	79.4%				
23-Feb-95	907			-i	173.0	<	42.7	75.3%	<	39.6	7.3%	77.1%
27-Feb-95	1054		187.0		126.0	<	37.4	70.3%				
1-Mar-95	1118			 	170.0	<	41.8	75.4%	<	41.0	1.9%	75.9%
3-Mar-95	1187		166.0		186.0	<	43.1	76.8%				
6-Mar-95	1293				169.0	<	33.4	80.2%	<	35.8		78.8%
,10-Mar-95	1426		185.0	Ţ	176.0	<	41.4	76.5%			[, (
13-Mar-95	1533				167.0	<	42.4	74.6%	<	35.0	17.5%	79.0%
17-Mar-95	1674		147.0		150.0		36.6	75.6%			100.0%	100.0%
24-Mar-95	1797			-	187.0	<	40.0	78.6%	<	40.6		
30-Mar-95	1797		169.0		163.0	<	36.6	77.5%				
5-Apr-95	1898			<u> </u>	157.0	<	36.6	76.7%	<	37.8		75.9%
10-Apr-95	2006		145.0		165.0	<	39.2	76.2%				
12-Apr-95	2080]	170.0	<	41.4	75.6%	<	42.7		74.9%
19-Apr-95	2217		169.0		172.0	<	41.1	76.1%				
21-Apr-95	2283	i	161.0			<	43.7		<	39.6	9.4%	
26-Apr-95	2372	1	157.0		166.0	<	36.6	78.0%				l
28-Apr-95	2440				178.0	<	38.5	78.4%	<	37.8	1.8%	78.8%
4-May-95	2549		202.0	-	181.0	<	42.8	76.4%	_		<u> </u>	i
16-May-95	2782		197.0	-[i	150.0	<	32.0	78.7%] · · · · · · · · · · · · · · · · · · ·	[
18-May-95	2821		177.0	1	161.0	<	43.7	72.9%	<	35.0	19.9%	78.3%
					Of	site L	aboratory Results					
4-May-95	2549		168.0			<	10.5			i		
18-May-95	2821				147.0						1	

C-9

Table C-8.

216-B-5 Reverse Well Secondary Contaminant Data.

(6 Sheets)

HEIS # DATE LOCATION PORT Anions, mg/l (ppm) Alkalinity Cl- \overline{F} -NO3 S04 **Extraction Well** 18.4 0.26 BOFBN6 11-Apr-95 AC-1 101 2 30.4 SC-55B 19.2 BOFBN7 103 0.18 30.3 11-Apr-95 Bone Char Effluent 2.1 BOFBN8 11-Apr-95 Lead Clino Effluent SC-62B 103 19.3 0.2 2.1 29.6 SC-68B 19 0.22 2.08 29.8 BOFBN9 Second Clino Effluent 108 11-Арг-95 **BOFJF8** 5-May-95 Extraction Well AC-1 103 18.9 0.38 2.18 29.5 AC-2A 102 0.37 2.01 29.2 BOFJG2 5-May-95 Before Bag Filter 18.4 BOFJG0 5-May-95 AC-3 102 5.54 0.43 62.1 After Bag Filter 4.6 BOFJF9 Lead Clino Effluent SC-62B 102 18.2 0.35 2.15 29.1 5-May-95 SC-68B 29.1 BOFJG1 5-May-95 Second Clino Effluent 18.3 0.36 2.08 103 Extraction Well AC-1 16.5 0.39 2.23 BOFL52 19-May-95 104 27 BOFL54 19-May-95 AC-3 27 After Bag Filter 103 16 0.4 2.08 26.6 BOFL55 19-May-95 Bone Char Effluent SC-55B 104 15.9 0.38 2.09 SC-62B 26.9 BOFL53 19-May-95 Lead Clino Effluent 104 16.4 0.37 2.1 Second Clino Effluent SC-68B 104 27 BOFL56 19-May-95 16.2 0.38 2.12 Duplicates Duplicates BOFBN6 11-Apr-95 Extraction Well 30.4 AC-1 0.28 102 17.8 1.98 BOFJG2 5-May-95 Before Bag Filter AC-2A 103 18.4 0.37 2.02 29.3 BOFL52 19-May-95 **Extraction Well** 104 15.9 0.4 26 AC-1 2.13

Cations, ug/I (ppb) Zn Al Cu Ba Be Ma:-14508000 1248 140400 858 213720 15132000 3744 139620 1014 265200 365 36 23.6 18.7 37.8

Ca Fe Mg K Na BOFHG6 27-Apr-95 Raw Clino Resin 8658000 2667600 1825200 18720000 16224000 13416 BOFHG7 27-Apr-95 Raw Clino Duplicate 9750000 Resin 3174600 2004600 17550000 16068000 17004 BOFBN6 11-Apr-95 Extraction Well AC-1 28300 85.2 9140 5560 22600 70.6 BOFBN7 11-Apr-95 Bone Char Effluent SC-55B 28100 117 9470 4960 24500 45.4 BOFBN8 11-Apr-95 Lead Clino Effluent SC-62B 3210 130 9940 1180 54000 32.1 BOFBN9 11-Apr-95 Second Clino Effluent SC-68B 2790 372 2350 1180 70700 36.3 BOFJF8 5-May-95 **Extraction Well** AC-1 30400 93.5 9700 7610 24000 83.8 BOFJG2 5-May-95 Before Bag Filter AC-2A 29800 1990 9540 7040 23300 1120 BOFJG0 5-May-95 After Bag Filter AC-3 29600 225 9460 6630 23200 63.2 BOFJF9 5-May-95 Lead Clino Effluent SC-62B 29300 201 9550 7380 24100 27.5 BOFIGI 5-May-95 Second Clino Effluent SC-68B 29500 119 9430 6130 24300 22.9 77.14 BOFL52 19-May-95 **Extraction Well** AC-1 30300 88 9460 6930 23100 109 BOFL54 19-May-95 After Bag Filter AC-3 30800 78.7 9780 7770 24000 68.4 BOFL55 19-May-95 Bone Char Effluent SC-55B 31400 1200 9960 7000 24400 168 BOFL53 19-May-95 Lead Clino Effluent SC-62B 30800 226 9810 8500 24100 69.6 BOPL56 19-May-95 Second Clino Effluent SC-68B 30400 338 9760 8730 24800

HEIS #

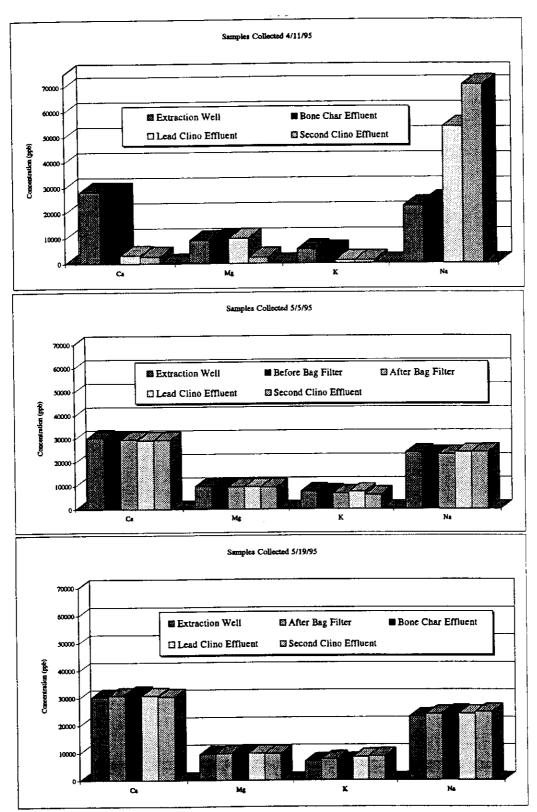
DATE

All to Markey St. St. Co.

LOCATION

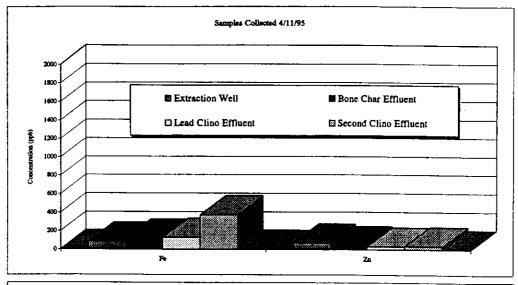
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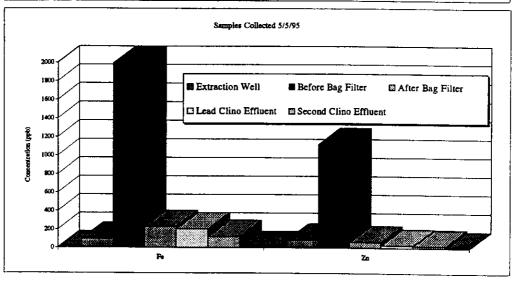
Table C-8. 216-B-5 Reverse Well Secondary Contaminant Data. (6 Sheets)



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Table C-8. 216-B-5 Reverse Well Secondary Contaminant Data. (6 Sheets)





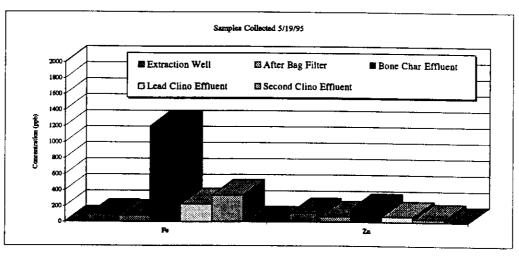


Table C-8. 216-B-5 Reverse Well Secondary Contaminant Data. (6 Sheets)

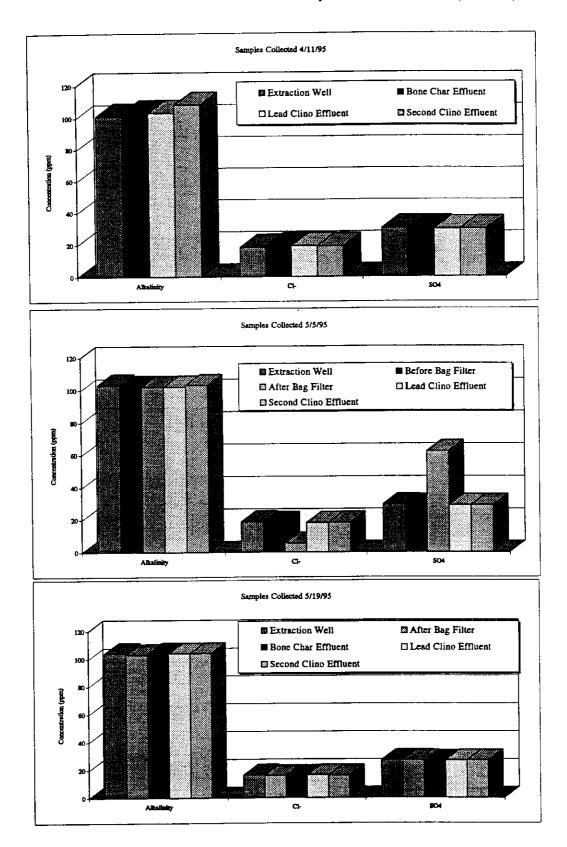


Table C-8. 216-B-5 Reverse Well Secondary Contaminant Data. (6 Sheets)

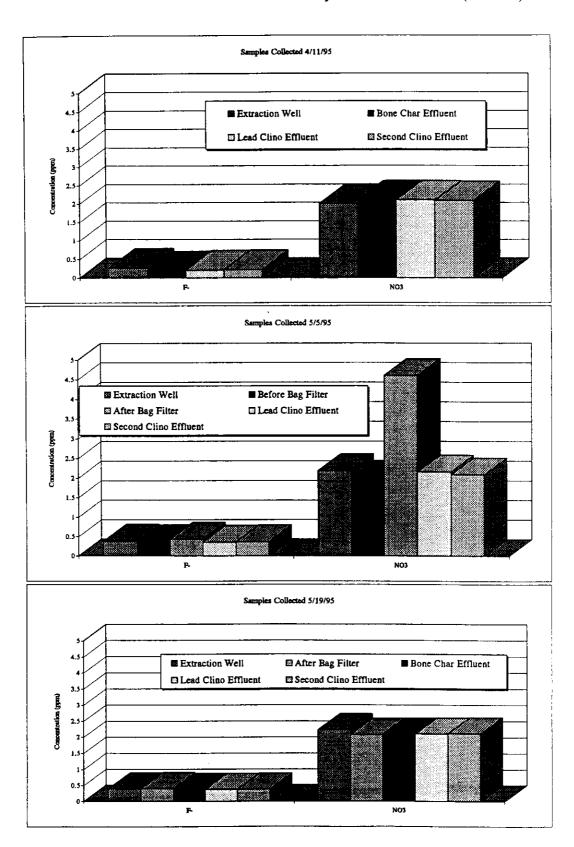


Table C-9.
216-BY
Cribs Se
econdary
Table C-9. 216-BY Cribs Secondary Contaminant Data. (4 S.
Data.
(4 Sheets)

						Anions		usezanê g j.
HEIS#	DATE	LOCATION	PORT			mg/l (ppm)	N. P. Bei	1.70
	*.			Alkalinity	Cl-	F-	NO3	SO4
BOFJK8	4-May-95	Before Filter	BV-31	72.4	43.3		95.7	346
BOFJK9	4-May-95	Lead Column Effluent	BV-53B	74.4	44.4		98.3	353
BOFJL0	4-May-95	After Filter	BV-51A	72.4	42.1		97.2	348
BOFL57	18-May-95	After Filter	BV-51A	79.6	37.2	0.66	96.3	332
BOFL98	18-May-95	Lead Column Effluent	BV-53B	,	38.4	< 0.50	99.5	339
	1 11 _ 1		X*ilea					
						Duplicates	File tagety set	
BOFJK8	4-May-95	Before Filter	BV-31	76.4	42.5	0.65	97.7	348
BOFL57	18-May-95	After Filter	BV-51A	79.7	37.6		92.2	332

HEIS#							Cations			erikanes (1. j.)
HEIS#	DATE	LOCATION	PORT		kg i Silleni		ug/l (ppb)			
				Ca	Fe	Mg	K	Na	Zn	Cu **
BOFJK8	4-May-95	Before Filter	BV-31	200000	299	58300	12500	48600	199	25.9
BOFJK9	4-May-95	Lead Column Effluent	BV-53B	202000	74.5	59100	13200	49500	93.6	14.7
BOFJL0	4-May-95	After Filter	BV-51A	205000	333	59600	12600	50000	162	27.9
BOFL57	18-May-95	After Filter	BV-51A	201000	489	58700	13500	49600	128	14.2
BOFL98	18-May-95	Lead Column Effluent	BV-53B		·		• ·		į	
							•	•	*	

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Table C-9. 216-BY Cribs Secondary Contaminant Data. (4 Sheets)

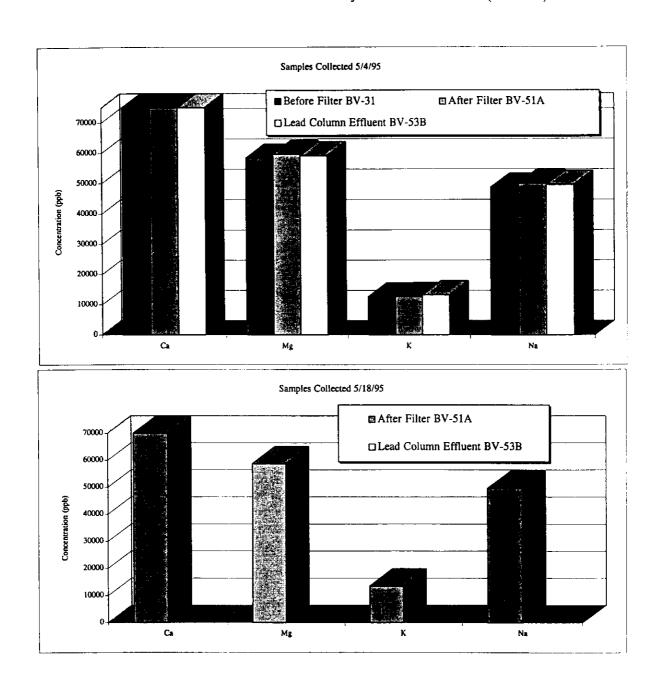
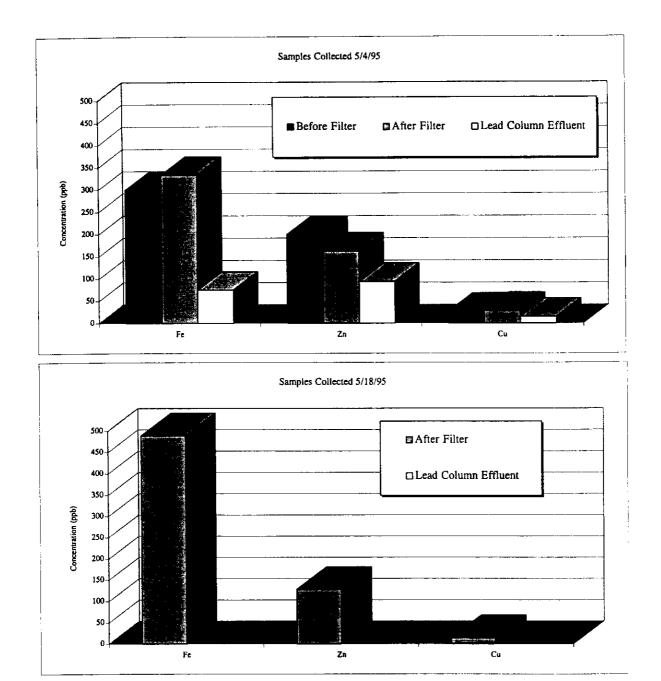


Table C-9. 216-BY Cribs Secondary Contaminant Data. (4 Sheets)



9613494.0170 DOE/RL-95-59 Rev. 0

Table C-9. 216-BY Cribs Secondary Contaminant Data. (4 Sheets)

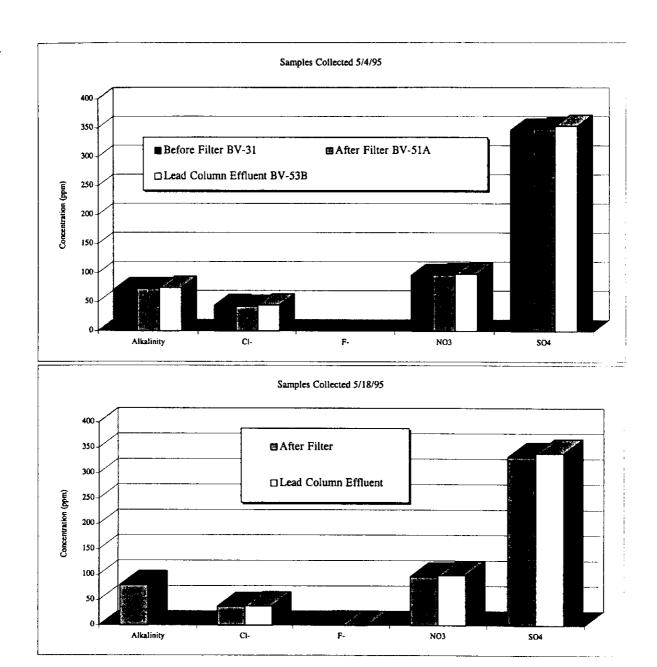


Table C-10.

Batch Equilibrium Test Data (216-B-5 Reverse Well).

Sr Loading Residual Pu Pu Loading Residual Cs-137 Cs-137 Loading Residual Sr-90 (pCi/g) Adsorbent & Dose (g/L) (pCi/L) (pCi/g) (pCi/L) (pCi/g) (pCi/L) Initial Conc. Clino (AmRec) 0.2 Clino (AmRec) 0.5 Clino (AmRec) 2.0 Clino (AmRec) 20 Clino (Resin Tech) 0.2 Clino (Resin Tech) 0.5 Clino (Resin Tech) 2.0 Clino (Resin Tech) 20 Bone Char (RT) 0.2 Bone Char (RT) 0.5 Bone Char (RT) 2.0 Bone Char (RT) 20 Bone Char (TIGG) 0.2 Bone Char (TIGG) 0.5 Bone Char (TIGG) 2.0 Bone Char (TIGG) 20 IRC-718 0.2 IRC-718 0.5 IRC-718 2.0 IRC-718 20

Table C-11. Batch Equilibrium Test Data, Dowex 21K (216-BY Cribs).

Adsorbent & Dose (g/L)	Residual Co-60 (pCVL)	Pu Loading (pCVg)	Residual Tc-99 (pCVI)	Tc-99 Loading (pCVg)
Initial Conc.	29		1480	
Dowex 21K 0.2	19	51	487	4965
Dowex 21K 0.5	0	58	195	2570
Dower 21K-2.0	8	10	55	712
Dowex 21K 20	46	0	4	74

Table C-12. Clinoptilolite Flow Through Column Test Data.

Bed Volumes	Cum Bed Volumes	Sr-90 (pCi/L)	Removal Efficiency
0	0	11400	
104	104	73.6	99.4%
64	168	88.3	99.2%
112	280	154.6	98.6%
17	297	242.9	97.9%
66	363	279.7	97.5%
136	499	647.7	94.3%
25	524	1067.2	90.6%
47	571	1435.2	87.4%
111	682	1023	91.0%
86	768	1369	88.0%
64	832	1346.9	88.2%
44	876	1361.6	88.1%
47	923	1361.6	88.1%
50	973	1567.7	86.2%

Table C-13. Isotherms. (3 Sheets)

Adsorbent & Dose	SrCl2 Addition (ppm)	Residual Sr- 90 (pCi/L)	AVG Residual Sr-	Sr-90 Loading (PCi/g)	Residual Cs-137 (pCL/L)	Avg Residual Cs- 137	Cs-137 Loading (pCVg)	Residual Pu (pCUL)	Arg Residual Pu	(2/104)
Initial conc.		3790	=average(B3,B4)	=(\$B\$2-C3)/0.4	1390	=average(E3,E4)	=(\$E\$2-F3)/0.4	18.8	=average(E3,E4)	=(\$H\$2-L3)/0.4
Bottechar 0.4 g/l	2 ppm	4300	4300	0	1480	1490	0	4.6	4.525	35.7
Bonechar 0.4 g/l		lost			1500			4.45		
Bonechar 4.0 g/l	2 ppm	lost	1990	450.0	1330	1395	0	4.39	4.455	3.6
Bonechar 4.0 g/l	2 ppm	1990			1460			4.52		
Bonechar 20 g/l	2 ppm	178	991	181.2	1370	1340	2.5	6.8	6.865	9.0
Bonechar 20 g/l	mdd 7	154			0161			4.83		
Bonechar 0.4 g/l	2 ppm	4140	3890	0	1500	1430	0	15	16.6	22.2
Bonechar 0.4 g/l	S ppm	3640			1360			4.82		
Bomechar 4.0 g/l	S ppm	2120	2880	227.5	1390	1385	1.3	4.75	4.705	3.5
Bonechar 4.0 g/l	2 ppm	2150			1380			4.66		
Bonechar 20 g/l	2 ppm	159	172	180.9	1280	1305	4.3	4.79	4.61	0.7
Bonechar 20 g/l	S ppm	185			1330			4.43		
Bomechar 0.4 g/l	0.2 ppm	4190	3970	0	1350	1420	0	92'9	6.825	29.9
Bonechar 0.4 g/l	0.2 ppm	3750			1490			68.9		
Bornechar 4.0 g/l	0,2 ppm	1510	1555	558.8	1290	1345	11.3	13.1	9.035	2.4
Bonechar 4.0 g/l	0.2 ppm	0091			1400			4.97		
Bossechar 20 g/l	0.2 ppm	201	270	176.0	1370	1340	2.5	4.54	6.585	9.0
Bonechar 20 g/l	0.2 ppm	436			1310			8.63		
				1.						
Cline 0.4 g/l	Nome	2900	3050	1850.0	279	256	2835	14.2	14.05	11.9
Clino 0.4 g/l	None	3200			233			13.9		
Cline 4.0 g/l	Nome	257	257	883.3	70.7	63.3	331.7	7.92	20.05	0
Clino 4.0 g/l	None				55.8			13.4		
Cline 20 g/l	None	52.4	59.25	186.5	76	59.2	66.5	8.6	8.6	0.5
Cline 20 g/l	None	66.1			42.3					
Durasil 0.4 g/l	2 ppm	2710	2760	2575.0	141	141	3115	12.9	13.05	6.11
Durasii 0.4 g/l	2 ppm	2810			147			13.2		
Durasil 4.0 g/l	2 ppm	391	370	855.1	63	62.4	331.9	12.7	12.4	1.6
Durasil 4.0 g/l	2 ppm	348			8.19			12.1		
Durasil 20 g/l	2 ppm	71.4	57.00	186.7	63.9	61.7	66.4	12.3	12.2	0.3
Durasil 20 g/l	2 ppm	42.6			59.4			12		
Durasil 0.4 g/l	5 ppm	2950	2900	2225.0	160	130.5	3148.75	13.7	14.5	11.9
Durasil 0.4 g/l	5 ppm	2850			101			15.3		
Durasil 4.0 g/l	S ppm	412	227	890.7	47.4	54.6	333.9	19.3	24.6	0
Durasil 4.0 g/l	S ppm	42.7			8.19			59.9		
Durasil 20 g/l	5 ppm	54.6	53.90	8.981	44.2	52.6	6.99	13.1	12.7	0.3
Decree 1 20 g/l	S ppm	53.2			19			12.2		
Duratel 0.4 g/l	0.2 ppm	1440	2020	4425.0	104	109.5	3201.25	15.7	16.15	6.11
Durani 0.4 g/l	0.2 ppm	2600			115			9.91		
Durasil 4.0 g/l	0.2 ppm	350	357	858.3	2.65	\$.09	332.4	15.4	15.9	0.7
Duradil 4.0 g/l	0.2 ppm	364			61.2			16.4		
Duried 20 c/l	0.2 ppm	54.8	51.15	6'981	58.5	56.6	66.7	14.4	14.4	0.2

SrC12 Addition

Residual Sr

28

45.3

0.2 ppm

36.65

187.7

AVG Residual Sr-

Sr-90 Loading

Pu Londing

Table

C-13.

Isotherms.

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Sheets)

Adsorbent & Dose Avg Residual Pu (ppm) * 90 (pCi/L) 90 (pCi/g) 137 (pCl/L) 137 (pCl/L) (pCVz) (pCVg) Initial *conc. 3790 =average(B3,B4) =(\$B\$2-C3)/0.41390 =average(E3,E4) =(\$E\$2-F3)/0.4 18.8 =average(E3.E4) =(\$H\$2-J3)/0.4 UOP A-51 0.4 g/l 2 ppm 1130 1105 6712.5 402 372 2545 15.2 14.4 11 UOP A-51 0.4 g/l 1080 342 2 ppm 13.6 UOP A-51 4.0 g/l 2 ppm 9.5 8.14 945.5 60.5 58.8 332.8 9.2 2.4 UOP A-51 4.0 g/l 6.78 57.1 9.4 2 ppm UOP A-51 20 g/l 16.4 11.5 188.9 63.3 62.5 4.6 5.4 0.7 2 ppm 66.4 UOP A-51 20 g/l 6.59 2 ppm 61.6 6.15 UOP A-51 0.4 g/l 1220 1145 357 12 5 ppm 6612.5 375 2582.5 15.2 14 UOP A-51 0.4 g/l 339 5 ppm 1070 12.8 UOP A-51 4.0 g/l 4.09 7.8 945.5 61 52.1 334.5 9.49 2.5 5 ppm 9 UOP A-51 4.0 g/l 5 ppm 11.6 43.2 8.5 UOP A-51 20 g/l 7 58 5 ppm 8.03 189.2 59.5 66.5 4.98 5.4 0.7 UOP A-51 20 g/l 5.97 60.9 5 ppm 5.9 UOP A-51 0.4 g/l 0.2 ppm 664 774.5 7538.8 310 341.5 2621.3 14.8 14.1 11.875 UOP A-51 0.4 g/l 0.2 ppm 885 373 13.3 UOP A-51 4.0 g/l 0.2 ppm 12.3 11.1 944.7 55.1 59 332.8 9.7 10.5 2.1 UOP A-51 4.0 g/l 0.2 ppm 9.93 62.8 11.2 UOP A-51 20 g/l 0.2 ppm 5.69 9.05 189.05 58.9 61.2 7.28 9.8 0.5 66.4 UOP A-51 20 g/1 0.2 ppm 12.4 63.5 12.3 Clino 0.4 g/l 2820 2425 207.5 2 ppm 2730 2956.3 13.95 12,125 237 14.7 Clino 0.4 g/l 2 ppm 2910 178 13.2 Clino 4.0 g/l 2 ppm 302 275.0 878.8 40.2 334.9 50.5 13.6 14 1.2 Clino 4.0 g/l 2 ppm 248 60.8 14.4 Clino 20 g/l 45.35 2 ppm 53.8 187.23 64.7 62.9 66.4 11.7 11.2 0.4 Clino 20 g/l 2 ppm 36.9 61 10.6 Clino 0.4 g/l 3215 5 ppm 3060 1437.5 148 191.5 2996.3 12.25 16.375 10.6 Clino 0.4 g/l 5 ppm 3370 235 13.9 Clino 4.0 g/1 5 ppm 335 348.5 860.4 40.7 51.15 334.7 15.5 14 1.2 Clino 4.0 g/l 5 ppm 362 61.6 12.6 Clino 20 g/l 5 ppm 43.6 44.85 187.26 60 58.3 8.61 66.6 11.0 0.4 Clino 20 g/l 5 ppm 46.1 56.6 13.3 Clino 0.4 g/l 2610 0.2 ppm 2620 2950 232 248 2855 13.5 14.4 11.125 Cline 0.4 g/l 0.2 ppm 2600 264 15.2 Clie 4.9 g/l 0.2 ppm 39.4 34.7 122.4 916.9 49 335.3 12.7 13.4 1.4 Climo 4.0 g/l 0.2 ppm 210 58.2 14.1 Cline: 20 g/1 0.2 ppm

61.3

61.4

61.35

66.4

9.65

12

10.8

0.4

Residual Cs-

Avg Residual Cs-

Cs-137 Loading

Residual Pu

Clino 20 g/1

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'n	
•	_

Adsorbent & Dose	SrC12 Addition (ppm)	Residual Sr- 90 (pCi/L)	AVG Residual Sr-	Sr-90 Loading (pCi/g)	Residual Cs- 137 (pCl/L)	Avg Residual Cs-	Cs-137 Loading (pCVg)	Residual Pu (pCVL)	Avg Rasidual Pu	Pu Loading (pCVg)
Initial conc.		3790	=average(B3,B4)	=(\$B\$2-C3)/0.4	1390	=average(E3,E4)	=(\$E\$2-F3)/0.4	18.8	=average(E3,E4)	=(\$H\$2-13)/0.4
IRC-718 0.4 g/l	2 ppm	2700	2600	2975	1350	1400	0	9.17	7.63	27.925
IRC-718 0.4 g/l	2 ppm	2500	·		1450			6.09		
IRC-718 4.0 g/l	2 ppm	4.63	5.45	946.1	785	779	152.8	9.76	8.285	26.2875
IRC-718 4.0 g/l	2 ppm	6.27		 -	773			6.81		
IRC-718 20 g/l	2 ррт	25.2	17.9	188.6	464	443.5	47.3	6.92	7.13	29.175
IRC-718 20 g/l	2 ррт	10.5			423			7.34		
IRC-718 0.4 g/l	5 ppm	1840	1960	4575	1350	1385	12.5	4.6	8.05	26.875
IRC-718 0.4 g/l	5 ppm	2080		-	1420			11.5		
IRC-718 4.0 g/l	5 ppm	6.14	4.72	946.3	806	799.5	147.6	7.3	8.485	25.7875
IRC-718 4.0 g/l	5 ppm	3.3			793			9.67		20.2075
IRC-718 20 g/l	5 ppm	5.53	4,72	189.3	486	473.5	45.8	4.59	10.645	20.3875
IRC-718 20 g/l	5 ppm	3.91			461			16.7		20.005
IRC-718 0.4 g/l	0.2 ppm	2610	2590	3000	1400	1390	0	7.11	7.19	29.025
IRC-718 0.4 g/l	0.2 ppm	2570			1380			7.27		22 2075
IRC-718 4.0 g/l	0.2 ppm	7.24	12.4	944.4	634	649	185.3	4.28	5.485	33.2875
IRC-718 4.0 g/l	0.2 ppm	17.6			664			6.69	100	26.1
IRC-718 20 g/l	0.2 ppm	5.24	9.17	189	419	409.3	49	4.85	4.36	36.1
IRC-718 20 g/l	0.2 ppm	13.1			399.5		<u> </u>	3.87		
A WISTING	d disk									-10.5
UOP E-95 0.4 g/l	2 ppm	2560	2595	2987.5	124	134	3140	26.9	23	-10.5
UOP E-95 0.4 g/l	2 ppm	2630			144		222	19.1	10.65	20.375
UPO E-95 4.0 g/l	· 2 ppm	430	453.5	834.1	61.2	61.9	332	11.3	10.65	20.373
UPO B-95 4.0 g/l	2 ppm	477		106.6	62.6	(1.0		15.8	12.95	14.625
UOP E-95 20 g/l	2 ppm	48.6	57.8	186.6	61.2	61.9	66.4		12.93	14.023
UOP E-95 20 g/l	2 ppm	67	2210	1200	62.6	171.5	3046.3	10.1	13.35	13.625
UOP E-95 0.4 g/l	5 ppm	3350	3310	1200	191 152	1/1.3	3040.3	14.1	13.33	13.025
UOP E-95 0.4 g/l	5 ppm	3270	502.5	821.6	40	51.6	334.6	13.2	12.65	15.375
UPO E-95 4.0 g/l	5 ppm	539	503.5	821.0	63.2	31.0	334.0	12.1	12.03	13.575
UPO E-95 4.0 g/l	5 ppm	468	58.75	104.4	70	66.2	66.2	8.8	9.15	24.125
UOP E-95 20 g/l	5 ppm	57.4	38./3	186.6	62,4	00.2	00.2	9.5	7.13	27,14-2
UOP E-95 20 g/l	5 ppm	60.1	2325.5	3661.3	113	114	3190	13.5	14.5	10.75
UOP R-95 0.4 g/l	0.2 ppm	2470	2323.3	5,1005	115	114	3170	15.5	17.5	10.75
UOP R-95 0.4 g/l	0.2 ppm	2181	407	845.8	60.8	61	332.2	15.9	15.2	9
UPO E-95 4.0 g/l	0.2 ppm	410	407	843.6	61.9	01	332.2	14.5	13.4	
UPO E-95 4.0 g/l	0.2 ppm	404	41.2	187.4	84.4	70,3	66	10,5	10.75	20.125
UOP E-95 20 g/l	0.2 ppm	43	41.2	107.4	56.2	10.3	- 00	11	10.73	20.120
UOP E-95 20 g/l	0.2 ppm	39.3	L		30.4		<u> </u>	I	<u> </u>	<u> </u>

Table C-14. Strontium-90 Isotherms.

TEST	DESCRIPTION	pН	Residual Sr-96 (pCi/L)	AVG Residual Sr-90	St-90 Loading (pG/g)	TOT Sr (ppm)
				=AVERACE(B3,D4	#(SD\$10-#3)/8.2	
1	.2 g/l	8.33	3920	3985	3025	
1	2 g/l	8.34	4050			· · · · · · · · · · · · · · · · · · ·
	2 0 g/l	8.36	441	470.5	2059.75	
1	2.0 g/i	8.37	500			· · · · · · · · · · · · · · · · · · ·
1	20 g/l		23.8	24.4	228.28	
	20 0 g/i	8.65	25			
	Blank/Feed	8.33	4590			"
	Blank/Feed	8.1				0.164
14	2 g/l	6.3	4410	4370	3000	
ia	2 g/l	6.3	4330			
16.	2.0 g/l	6.3	889	845	2062,5	
1:	2.0 g/l	5,8	801			
la	20 g/l	6.3	56	46.9	246.155	
la	20 g/i	6.4	37.8			
la .	Blank/Feed	6.3	4970	<u> </u>		
18	Biank/Feed	8.5				0.167
2	2.0 g/l	8.1	528	551	2514.5	0.101
2	2.0 g/l	8	574			
	20 g/i	8.3	62.1	62.65	275.8675	
2	20 g/i	8.3	63.2			
	Blank/Feed	8	5580			· · · · · · · · · · · · · · · · · · ·
	Blank/Feed				***	0.159
	Blank/Feed					0.159
	6 g/l		119	137.5	845.4166667	
	6 g/l		156			
	60 g/l		26	28.85	863,525	
	60 g/l		31.7			
	Biank/Feed		5210			**************************************
	Blank/Feed				······································	0.426
	Stock Soln		4.			175

Table C-15. MCT-1 Analytical Data.

44 S-94 La Gall	62	z :	z į z	z	z	z.	z	2 :	z. 2			_	_			1	1		1		T	1	1					T		П
Scriet In CE-137 Per (CCLL)	+ +		2 2	1.3	8.9	7.7	2.9	6.5		+					+	1	1	\dagger						. P. A.				1		_
Servet Land Ca-137 Date Date Date D		+ +		•	ш.		-+	-		-	H				4	-	4	4	_	4	+	+	+			-	_	-		L
Serve La Ca-137 Der Process Der Ca-137 Der Process Der Ca-137 Der <t< td=""><td>. Z</td><td>; Z</td><td>zz</td><td>z</td><td>2</td><td>z</td><td>z</td><td>z :</td><td>z :</td><td>-</td><td>L</td><td></td><td></td><td>_</td><td>4</td><td>4</td><td>4</td><td>4</td><td>+</td><td>+</td><td>+</td><td>+</td><td>z</td><td>-</td><td></td><td>-</td><td></td><td>_</td><td>,</td><td>-</td></t<>	. Z	; Z	zz	z	2	z	z	z :	z :	-	L			_	4	4	4	4	+	+	+	+	z	-		-		_	,	-
Scriet In CE-137 Der Pra (pCIL) Der Pra (pCIL) Der CE-137 Der Pra (pCIL) Der <	262				ļ l	Ш		59.2	20.7	;													58.4							L
Sr-96 La Cb-137 Der			* *	>	-	-	-	>	> 3	-	L						_	_	۲	1	\downarrow	\perp	۲		Ï					L
Servet In CE-137 Description CE-137 Description	369	98	230	224	281	247	236	230	987	à									1220				807							
Scriet In CE-137 Des Des CE-137 Des	z z	z z	z >	z	z	z	z	z	z :	-															N					٨
Scriet In CE-137 Des Des CE-137 Des	3,00	3.00	28 7	6.51	10.7	5.46	16.4	5.11	98.	-									İ						8.8					0.742
Sr-96 La Ca-137 Data Pa (Calla) Data Ca-137 Data Pa (Calla) Data Sr-96 Data	z z	2	z z	z	z	z	z	z	z ;	z 2	z	z	۲,	z		z	z	>-	z	2	2 :	, ;	zz	١.	2	z	>-	z		z
Sr-96 La Cb-137 Data Sr-96 Data Cb-137 Data Br (bc/12) Data Cb-137 Data Br (bc/12) Data Cb-137 Data Br (bc/12) Data Cb-137 Data Br (bc/12) Data Da	56.8	56.8	25.2	57.8	5.09	9.99	56.3	57.9	20.0	0.50	61.5	58.2	61.5	9.69		58.5	57.3	24.6	9:59	61.5	2	3,5	è 9		2.95	39.6	58.3	59.3	-	-1.59
Sr-96 La Cb-137 Data Sr-96 Data Cb-137 Data Br (bc/12) Data Cb-137 Data Br (bc/12) Data Cb-137 Data Br (bc/12) Data Cb-137 Data Br (bc/12) Data Da	-	- >	> >		>	-	>	>	> :	- >	. >-	-	٨	٨	>	-	>	>	>	>	> :	- ;			>	>	>	٨		,
Sr-96 La Ca-137 Dat Pa (Calla) Dat Sr-96 Dat Ca-137 Dat Pa (Calla)	30.05	1430	001	924	1120	1200	0101	99	1220	1570	2180	3070	3530	2300	2700	8	3020	2300	2550	2200	1820	00+7	2030		1230	280	3620	2080	1 7 m 14	98 ==
Sr-96 La Ca-137 Dat Pa (Calla) Dat Sr-96 Dat Ca-137 Dat Pa (Calla)	zz	z	z : 2	z	z	z	z	z	z	1					!	-	1	1		1	1	1	$^+$		┢					H
Scriet Dave Child Dave (house) Dave Scriet Dave Child Dave (house) Dav			 -	+	•		·	-	4.88	+	<u> </u>			_										Ì		-				-
Sr-96 La Cb-137 Dat Pa GCH2) Dat GCH37 Dat GCH37 Dat GCH27 GCH27 Dat GCH27 GCH		_ ! . !	_ :_					!	i	t	+	<u> </u>				-	-	-		+	+	\dagger	+	l	-		_			ŀ
Sr-96 Day CG-137 Day Pr. (GC/LL) Day (Sr-96) Day 13470 V 1340 V 1310 V 1310 V 1340 V 1460 V 1467 V 1460 V	╅	++	-					-	\dashv										•									_		-
Sr-96 Day Ch-137 Day Pa (PC/IL) Day Sr-96		++	- 2	- , >	- >		<u>~</u>	>	>	t				_	-			-	Υ.	-	-+	+	>			_	-			-
Seve De G-137 De Pe (COLL) 4310 Y 1540 Y 137 4310 Y 1600 Y 147 4650 Y 1600 Y 147		•	-	÷	-			H	4030	Ì								1	4840				3720							-
S-W C-137 Dr C-137 Dr 3470 Y 1340 Y 1440 Y 4310 Y 1460 Y	<u>- - </u>	- -	z¦ 2	<u> </u>	z	z	<u> </u>		z	<u> </u>	t			<u> </u> 							1	1	\dagger	1	_	<u> </u>	_			H
Sr-96 Par (Cs-137 Par (pCUL) 7 (540 V 4310 V 1440 V 4550 V 1460 V	4 4	4 4	33.9	13.1	14.9	77	14.3		6.17	0	-	 		-								†	+							
Sr-96 Pa Cr-137 6CVL) V (6CIL) 3470 Y 1340 4310 Y 1440 4630 Y 1460	> >		<u> </u>	· - >-	· >-	· ;-	 -	Н	> 1	> 3	- >-	>	>	>	٨	٨	٠,	~	٨	۸-	-	٠,	- >	l	-			H		ŀ
Sr-96 Day 3470 Y 4310 Y 4630 Y	+	\dashv	99.	1270	1310	1310			1290	/88	876	767	1150	180	1360	1410	1420	1620	1580	1530	0291	+	98 9			-	-	_		-
Sr-96 (PCUL) 3470 4310 4630		 > >	٠-: ١		. ~	! ~	 	<u> </u>	۰,	- ;		,	,	>	Y	٨	٨	۸.	¥	*	-	-	» »		-	\vdash	-	\vdash		H
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Com Red Volumers 0 535 973		452	100	2 4	419	ĘŠ.	4340		4220	2 5	4530	526	5030	5120	4930	\$190	5260	4180	4570	521	2470	20,00	30,70	1.	L	_		Ц	TORY	L
 	4630	1471	1768	2526	2628	2863	3588	3364	3517	200	4122	\$33	4780	5151	2552	\$104	233	3	7125	7363	7513	8		DUPLICATES	3517	9166	3	7446	OFFSITE LABORATORY DATA	2628
24 255 256 257	4630	1471	1768	2526	2628	2863	3068	336	3517	, S	\$	918	1363	163	7034	1697	2967	3368	3607	3846	Š		207	L	3517	:	¥	3846	OFFSTE	- 2428
3 2 2 3	973 4630 1471 4520	š š	17-Feb-95	27-Feb-95	28-Feb-95	03-Mar-95	ler-95	13-Mar-95	56-Je	13. Apr. 95	18-4 pr 95	19-4 pr 95	¥.	25-Apr-95	27-Apr-95	62-May-95	94-May-95	6.4	. S.	SG-483	18-657-95	<u>ا</u>	26.246-45		115-Mar-95	17-Apr-95	****	16-May-95	3	28-Feb-95

Table C-16. MCT-2 Analytical Data.

				* MCT-3	3					ACT-3-2	4					ACT-3.3A	F-34					77.297	7		
1		100	7	1 7 7 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	C. Dane	000	Par Dance	\$ (1/0)) Dames	S C	i Defec	1 20	T. Detec	S CHO	C Dates	33	Ca Dates	COO E	P. Daniel	200	1	\$ 137 \$ 023	100	P. 60011	Į
G-Feb-95	•	3790	٨	1390	>	+	<u></u>	737	- >	1260	>	9.57	2	9.15	z	57.6	2	5.52	z	97) }	28 7	z	2.84	Z
14 Page 18	37.565	4190	>	1520	>	5	>	3990		1450	>	6.93	_	218	t	57.6	z	6 16	z	4.58	z	. 65	2	4.23	2
B-Feb-95	27.27	4290	¥	1420	>	3.0	>	4210	>	1470	>	4.31	!	1400	>	88.8	z	477	Z	줐	+	50.2	7	5 05	z
Feb.5	141.8	28	>	1340	¥	14.7	>	4120	>	1460	>	156	z	1760	>	601	z	7.68	2	177	>	58.2	z	9 58	z
7-Feb.75		4050	>	1290	,	13.7	· -	3810	<u></u>	1330	 -	=	╌	1470	<u></u>	57.6	z	15.7	iz	598	×	58.9	L z	34.7	z
23-Feb-95	2369.42	3950	>	1330		123	 }	3900	·-	1310	 -	502	_	8	>	55.6	!z	5 62	z	705	<u> </u>	986	z	95	z
27-Feb-95	2526.06	4090	*	1370	>	10.5	· ·	4140	×	1380	>	E)	:	1360	>	58.8	z	3.85	Z	550	, ,	1 65	·		z
28.54.55	2628.25			:		 		3820	· -	1370	·		-	1430	<u> </u> }-	- 85	z	5.24	z	594	×	- 20	2	8 52	z
C3-Mar-45	2163.27		•	-		_	_	3820	·	1410	>	5 95	z	1510	>	583	z	8 85	z	631		585	2	10.2	z
98-Mar-95	344B. PS		•		• • • •	•	:	4020	>	1290	>	9.51		1360	· - ->	516	z	7.54	z	528	*	1 53	z	4 07	z
D-Mar-95	1343.99	3780	>	1320	 >-	12.2		3920	· >	1290		4 37		1480	 -	795	z	88	z	505	,	55.5	· · ·		iz
S-Mar-95	3517.27		-	•		_		4140	>	1260	 >-	. B 26		1840		57.4	z	16.2		75.8	-	57.5	: '2	5:5	
			DUFLICATES	-																					ſ
23-Feb-95	2369.62		ŀ	ľ	-		F	-			-	l	-	1460	,	58.9	Z	176	<u></u>		-	-	-	ŀ	T
27-Feb.95	2526.06				-		-	1330	 >-	59.5	z	4 45	! :Z	-	-	•		+		1. 1. 1.	+			+	
56-3414-95	3068.05			_	• • •			3820	>	1280	>	7 89	z			•	1	!		+	-	ļ.		-	
13-X4-45	3,63.5		i.	_	•••		-				:	+	l		-	-		-	1	501	^		-		1

DOE/RL-95-59

Table C-17.

2MCT-2 Strontium-90 Data

FEED 2MCT-2-2 2MCT-2-3 2MCT-2-4 System Sr-90 Sr-90 Sr-90 Sr-90 Rem Eff Date Bed Volumes Sr Detect Rem Eff Sr Detect Rem Eff Sr Detect Rem Eff Sr Detect (pCi/L)(pCi/L) (pCi/L) (pCi/L) 24.68 13-Apr-95 556 87.6% 4.94 99.9% 3.78 23.5 4470 Y Υ Ν 99.9% 17-Apr-95 199.54 2920 Y 30% 2.92 N 99.9% 2.77 N 5.1 99.9% 4170 Y 19-Apr-95 405.24 Y 4480 Υ 96.7% N 4920 8.9% 164 5.33 96.8 99.9% 21-Apr-95 631.52 Υ 5120 0% 619 Υ 87.8% 8.3 Ν 98.7 99.8% 5070 25-Apr-95 816.65 192 Υ 96.3% 5.44 Ν 97.2 99.9% 5120 Υ 27-Apr-95 1017.22 432 Υ 91.2% Υ 99.96% 4930 Υ 2.11 99.5 02-May-95 1248.64 4970 Υ 4.2% 487 Ý 90.2 90.6% 5190 Υ 04-May-95 1482.12 2850 Υ 45.82% 816 Ÿ 84.5% 102 \overline{Y} 87.5 98.1% 5260 1693.99 09-May-95 Ŧ 648 84.5% 61 Υ 90.6 98.5% 4180 Υ 11-May-95 1913.41 0% Y 5310 Υ 1010 77.9% 130 Υ 87.1 97.2% 4570 Υ 16-May-95 2152.03 569 Y 89.2% 78.7 Υ 86.2 98.5% 5280 18-May-95 2301.51 76.4% 1280 Υ 149 Υ 88.4 97.3% 5420 Υ 22-May-95 2523.68 700 Y 85.0% 74.6 Υ 89.3 98.4% 4670 24-May-95 2723.90 1080 Y 76.9% 96.2 Υ 97.9% 4680 91.1 26-May-95 2888.46 1090 Υ 72.5% 129 88.2 96.8% 3970 **DUPLICATES** 19-Apr-95 405.24 138 Υ 21-Арг-95 631.52 6.69 Ν 18-May-95 2301.51 118 Y

Table C-18. 2MCT-3 Strontium-90 Data.

		71	2MCT-	3-2		2MCT-	3-3		2MCT-	3-4	System		FEED
Date	Bed Volumes	<	Sr-90 (pCi/L)	Rem. Eff.	<	Sr-90 (pCi/L)	Rem. Eff.	<	Sr-90 (pCi/L)	Rem. Eff.	Rem. Eff.	<	Sr-90 (pCi/L)
13-Apr-95	24.7		322	92.8%	<	2.5	99.2%	<	5.08		99.9%		4470
17-Apr-95	199.5		2630	36.9%		4.01	99.8%	<	14.3	†	99.7%	I	4170
19-Apr-95	405.2	l · · · · i	4500	8.5%	- 1	140	96.9%]	5.82	95.8%	99.9%	ļ	4920
21-Apr-95	631.5		4710	7.1%	İ	397	91.6%	<	6.08	98.5%	99.9%		5070
25-Apr-95	816.7					238	95.4%	<	10.2	95.7%	99.8%		5120
27-Apr-95	1017.2					371	92.5%		5.64	98.5%	99.9%		4930
2-May-95	1248.6		41.3	99.2%		471	90.9%		8.69	98.2%	99.8%		5190
4-May-95	1482,1		5120	2.7%	1	976	80.9%		60.8	93.8%	98.8%		5260
9-May-95	1694.0					631	84.9%		37.8	94.0%	99.1%	1	4180
11-May-95	1913.4			T		1150	74.8%		69.7	93.9%	98.5%		4570
16-May-95	2152.0	[5270	0.2%		640	87.9%		38.5	94.0%	99.3%		5280
18-May-95	2301.5					966	82.2%		82.7	91.4%	98.5%		5420
22-May-95	2523.7				1	940	79.9%		83.5	91.1%	98.2%		4670
24-May-95	2723.9					978	79.1%		95.4	90.2%	98.0%		4680
26-May-95	2888.5					777	80.4%		79.1	89.8%	98.0%		3970
m ex emistrati	-1				DUP	LICATES	:						
13-Apr-95	24.7	-						<	4.31				
2-May-95	1248.6						†i		10.00	 			
11-May-95	1913.4			T			 		67.50	ti			
22-May-95	2523.7	1				979.00		i		 			 -
26-May-95	2888.5	.		i l	i i		† .	·	79.70			+	

Table C-19. Mini-Column Test 1 Analytical Results.

Bed Volumes	Tc-99	Tc-99	Removal	C- 60	0.60	T _
Treated	Feed	Effluent	Efficiency	Co-60 Feed	Co-60	Removal
	(pCi/L)	(pCi/L)	Tc-99 (%)	(pCi/L)	Effluent	Efficiency
	(20.2)	(penz)	10-99 (70)	(PCI/L)	(pCi/L)	Co-60 (%)
205	9,010	125	99	146	43.1	70
820	10,900	656	94	146	37.4	74
1,435	8,880	90	99	160	38.5	76
2,050				142	41.4	71
2,460	9,960	725	93	162	37	77
2,870	11,300	1,100	90	160	42.7	73
2,993	10,700	1,350	87	156	40.7	74
3,050	10,800	1,560	86	142	41	71
3,100	10,600	1,660	84	174	32	82
3,550	11,400	3320	71	164	39.6	76
3,755	10,600	4170	61	151	35.4	77
3,895	11,300	4900	57	167	39.2	77
4,042	10,900	8880	19	171	39.3	77
4,141	11,100	8140	27	155	39.6	74
4,264	10,500	7400	30	166	41	75
4,387	9,890	7840	21	160	41	74
4,526		-		155	38.9	75
4,674	10,900	7980	27	135	40.3	70
4,858	9,620	11200	-16	182	33.8	81
5,043	10,500	6710	36	162	42.1	74
5,207	10,900	10400	5	166	38.1	77
5,395	10,700	12800	-20	154	37.4	76
5,576	10,800	11000	-2	166	35.8	78
5,740	6,440	12500	-94	166 34.6		79
5,900	8,760	6540	25	149	36.6	75

C-30

RL95-59.W51/A2

Table C-20. Mini-Column Test 2 Analytical Results.

Bed Volumes Treated	Tc-99 Feed (pCi/L)	Tc-99 Effluent (pCi/L)	Removal Efficiency Tc-99 (%)	Co-60 Feed (pCi/L)	Co-60 Effluent (pCi/L)	Removal Efficiency Co-60 (%)
205	9,010	84	99	146	38.5	74
820	10,900	696	94	146	39.6	73
1,435	8,880	97	99	160	43.9	73
2,050				142	38.5	73
2,460	9,960	840	92	162	38.5	76
2,870	11,300	677	94	160	38.5	76
2,993	10,700	2,040	81	156	37	76
3,050	10,800	2,200	80	142	42.1	70
3,100	10,600	2,230	79	174	43.1	75
3,550	11,400	4,040	65	164	39.6	76
3,755	10,600	4,520	57	151	38.5	75
3,895	11,300	5,310	53	167	38.5	77
4,042	10,900	6,970	36	171	37	78
4,141	11,100	7,380	34	155	35.9	77
4,264	10,500	8,140	22	166	39.3	76
4,387	9,890	7,000	29	160	41	74
4,526				155	36.2	77 ,
4,674	10,900	9,510	13	135	39.6	71
4,858	9,620	433	95	182	37.4	79
5,043	10,500	7,930	24	162	46	72
5,207	10,900	10,000	8	166	37	78
5,395	10,700	12,100	-13	154	38.9	75
5,576	10,800	11,600	-7	166	33.4	80
5,740	6,440	11,000	-71	166	38.5	77
5,900	8,760	8,150	7	149	37.4	75

Table C-21. Pilot-Scale System Run 1. (4 Sheets)

papg 1)	(TIDO D	jaarag 45	(T/DIF) IS	en Datect	(9/9)) 14	MOA	NOLLVOOT	BYZE	EVA N
Y	1370	Å	09\$+	Ā	2.11	TOY.	ISV negeroos	M-q52-10	16000
A	0721	, ,	4200	λ	4. 7 <u>I</u>	I-OV	lisW nonsettxd	(8-40-9-9)	7301S
, A	1410	, A	4030	Ä	\$.T1	VC-1	lew nonears.	26-46-4-77	20115
<u> </u>	1380	, ,	3880	Ā	1.91	I-DV	Extraction Well	08-1-90-02	SSUIS
- X	1320	, ,	0998	, ,	1.01	1-DV	Istraction Well	26-49-1-60	65118
<u> </u>	1410	X.	3730	, ,	7.91	VC:1	lleW nouseuxd	10-6-69-95	291162
- X	1320	J.	3780	, X	14.4	I-OV	listraction Well	12:1-49-62	Ç9 18
<u> </u>	1330	, X	3750	Ĭ.	9,91	YC+1	Extraction Well	36-49-31	201168
Į.	1300	Ĭ.	3940	X.	6.91	1:3¥	listraction Well	\$6- 99-1 -21	12118
Ĭ.	1310	, J	0E9E	N	87	VC+1	Extraction Well	36-4-2-18	21565
X	1320	, ,	3580	,	15.3	YC+1	Letracion Well	56-494-27	21268
X	1300	Ā	3800	Į.	12.2	I-DV	Hew noneartest	56-49d-EZ	17216
, ,	1300	Ä.	4130	Ä	9.91	1:0Y	liaW nousebad	24-H-H-95	PEZIS
	1400	<u> </u>	4350	, ,	4.7I	I-DV	lieW neusanx3	\$6- 49-1 -72	77.518
, ,	1270	<u> </u>	0804	Ā	81	YC-1	lieW goussond	\$6-4 -21- 87	458 (5
, X	1330	Å	4020	N	£6.8	I-DV	Haw nonemed	CO-MM-TO	OPETS
, ,	1300	, A	0704	Å	t 7 I	YC:1	lisW neitsettxH	26-16M-SD	CHETE
, X	1270	Ā	3810	Ä	9.71	YC-1	How mouseand	29-18M-E0	PA:CLS
Å	1380	Å	SSEI	N	LI'S	VC-1	HeW nonsental	89-18M-80	opers
X.	1380	Ā	4030	Å	£.81	I-O¥	Extraction Well	26-44M-T0	ESEIS
Ā	1170	Y	3250	, ,	18.4	1:08	Haw notherned	26-15M-80	\$19.6
Į.	1760	, X	3900	Å	6,21	LOA	IOM HORORAL	26-18M-90	65915
, ,	1400	Y	3890	Å	8.51	I-DY	Extraction Well	26-18IM-01	EVP16
Å	1400	Ā	3890	Ä	9.41	I-DV	Extraction Mell	CO-MAIN-CL	99915
, A	1310	Ā	3940	Å	16.4	FOV	Extraction Well	86-18M-41	69 2 15
Ā	0611	Ā	31.5	Ä	91	VC-1	Haw nousemed	29-mM-21	74DIS
	1280	λ	3790	Å	7.51	I-DY	Extraction Well	\$6-J#IM-91	52915
- A	1210	Ā	3250	N	31.5	1-07	Haw gomestrad	C6-MM-LT	31 9 18
	1040	, ,	4730	N	10	TH-DV	gelote Ball Fillers	96-AON-LO	81982
Å	1180	Ä	3950	Ā	4.81	VCUT	refore Bag Filters	76-AUN-62	611783
Å	1550	Y	3840	Ā	1.05	VC-11	Before Bag Filters	P6:-39(]:10	68 768
,	1470	Ā	0144	Ā	32.5	VC:JV	Statiff ged stoisd	\$6*W#1 -8	SSIEN
Å	1200	, Y	4430	Ā	33.5	VC:OV	Statiff Suc State	\$6*PF(*61	860FN
, A	1390	Y	0757	Ä	9.44	V7:3V	areally ged eroied	\$6* 49[*0 2	£96\$
,	1430	Ā	4010	Å	6.92	VZOV	Before Bag Fillers	23-1411-92	2620
Į.	09ET	Y	3210	Ä	9.62	Virily	Scione Bog Piliters	56-1884-98	25.65

. .

Table C-21. Pilot-Scale System Run 1. (4 Sheets)

, A	T.TZ	, ,	0461	N	3.59	2C-03B	mauffill omiD bes I	SE-49-1-91 69 LE
N	E.TZ	,	0591	N	4.81	1 (0° 0)\$	mouffit onf bes.	56-9 4-1 -51 991.16
N	2.65	Ĭ.	7000	N	86.€	17.5	stemptic outpoints	(C) (C) (C) (C) (C) (C)
N	çç	Ä	0861	N	9 <i>L</i> .2	11701-035	Battley (Bill pro	77595F20 F2 F3
N	6 \$	Y	848	N	2.39	(10°) (S	ALERTING CHILD PORT	36 49-190 39013
N	9.8 ₇	Ā	3440	N	Lt*t	F7(9-3)\$	Biolity's Chip (per	9745-000 S0S
N	85	, A	0611	N	2.26	410.725	Elenigia (XIII), Pro-	\$6,045-100 \$90.15
N	7 .18	, ,	1150	N	12.2	19 (9° 9)	Biolitics (Billy) Des	(98)(S
N	2.88	Y	946	N	2.23		Strauffici (Mail) bare.	GGSEFT (V.) CANIS
Ä	797	Ä	L£6	N	2.48	186970330	Bracity's (Billio Pass	(2) (22.6)
N	L'09	Ä	દદદ	N	2.2		mealfill intiff the	G6-08/9/2 G60/G
N	4.62	Ä	7.86	N	3.29	100	Patitys (Blig Mar)	\$500055183 (09 7 50)
A	1310	Ä	3910	N	LL.4		BARTON GELOVERS	35 G-25 G
,	1450	VΝ	NDV	N	£7.£	#1355.035V	PERMIT PROPERTY.	Control of the Control
X	1430	Å	3730	N	2.18	1(1)0000	B-BISS OF STREET	(6.453-6) (C) (C
, ,	1380	, X	0614	N	74.2		BOOK CIRCLES SECTION	Contracted Project
Y	1760	Y	0911	N	82.2	(GS::DS	Bone Char Effluen	C6-494-50 D5D19
Ā	1450	Y	0607	N	2.13	0.000	Bedliff Bris area	2,646-0,720 (0,746-0-1).
X	1480	Y	7920	N	2,64	115500)	ROBERT AREA SHOE	\$6.480 (i) \$40.05
A	0591	X	0617	N	2.89	0.000000	mouther send sand	Charle C 19910
, ,	1400	Ā	3800	N	19.2	165000	SOR CHE LITTER	56-001-01 91:01S
A	1480	Ä	3280	N	£.8I	689-38	month, ratio and	\$1022 \$6.0m-95
, ,	1440	X	4030	X	2.48	95000	stratilia mile steel	79.005-62 72.00
, A	1320	Y	0742	Ĭ.	3.47	1 (menticle and a care.	C9-08-0-4-2 24-0-05
, ,	1300	, X	3800	N	2.18	930'08	stadin rato atol	Secretiz ISSS
Ā	1240	, X	3320	N	2.26	(1899)	menth anthern	(6-28/102 196S
, X	1210	Å	3100	N	10	165.05	resettin mich snot	souped soupe
, X	1140	λ	7000	N	10	859:35	seeutha and and	Charles STORY
, X	072	, ,	0661	N	50	ELSCOX	Rose Cast Lilbert	recal·ii 0K60
N	4.82	, ,	3.65	N	\$1.2	÷07	meetherant and	96/35/10 665E
Ţ	LL6	, X	0688	, ,	21.8	0.09	erang day rang	16 CES 10 Sec 28
Å	1410	Ä	3040	, ,	81	7////	eretiri gazi exoleti	\$649-1-00 64015
, X	1450	Y	0901	, ,	L'91	VC:DV	STAILT REIT STOLDE	\$699-170 99018
Å	1360	, ,	3620	, ,	9.81	YC:3Y	ershiri gazi ərufədi	\$1043 014-49-62
, ,	06 † I	, ,	0724	<u> </u>	£.81	VC:OV	TIME IN COM	SOUTHIE OHOIS
Д	1450	, A	4390	X_	£.91	V((1))	metric fag Titlen	\$649.00c (CD)\$
, ,	1460	, ,	4370	Į,	22.6	V70.1V	STOLE BRE STOLES	\$6:000 Z \$50.68
Å	0781	Å	34.9	Å	9.42	Yes	3506 (45 (20)5)	
pang tij	cióas	panet 15	(2/204) 45	mand st	(1/00) 14	unu	HOLLY307	SLIVE I APP

DOE/RL-95-59 Rev. 0

Table C-21. Pilot-Scale System Run 1. (4 Sheets)

N	8.82	Ā	1200	N	92.9	889-06	MONITED DANS DANSEN	CO-YEAR CO	89915
N	€.09	Ā	1360	N	2.36	889-05	THROUGH DAY DON'T	50:3939720	39.53
N	6.72	X	1280	N	70.T	86-58	Alixad Bood Billinein	\$6:4970010	70.88
N	9.68	Å	3090	N	79.2	2C:98B	MANUAL DAG DAXIA	56-49-1-82	63.9(8)
N	2.82	, X	1250	N	2.19	889-36	Mixed Bod Elithera	33-1-69-02	9,938
N	1.94	Å	1380	N	T.2	20:98B	BERNESS DAG DOXUM	56-98-172	1000
N	2.85	Å	1310	N	12.5	3C-68B	Mixed Bod Eigheit	\$65021537	2221.8
N	9.85	Ā	1790	N	9,5	2C-98B	Mixed Bed Millions	V6=041577	0.536
N	p.TZ	Å	1120	N	2.3	889-38	Manffill bed beatM	\$6* 4# 1*17	19118
N	9.65	Ā	482	N	3.43	800.00	mentite text begins	56-99d-LT	72.116
N	0	, ,	1270	N	4.39	(1)	Statiffië bed bealia	Sorte-191	ars.
N	1.62	, I	\$66	N	6.65	1889: ON	mantity too besile	\$6:99£I:\$T	29.116
N	p.TZ	, ,	1240	N	2.54	889-36	CONTRACTOR POLICY	10-1 4-1 -01	160 (5
N	2.85	, ,	0704	N	17.5	339.08	HORIUS DE PARIA	56-994-60	192115
N	£.82	, ,	1480	N	3.42	689-06	trouffid bod beatle	\$6-994-90	251152
N	5.29	, A	3.9£	N	34.5	\$100	mentiti bed boxiv	parties III	10166
N	4.72	λ	2140	N	11.4	H(G-9)6	tradfill only bead	561 # [4 *]	62015
N	8.88	, A	2020	N	2.53	2C-033	tread Cimo Efficient	\$6-1414-91	924 S
N	£.82	Å	2210	N	29.2	8179 98	mouffiti omiO bea.i	S6-PM-SI	21433
N	2.62	Å	2210	N	3.32	2C-95B	mentital outil bead	S6-PIN-PI	02718
N	4.62	Ä	1980	N	71.5	6729-38	treatflid only best	C6-1814 C1	21493
N	8.09	Ā	2140	N	79.2	97.90	mealfth outil) best.	S6-PINGI	19915
N	9.72	X	7370	N	25.2	0.655.05	RIGHTLES (AUE) (Rear)	C6-85M-90	73)215
N	8.72	Ā	2300	N	9.9	179 05	manifig only beat	S6-PEIN-60	19716
X.	6.68	, ,	0791	N	94.4	469.08	gauffe mile incel	18: PARISO	98938
N	9.52	X	122	N	89.2	97.9 OS	Beatific (name the	70 mM-70	63715
N	T.82	, ,	0981	N	4.14	679° X	maufficient Cline	(6 - 10 34 (9)	0.52-15
N	19	, ,	7160	N	19.8	2C 978	mouffel omil's best	Se Borio	4771
N	1.72	<u> </u>	7000	N	IL'S	879-38	trauffill onlin beat	C2-MM-50	
N	6.68	<u>, Y</u>	0161	N	L1.9	270108	head Clino Effinent	56 1893-10	1701
N	2.18	, ,	7300	N	8T.2	879738	mauffill onlin teal	58-44-1-87	
N	6'55	, ,	2040	N	2.55	2C-93B	manith onic beal	22-6-62	87.54 (S)
N	6.82	- X	7070	N	87.£	079-08	mauffill onffO best	24-14-65	
N	T.T2	- X	0961	N	2.31	309 OS	monthly ond Dead	23-Feb-95	
N	9.88	X.	1840	N	2.11	3C:45B	Insufficient Diss.	\$64545X	777715
N	65	Y	0081	N	2.69	823.00	mential onito bear	26-99-112	
N	1.72	Y	1740	N	16.2	407.35		\$6.00	
EARCE (S)	(m)® ro	pape as	(2/2)4)-5	m Descr	(1/0)() 14	2304	X021Y307	MAG	7000

Table

C-21

Pilot-Scale System Run

(4 Sheets)

Υ

CLOCKL Ca Derret Se Deleti Pa (ocida) Pu Desect Sr (aCL/L) 200 LOUATION 2027 No Mar 95 Mixed Red Eiffrent SE-68B Y N N 60 \$10.60 3.5 1150 Y 57.7 Ν 07.43 at 95 Mixed Beil Efflüch SC-688 N 3.03 1140 11.67 083 Mais 95 Mixed Bed Liftuent S (6.68) N Y 55.8 N 3.6 1360 Y N 9 (4.0) opakina ek Mixed Bell Efflueni SC 688 Ń 1600 58.6 3.8 10-Mar-95 N Y 58.1 N Mixed Red Eitherst 4.05 1540 Y 91468 13-Mar-95 Ν 1320 60 Ν Mixed Beil Efficien SC-68B 2.84 **SS (1977)** 14 Mar 95 SC-683 Y 56.2 N N Mixed Red Effluent 3.83 1540 15-Map.95 S(0.58) 2.25 N 1250 Y 57.1 N 91474 Mixed Beil Efflueni Y Ν 16 Marays Mixed Bea Eithers 2.35 N 60.4 SC-68B 1640 N. ... \$100,631 N Y N 17-Mac-95 56.4 Mixed Bea Effluers 33.7 1660 \$1481 17-Mar-95 SC-68B N 1660 Y 58.6 N Mixed Bed Effluent 4.09 **134**17 07-Nov-94 GAC Efficient SC-6 N Y 608 N 40.1 10 F9420 29 Nov-94 GAC Efficia Y 59.1 N 10 Ν 15.9 E924 01-Dec-94 GAC Efficien N 21.9 N 59 N 20 Naixy SC-6 N Y 55.9 Ν 18 Jan 95 GAC Efficient 10 10.5 \$6.46 Y Ν NADAO 9-fan-95 GAC Efficient 10 N 76.7 59.8 \$949 N 11.1 Y 61 N 20-Jan-95 SC-6 GAC Efficient 3.16 3(0.59) GAC Efficient SC-6 N 10.5 Y 60.2 N 23-Jan-95 2.81 SC-6 Y Y -5955 24-Jan-95 **GAC Efficit** 2.68 Ν 40.8 60.7 Y \$1000 25-Jan-95 GAC Efficient SC-6 2.42 Ν 94.3 61.1 N **S** 48.1 Y N \$1023 26-Jan-95 Ν 0 GAC Elficen 3.24 S&-6 N 151 Y 63 N S107.6 27-Jan-95 **GAC Effluent** 2.63 Pu Detect Sr (pCi/L) Sr Detect Cs (pCi/L) LOCATION Pu (pCl/L) 5/8/8 DATE PORT Cs Detect (19.8Van.95 AFTER TANK 4 SC-62B 1.6 Y 2300 Y 11 09-Mat-95 AFTER TANK 4 Υ DUP SC-62B Y 1.5 2200 4.8 (pCt/p) (pCi/p) (pCUp) Lead Clino Column Y Y 7810 Y N5063 21-Mar-95 Resin 1.69 6820 Y Y 8460 Y DUP 21-Маг-95 Lead Clino Column Resin 1.97 8150 23-Mar-95 Y Y N5704 Mixed Bed Column 10.5 3390 Y 5.53 Resin

13.3

Y

3200

Y

6.44

DUP

23-Mar-95

Mixed Bed Column

Resin

Table C-22. Pilot-Scale System Run 2. (4 Sheets)

HEIS #	ENS	DATE	LOCATION	PORT	Pu (pCIA)	Pri Detect	Sr (pCIA)	Sr Detect	сь (рсіль	Cs Deper
BODX07	\$1607	1-Apr-95	Extraction Well	AC-I	22.4	Y	4610	Y	1380	Y
BODXG8	\$1608	11-Apr-95	Extraction Well	AC-	19.3	Y	4740	Y	1440	Y
BODXH3	81613	13-Apr-95	Extraction Well	AC-I	18.2	Y	4890	Y	1450	Ÿ
BODXH6	S1616	17-Apr-95	Extraction Well	AC-I	12.2	Y	4100	Y	1400	Y
BODXII	51621	19-Apr-95	Extraction Well	AC-I	19.3	Y	4820	Y	1530	Y
BOFBW0	S1913	21-Apr-95	Extraction Well	AC-I	21.5	Y	5380	Y	1490	Y
BOFBN7	N5160	11-Apr-95	Extraction Well	9C-55B				†		
BOFENS	Natel	11-Apr-95	Extraction Well	SC-62B						
BOYENS	35165	11-Apr-95	Extraction Well	SC-68B						
BOFBWS	31918	242Apr-95	Extraction Well	AC-I		1	5180	Y	1620	Y
9:(9:93) 9/3	83.924	26-Apr 95	Extraction Well	A(6-1	22.4	Y	5690	Y	1600	Y
100001974	31/27	28-Apr-93	Estruction (C)	AC	21.1	Y	5440	Y	1560	Y
BOFFER	21031	1-May-95	Extraction Well	AC-1	20.2	Y	4930	Y	1480	Y
13011572	3 (2.12)	3-May-95	Extraction Well	ACC	22.7	Y	5260	Y	1760	Y
EODBYG	5) 0) 0	3-May-95	Extraction Well	AC-1	18.8	Y	5340	Y	1420	Y
BOFFE	\$2224	8/May-95	Extraction Well	ACH	18.6	Y	5040	Y	1630	Y
BOFJHI	52242	10-May-95	Extraction Well	AC-I	21.7	Y	4440	Y	1570	Y
BOPJES	\$2,246	12-May-95	Extraction Well	AC-I	15.7	Y	5250	Y	1530	Y
BOFIR	8225	15-May-95	Extraction Well	AC-I	18.4	Y	4710	Y	147	Y
ВОРЛЯ	\$2253	17-May-95	Extraction Well	AC-I	21.3	Ÿ	5210	Y	1540	Y
BOF118	\$2259	19-May-95	Extraction Well	AC-	21.8	Y	4980	Y	1530	Y
BOFIES	\$2264	22-May-95	Extraction Well	AC-I		NA	6410	Y	1820	Y
BOURT	852268	23-May-95	Extraction Well	AC-I	21	Y	4160	Y	1510	Y
BOFM47	\$2503	24-May-95	Extraction Well	AC-I	21.1	Y	4140	Y	1420	Y
BOFMSI	52537	25-May-95	Extraction Well	AC-1	19.9	Y	4130	Y	1360	Y
BOFMSS	\$2941	26-May-95	Extraction Well	AC-I			3600	Y	1310	Y .
BODXET	31617	17-Apr-95	Beiture Beg Filters	AC-2A	11.2	Y	4380	Y	626	Y
BOFBV7	S1910	197Apr-95	Before Bag Filters	ACOA			4790	Y	1360	Y
BORBW	300	21-619-95	Betwee Day Filters	AC-2A	21.8	Y	5130	Υ	1430	Y
BOHEW2		4/4/2025	Before Bag Filters	AC-2A	23.8	Y	5380	Y	1380	N
BODDW6	Siele	24-April 93	"Hefore Pay Filters	AC-2A			4840	Y	1440	Y
9000X60	9 (609	11-A12-95	After Bay Filters	ACH	13.1	Y	3840	Y	1000	Y
BOOKES	5)(5)(8	17-App.95	After Big Filters	3.03	14.7	Y	4290	Y	280	Y
воквхо	91923	26.Apr-95	After Bag Filters	AC-3			4810	Y	1390	Y

Table C-22. Pilot-Scale System Run 2. (4 Sheets)

HEEL	EN	DATE	LOCATION	PORT	Pa (pCl/L)	Pu Detect	Si (pCIA)	Sr Detect	G (pCIZ)	Cr Detect
10,000,000	\$31926	220000000000000000000000000000000000000	After Bag Fifters	AC-3			4950	Y	1460	Y
BODEXS	\$1928	28-Apr-95	After Bag Filters	AC-3			4880	Y	1360	Y
BOFEXS	S1932	01-May-95	After Bag Filters	AC-3			5300	Y	1470	Y
BOFBY3	\$1936	03-May-95	After Plag Filiters	AC-3			4970	Y	1430	Y
BOFA()	82234	5-May/95	After Bag Filters	AC-3		,	4550	Y	1500	Y
POPJG7	S2238	8-N44y-95	After Bag Filters	AC-1			4800	Y		
BOF/H2	32243	10-May-95	After Bag Filters	AC-3			5030	Y		
13.4)6.30(6)	\$2247	12-May-95	After Bay Fisters	74,655			4720	Y	1590	Y
	372212	Bedoubyed)	After Car Piters	AC-3			5060	Y	7.0	
9.000	Selection	17000000	After Day Filters	A C C			5070	Y		
BOOK	53.00	Salvaya)	Aller Bag Elbert	AC 3			5100	Y	1500	Y
0.007/50	\$22,65	22-May-93	Aller Brillier	4.00			4670	Y	1720	Y
(2) (7) (7) (8)	52538	2558/66595	After the fillion	AC-3			3990	Y		
GOTM 6	\$2.442	26-May 95	After Basy States	ACS			3740	Y	1300	Y
B(M)XE()	\$1618	11-Apr-93	Boxe Cher Efficient	SE-558			3790	Y		
ROPAS 7	\$2548	26-May 95	Bone Cher Efficient	Sessi			3660	Y	1290	Y
RODXIII	\$1611°	11-Apr-93	Lend Clino Efficient	SE-62B	2.56	N	56	Y	59.1	Ň
SODXHA	S1614	13-Apr-95	Lead Cline Efficient	\$6423	6.47	N	53.2	Y	59.9	N
BODXES.	31619	17-Apr-91	Lead Clino Efficient	96.628	4.07	N	136	Y	59	N
8000.78	899.1	19-Apx-95	Lead Clino Efficient	\$656212			289	Y		
(31916	21-Apr-95	Land Cityo Officers	90,628	6.38	N	440	Y	60.9	N
ecessura.	31920	24 Apr-95	Lead Clino Efficient	\$0.528			298	Y	58	N
BOURNE	31924	26-Apr-95	Lead Clino Efflich	(30.624)			387	Y	60.2	N
HOMEN	\$12.2	28943235	J. east Clinic Ulfraen	SC-62B			631	Y	58.2	N
9.700	SUZE	01-May-95	Last Cline office;	(10-(723)			537	Y	57.1	N
	31777	03-May 91	Leas Cincollifform	Sec. (2)			641	Y		
100000	2.35	3.00(0.00)	Leat ChropPiliten	88(6,5718)	4.12	N	828	Y	58.7	N
0.2.2.3.58	222.34	800600	Lead Class Efficien	N. 800 (\$1)	5.54	N	785	Y	59.4	N
	X.	0.0000000		322(9)			707	Y		
		1035505	Alexa Charles and a	30.62			773	Y		
		1200000	Lead Cline of Regal co	96-628		<u> </u>	1100	Y		·
1,012,101.58	39223	P. Aviny of P.	Lead Cline Efficat	\$36.6211			964	Y		
30(4)	100	B925925	Lead Class Efficient	9C-62B			940	Y		
	32257	17-May-95	Lead Clare Efficien	SC-62B		7	934	Y		

Table C-22. Pilot-Scale System Run 2. (4 Sheets)

, ,	9L'9 0SSI	, , ,	787 0694	Ä	1.25	HEQ. 3S	(resiftford) beat	Se-way res		Marie (e. 1
<u> </u>	1210	I A	0144	I A	16.9	100	levé-noinceurà	12.00		
		7.	0177	Α			[Streeted Act]		66 I SN	
		Å	0911	1	12.012.01			**		
-		Y	1080			889-05	modelly one threed	46-40/4-67	11110	
<u> </u>		\ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \	186	 		689-06	treatility or pl. breader	Section 1	035525	10.12
	 	I I	11/2		 -	9C-68B	months onto brecot	4000000	25.65.0	031/14(0)
N	95		689	 		H89-00	Bressifik (ann a beneve)	\$65.4565	43.00	990E03
	 	I I	134			889-36	machili coil 3 breeds		1985	EXIGES.
		\ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \	<u> </u>	 	 	H89-OS	Manufaction of the Second Cines (Silbury)	36/49/3	£97.58	BORNS
		, A	128	 	- 	889:06	Second Clinic Billiams Second Clinic Billiams	46 AB) Y-21	85765	1210000
N	8.09	I A	128	-	 	E89-OS	MINDER ONE DESCRIPTION	26 (884 C	PSZZS	STROET
		, ,	101	 		889-06	Second City of the Second	S6-ANA-OI	05275	SHI HOE
		, ,	5.28			689-08	Machina chira brease?	56 49A-9	9775	9:136 4
		Ä	901			689:06	MOUTH ON IO brook	CO-ARIA-C	097278 45778	69/408
		, X	£.73	-		E89-05	manfile only based	\$6 APPA-10	86615	906-100
	1	, ,	9 LS	<u> </u>	 	889.08	Second Cine Rithers	\$6-4 8 94-10	#661S	53,536061
		, ,	5.65	1	<u> </u>	H89-00	Second Chate Milliam	SK-KIV-81	06615	EXBECKE EXCEPTED
N	€.62	, ,	LIS		<u> </u>	889-06	Material orest beacods	56344-07	2 030 2 452	
		Y	LS		1	889-OS	manific one breson	76-200-7	176 8	EXELECTE BARBIOS
N	9.72	, ,	L'LÞ	,	10.4	889-26	headfith that I become	EGALV-17	2161S	**************************************
		, ,	29.3			H89-DV	Mental Child Sifteen	\$6-XIV-61	81618	741 23 63 6 67 5 6 6 7 8
N	6.82	X	9 8 <i>L</i>	N	3.2	889-05	Insufficient Dinocock	\$6-449-2	21620	07X(103E
N	19	X	87	Å	\$9.T	E189-OS	Manifile onlic tenesori	26-KIV-E1	91916	\$35000
N	9\$, ,	9.84		1	869-36	Impelificant Denoted	66-44V-[]	31915	CHAC OH
N	9.09	Ā	2880		<u> </u>	2C-03B	treat Distant Descri	10 ABA+01	Prices	SCIVLOR
		λ	2710			9C9-D6	igentifizi onni 2 bees.	\$6-090(67)	68578	ESPERCH
N	6.69	, ,	7390	N	29.8	80°08	Interest Colors of States	SC APPAY Y	0000	7.67 C C 11
N	2.09	, ,	2240	N	68.6	979-06	HAD DEVENOUS CHES	369000000	6.516	305 (CH184)
	ļ	Å	7060			3100 ox	pengga suga pera	9909.600367	11642	#130 E G
N	2.82	, ,	2230			8769-96	Manigis ongs pass	(400 W) 517		
N	85	Ā	288	N	SÞ.Þ	HEROX.	Testing colin peri	skelety at	10/14/5	Vic.
N	9. <i>T</i> 2	, ,	866	N	₹9.£	879-36	Hanitis ones peed	Signation is	09777	
Expert 13	and o	Sy Delect	(1/10/0) 45	hu Debet	(7/100) 14	1804	NOLLKJOT	atra	FUE	e staji

Table C-22. Pilot-Scale System Run 2. (4 Sheets)

				00	1130	uteall	canado Ontil Derose 8	CG-unr-L	ZÞSSN	BORNAS
				00	2480	Gesin	amulo0 onliO best	CE-mth-T	IFSSN	16N-108
				(8359) 4	latek latel					
λ	159	Y	9.02	Å	8.01	nieski	mulio OAO	č%-mil-1	DESA	76,607
Ä	L0 1	Y	7300	,	613	Keatt	Boxe Char Column	\$6-aut-8	ERSON	LENGE
	(37,54)		(37.70)		(2),5(0)					
N	SL'Þ	Y	178	Y	7 4.0	6170-035	incelled and beau	967499966		
λ	1370	, A	0544	λ	18	I-OV	list munstra	56-44N-61		CC1:(CE
Cobact	(VDP)	१३माग्स ४५	(245A) is	Pu Detect	CANDO M	DIDA	NOLIVXYI	airg	ENS	# STEEL

Table C-23. BY-Database (Picocuries). (4 Sheets)

College Considerate and Consid			B0000000000000000000000000000000000000	5 30800C		C 00000	
31.15	Eff or N#	Doze	Location	8	Co-60 (pCl/L)	8	To-99 (pCI/L)
SPIDITION.	\$1030	26-Jan-95	Before Filters		95.5	Π	4250
* BODR T	\$ 51031	26-Jan-95	After Filters		92.1		6600
HODRCS	%S1032	26-Jan-95	After 1st Column	<	40.7		118
BODROG	21055	31 -Jan-95	After 1st Column	<	43.4	<	77.2
EGDRD1	91056	31-Jan-95	After 2nd Column	<	38.2		1220
BODROZ	\$ \$1057	31-Jan-95	After GAC	<	38.2	<	84.5
NAME OF THE OWNER OF THE OWNER OF THE OWNER OF THE OWNER OF THE OWNER OF THE OWNER OF THE OWNER OF THE OWNER O							-
BUDAD3	\$1058	01-Peb-95	Before Filters		92.2	L	5470
BODRD4	\$1859	01-Feb-95	After Filters	Ц	100	L	5090
BODROS	\$ 21060	01-Peb-95	After 1st Column	\leq	40.7	<	60.9
BUDRED6		22.5					
EOORD7	S1061	02-Pet-05	After Filters	<	57.6		4710
BUDKUS	\$1062	02-Feb-95	After 1st Column	\leq	40.3	<	72.5
202000	S1963	02-Feb-95	After 2nd Column	\leq	38.9	<	65.7
BODROS	\$1064	02.53.65		\sqcup			
BODRE	\$1065	03- Pet- 95	After Filters	<	56.3		4720
EOUR	S1066	03-Feb-95 03-Feb-95	After 1st Column	\leq	36.6	<	67.9
	Siece	US-18-0-93	Before Filters	\dashv	133		5110
BODRE 2	5.1067	06-Feb-95		1			·
BODRES	\$ 1068	06-Feb-95	After 1st Column	<u> </u>	39.2	<	78.4
BODRE 4	\$ 10.69	06-796-05	After 2nd Column	4	40.3	\leq	77.8
	0.1003	30-7-50-93	After GAC		40.7	<	80.4
BODRES	\$ 1070	07-Feb-95	70.6 - 50	\vdash	1.50	-	
BODRF 6	\$ 1071	07.Feb.95	Before Filters	\vdash	158	\vdash	6180
BODRE 7	\$ 1072	07-Feb-05	After Filters		124		5910
			After 1st Column	\leq	42.1	<	79.4
BODRES	\$ 1073	08-Peb-95	After Filters	+	111	H	
HODRES	\$ 1074	08-Feb.95	After 1st Column	<	111 37.8	4	6770
BODRGO	\$ 1075	08-Feb-05	After 2nd Column	7	38.9	<u> </u>	82.6
			2000 200 00000	\rightarrow	30.9	<	82.3
BODRG 1	S 1175	09-Pet-95	Before Filiers	+	136	-+	9960
BODRG 2	\$ 1176	09-Feb-95	After Filters	-	157	+	8860
B(8)P)R(9)B	\$ 1177	09-Feb-95	After 1st Column	<	37.8	7	7030
			- Carrianili	\dashv	31.0	\dashv	134
\$(0)0)000	\$ 1178	IG-Pet-98	After Ist Column	<	39.2	<	86.8
BODRGS	Su 17/9	10-Feb-95	After 2nd Column	र ो	34.6	3	91.2
5(6)4)3(5 6	\$1139	10-Peb-95	90000.000000000000000000000000000000000	<	39.3	ो	86.4
				\top		+	
BODRG 7	\$ 181	15-Feb-95	Before Filters	\dashv	142	1	10100
BODRG 8	51182	15-Feb.95	After Filters		146		9230
BODROS	\$ 1183	1606668	After 1st Column			\dashv	317
						\neg	
BODRHO	Sales	17-Peb-95	After Filters	$oxed{oxed}$	172		9770
BOBRAI	51185	17-Feb-95		<u> </u>	40.7		211
BODEH 2	\$ 1186	17-Pet-95	Atter 2nd Column	<	41.7		352
(COD)(A) (C)	\$1187	21-Fet-95		<u><</u>		<	93.6
BOURH 4	S 1188	21-Feb-95	After 2nd Column	<	36.6	<	91.2

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Table C-23. BY-Database (Picocuries). (4 Sheets)

HEIS	Eff or Ng	Date	Location	B	Co-60 (pCt/L)		To 99 (pCI/L)
BODRH 5	\$ 1189	21-Feb-95	After GAC	-0		ess m	a eccession representation representation to
			ANGLUAC	<	38.8	- ≤	89.4
BODRES	\$ 1190	22-Pet-05	Before Filters	-	147	-	
BODRE 7	5 1191	22.Feb.95	After Filiers	8	147 183	+	9280
BODRH 8	\$ 1192	22-Pep-05	After 1st Column	8			8310
			ASSET ISLECTION		37.7	_ <	92.4
BODRES	\$1.00	23-Peb-95	After Filters	+	152	-	
BODRIO	5 1 94	23.Feb.95	After 1st Column		173		9860
BODRJ 1	\$ 1195	23-Ret-95	After 2nd Column	<	42.7	_ <	+
			ASSECT AND COMMEN		39.6	┨—	113
ECDR2 2	\$ 1279	24-Peb-95	After 1st Column		25.4	┰	
BOERI 3	\$ 1286	24-Feb-95	After 2nd Column		35.4		252
BODRJ 4	\$ 1281	24-Feb-05	After OAC	\	39.7	-	228
			ALGI UAC	1	38.9	-	105
BODRI 5	\$ 1282	27- 101 -95	7.6		107	4	
BODRI 6	\$10.57	27.Feb.95	Before Filters	-	187	┺	10200
BODKI 7	\$ 1294	27-7-80-95	After Filters After Ist Column	-	126		9980
			NAME INCLUDING	<	37.4		196
ECDE 8	\$ 1285	Gl-Mar-95	10	\vdash			
HODRI O	\$ 1286	01-Mar-95	After Filters		170	Ш	9520
BODEKO	\$ 1287	GI-Mar-95	After 1st Column	\leq	41.8	<	89.1
		3.55.00	After 2nd Column	<	41		160
BODRK	\$ 1288	02-Mar-95		- 			
RODRE 2	5 1289	02-Mar-95	After 1st Column	<	35.8	<	84.4
BODHER	\$ 1290	02 Mars95	After 2nd Column	4	39.9	<	80.2
			After GAC	<	41.7	<	78.5
BODRK 4	\$ 1291	03-Mar-95	D. C 271	-		\sqcup	
HO) DRIES	\$1292	03-Map.05	Before Filters	_	166	\sqcup	9930
BODRK 6	\$ 1293	63-Mar-95	After Hiters	_	186	┦╌┤	9640
		Co. Marco	After 1st Column	4	43.1	\mathbf{I}	<u> 177</u>
BODXJ 2	\$ 1359	G6-Mar-95	10 - 200			\bot	
(E) (E) (A) (C) (E)	\$ 1369	06-Mar-95	After Filters	+	169	\sqcup	9310
15(0)0):6(0)	S 1360	(65,842,435	CONTROL OF THE PROPERTY OF THE	<u> </u>	33.4	<	82.6
		GOSSITATO	Aute zie Calumn	<u> </u>	35.8	<	87.2
3000335	\$ 1361	08-Mar-95		- -			
BODXI 6	\$ 1362	08-Mar-95	COSP0009000000000000000000000000000000000	<u>< </u>	38.5	\leq	79.4
BODXI 7	\$ 1363	08 Mar 95	20000000000000000000000000000000000000	<u> </u>	40	\leq	85
			After GAC	<	37	<	77.4999
BODXIE	\$ 1364	10-Mar-95	Before Filters	+	102	\sqcup	
BODXIG	\$ 1365	10-Mar-95	900/90/00/0000000000000000000000000000		185	-	10900
BODXKO	\$ 1366	10-Mar-95	After Filters	_	176	$\vdash \downarrow$	10700
			After 1st Column	<u> </u>	41.4	\leq	81
BODXKI	\$ 1367	13-Mar-95	After Filters	4	167	$\vdash \downarrow$	
BODXK 2	S 1368	13-Mar-95	99900000000000000000000000000000000000	-	167	 -	9980
*BODXK 3	\$ 1369	en anaccoccoccoccoccoccoccoccocc	TOPOGOGO CONTROLO CON	<	42.4	$\vdash \vdash$	269
		- armstill and	arance was CORUMA	<	35	-	442
BODXK 4	\$ 1370	15-Mar-95	15.0	_		Щ.	
DODXK S	5 1371	NY TORONO TORONO CONTRACTOR AND ADDRESS OF THE PARTY AND ADDRESS OF THE		<	39.6	\sqcup	149
BODXK 6	\$ 1372	L5-Man-95	000000000000000000000000000000000000000	<u> </u>	38.5	<u>.</u>	211
			Aller UAC	_	39.2		183

Table C-23. BY-Database (Picocuries). (4 Sheets)

	St. or N#	Date	Location		Co-60 (pCUL)		To 90 OFFIE
THE PARTY OF	SEC.	17-Mar-95	Before Fillers		147		10600
	3.374	17-Mar-95	After Fillers		150	П	10700
11.010 7.216 7.41	S1818	17 (Marcy)	After 1st Column		36.6	П	112
	\$1669	24-Mai:95	After Filters		187		10900
	\$ 5.1670	24-Mar-95	After Ist Column	<	40	<	67.2
201/25/53	\$ 1671	24-Mar-95	After 2nd Column	<	40.6	<	67.7
\$2(0)P\(0)	00001672	28-Mar-95	afiei Isi Commi	<	37		131
300X 7	\$ 1573	28-Mar-95	After 2nd Column	<	37.8		116
BUDXLS	\$ 1674	28-Mar-95	After GAC	<	37		102
BODXLA		30-Man-95	Before Filters		169	П	11200
SEDXXX	52.6	30-Mar-95	After Fillers		163		10100
BODAM	8.00	30-Mar-95	After 1st Column	<	36.6		144
BODXNES	\$ 50.678	GS-Apri-95	After Friters		157	П	9920
BODXE	\$ 1679	05-Apr-95	After 1st Column	<	36.6		206
HOBXMSE	\$1,680	OS-Apr-95	After 2nd Column	<	37.8		200
B(OD)XM T	\$1.60	07-Apr-95	After ist Column	<	41	<	87.7
BODXW 6	5 1682	07-Apr-95	After 2nd Column	<	39.3	<	89.5
POD XM 7	\$ 1683	07-Apr-95	After GAC	<	42.4	<	90.3
\$1(6)B),QM(4)	\$11684	10-Apr-95	Before Filters		145		10500
6(0)00000000000000000000000000000000000	S 1685	10-Apr-95	After Filters		165		9320
RODXING	\$ 1686	10-Apr-95	After is Column	<	39.2	<	146
SKODEN IS	\$1687	12-Apr-95	After Filters		170		10700
5[8]8)(4(4)(2)	5 2 68 8	12-Apr-95	After 1st Column	<	41.4	7	160
(00)	\$ 1629	12-Apr-95	After 2nd Column	<	42.7	٧	158
(10)028000	0)	14-Agr-95	After 1st Column	<	38.9		123
02/01/93/03/55	##S1859	14-Apr-95	After 2nd Coloren	<	44.1	Y	96.9
300000	S 1860	14-App-95	After GAC	\leq	38.2	<	106
35(0)0)313	\$1861	17-Am-95	After ist Column	<	34.2		143
	\$ 1662	17-Apr-95	After 2nd Column	<	39.3		817
11187878181	SS 503	17-Apr-95	After GAC	\leq	41.1		392
	HES 1874	10.1 5-		\sqcup		\sqcup	
(10)B) (1)	5030.574 5050875	19-Apr-95	Before Filters	$\vdash \vdash$	169	\sqcup	10800
36000X4V2	88 S 1876	19-Apr-95 19-Apr-95	After Filters	H	172		9630
	Veto10	13-MH-52	After 1st Column	<	41.1	1	271
1000	S2877	21-Apr95		$\vdash \vdash$		-	10000
N1(0)B)(0)	3 .01 885 878	21-Apr-95	Before Filters	-	161	-	10800
		21-Apr-95	After 1st Column After 2nd Column	<	43.7	+	162
			AMEL AIRI CAMINI	\vdash	40	4	83.8
	S RED	24-Apr-95	After Ist Column	<	36.6	\dashv	175
		24-Apr-95	After 2nd Column	7	40	7	175 162
The second secon				_		_	102

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Table C-23. BY-Database (pCi). (4 Sheets)

HELS	E# or N#	Date	Lacation	Į.	Co-60 InCI/LI		To 99 (pCVL)
\$1000X48.#	S 1882	24-Apr-95	After GAC	<	45.3	<	170
						Ħ	
BOD XP6	\$ 1883	26-Apr-95	Before Filters		157	\Box	8910
BODXQO	S 1884	26-Apr-95	After Filters		166	1	9030
BODXQL	\$ 1885	26-Apx-95	After 1st Commo	<	36.6		188
						П	
BOFGB2	S2000	28-App-95	After Filters		178		10000
BOPGB3	S 2001	28-Apr-95	Alter 1st Column	<	38.5	<	166
BOFGB4	\$ 2002	28-Apr-95	After 2nd Column	<	38.1	<	138
BOFGBS	\$ 2003	02-May 95	After 1st Column	<	39.3		190
BOPGB6	52004	02-May-95	After 2nd Column	<	42.7	<	69.9
BOHGB7	\$2005	02-May-95	After GAC	<	36.2	<	69.9
							
BIOFCES	\$ 2006	04-May-95	Before Fillers		202		8130
BOPGES	S 2007	04-May-95	After Filiers		181		11300
BOFGCS	\$ 2008	04-May-95	After Ist Column	<	42.8		271
BOFFECT	\$ 2009	09-May-95	After Filters		163		10800
BORGE2	S 2010	09-May-95	After 1st Column	<	40.4		432
BOFGC3	S 2011	09-May-95	After 2nd Column	<	36.6	<	79
BOFGC4	\$ 2012	11-May-95	After (st Commi	<	41.3	П	346
BORGES	S 2013	11-Map-95	After 2nd Column	<	37	7	80.6
BOFGC6	\$ 2014	11-May-95	After GAC	<	38.5	7	84.3
						П	
BUFFECT	S2015	16-May-95	Sefare Filters		197		9830
BOFGCS	52016	16-May-95	After Filters		150		9760
BOFGCP	\$2017	16-May-95	After Ist Column	<	32	П	700
					-1.1		
BOFGDO	S2018	18-May-95	ESCOTE FILLER		177		6960
BOFGD1	\$2019	18-May-95	After Filters		161	П	8430
80096002	\$2020	18-May-95	After Ist Column	<	43.7		342
BORGES	S2021	18-May-95	After 2nd Column	<	35.4	<	84.8
BORGDA	\$2022	8-May-95	After GAC	<	36.6	<	106
		0	(site Laboratory Resi	ules			
		04-May-95	Before Fillers	İ	168.0		10500
BOERS		0445/39295	Aliet is Column	<	10.5	\dashv	251.0
BOFI-57		18-May-95	After Filters	_	147.0	\dashv	8590
	Spent	Resin Analyse	5		pClig		pCVg
BURNES	Resin	20-May-95	Lead IX Column		8600		47700
EDEVEC	Resin	20-May-95	GAC Column	+	2.12	\dashv	19.4
SERVICE SERVICE	Resin	20-May-95	Polish IX Column	+	1.2	+	1600

Table C-24. Final Resin Sample Anions/Cations 216-B-5 Reverse Well.

Raw Clinoptilolite			Clinoptilolite Spent Resin							
HEIS # :	BOFHG6	BOFHG7	BOF786	BOF821	BOFN93	BOFN94				
DATE:	34816	34816	21-Mar-95	23-Mar-95	9-Jun-95	7-Jun-95				
LOCATION:	As Packaged	Duplicate	Lead Clino Column	Mixed Bed Column	Bone Char Column	GAC Column				
	Units: microgram contaminant per gram resin (ug/g)									
Al	18600	19400	165000	5420	733	501				
As	3.4	7.3	3.4	2.7	2.8	4.2				
Ва	180	179	317	106	133	21.7				
Be Be	1.1	1.3	1.1	0.64	0.9	2				
Ca	11100	12500	19200	107000	237000	1050				
. i ⊂Cl			2.66	5.25						
Cu Cu	1.6	4.8	2.8	11.6	27.8	15.2				
Ta Fe	3420	4070	3840	2120	3540	3080				
π μ				2.11	2.74	2.54				
‡ ₹ Hg			0.08	***	1.1					
K	24000	22500	20500	12600						
- ≠ Mg	2340	2570	2990	3020	3220	169				
Mn	274	340	211	132	57.8	23.2				
` Na Na	20800	20600	2930	2540	1730	93.4				
NO3			0.26	2.07	2.98	7.13				
Pb	22.1	16.1	19.8	14.2	3.7	1.3				
T	0.34	0.36	0.44	0.21						
Vanadium			5.1	15.2	39.4	77.2				
Zn	17.2	21.8	29.2	44.2	1140	16.6				

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Table C-25. Final Resin Sample Anions/Cations 216-BY Cribs.

Dowex 21K Spent Resin							
HE1S # :	BOFMF5	BOFMF6	BOFMF7				
DATE:	24-May-95	24-May-95	24-May-95				
LOCATION:	Lead Column	GAC Column	Polishing Column				
Units: microgram contaminant per gram resin (ug/g)							
Al	369	516	24.2				
As	1.6	5.8	1.2				
Ba	31.6	23	1.2				
Ca	481	1300	564				
a Ca	17.4	16.5	14.3				
" Cn	9780	362	735				
Cr	1.9	3	3.6				
Cu		13.2	2.1				
Fe	6840	3270	659				
Hg	0.5	0.16					
Mn	2.3	18.1	0.84				
Na	184	81.6	69.8				
NO3	43.6	77	36.4				
Pb	1.3	3.2	0.39				
Se	2.2	0.72	3.8				
Zn	1270	59.9	22.3				

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APPENDIX D

WATER-LEVEL MEASUREMENTS

RL95-59,W51/A2 D-1

Figure D-1. 216-B-5 Reverse Well Site Water-Level Monitoring for January.

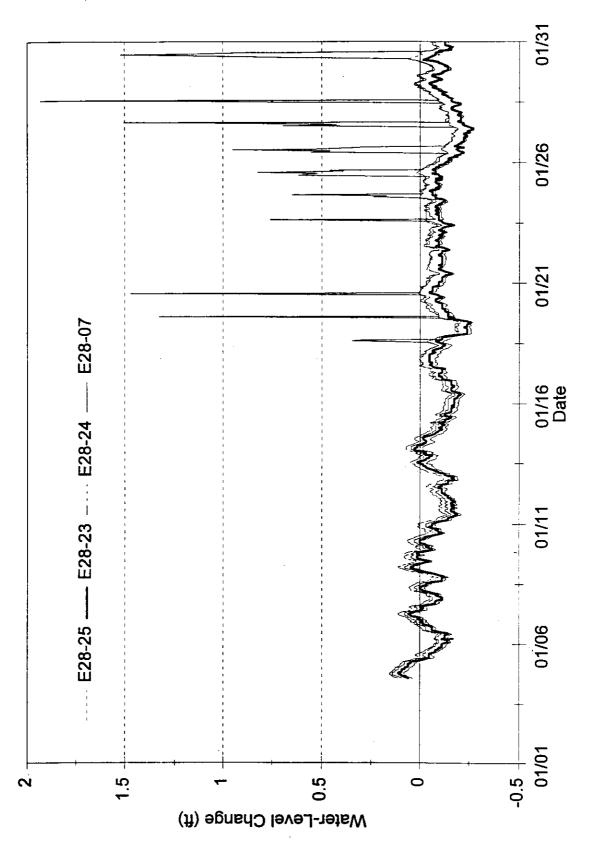


Figure D-2. 216-B-5 Reverse Well Site Water-Level Monitoring for Feburary.

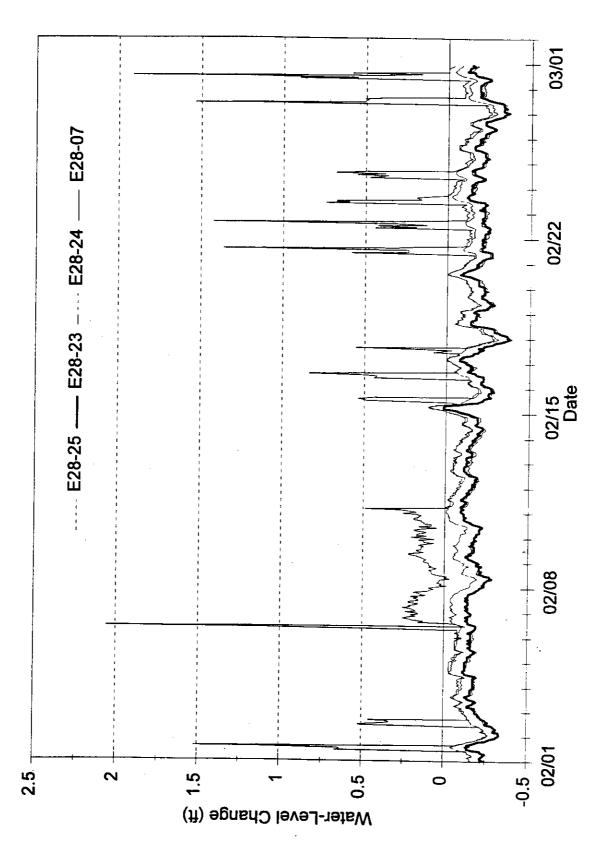


Figure D-3. 216-B-5 Reverse Well Site Water-Level Monitoring for March.

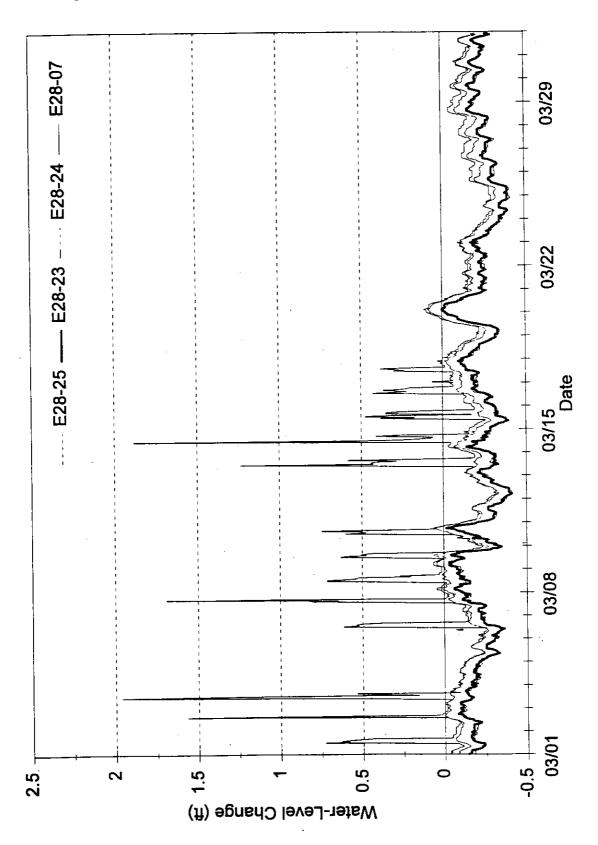


Figure D-4. 216-B-5 Reverse Well Site Water-Level Monitoring for April.

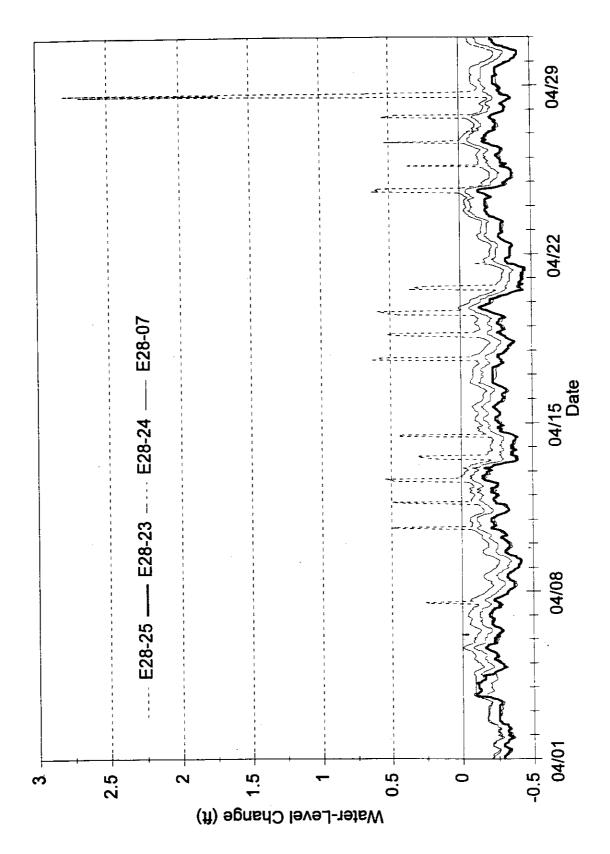
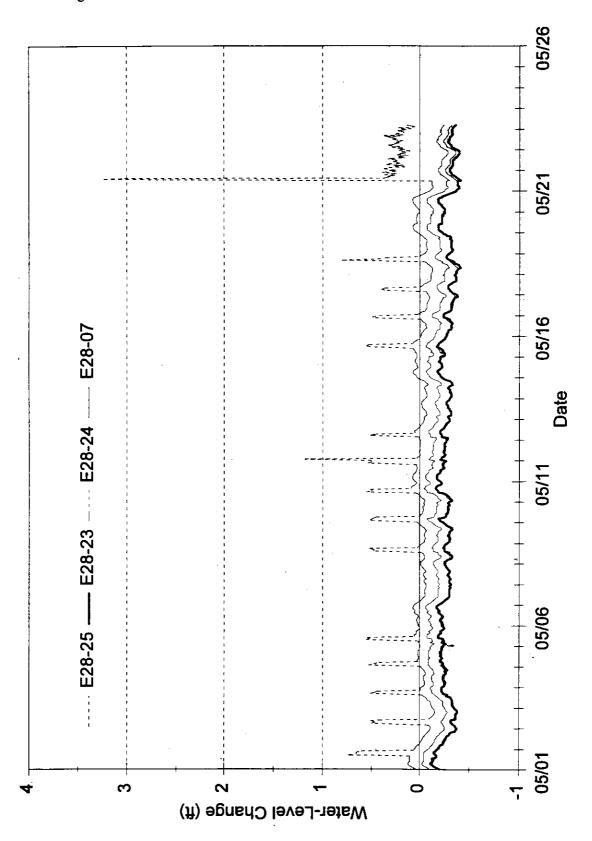


Figure D-5. 216-B-5 Reverse Well Site Water-Level Monitoring for May.



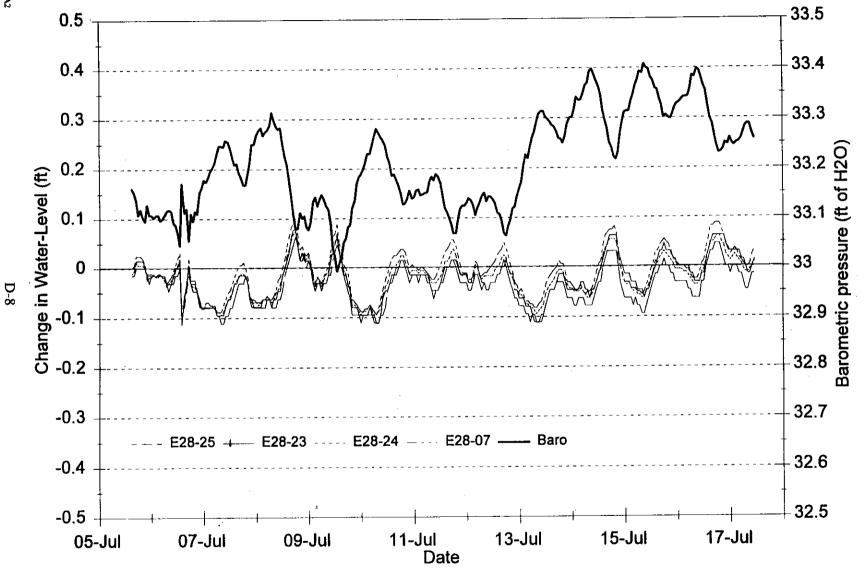


Figure D-6. 216-B-5 Reverse Well Post-Test Water-Level Monitoring.

Figure D-7. 216-BY Cribs Site Water-Level Monitoring for January.

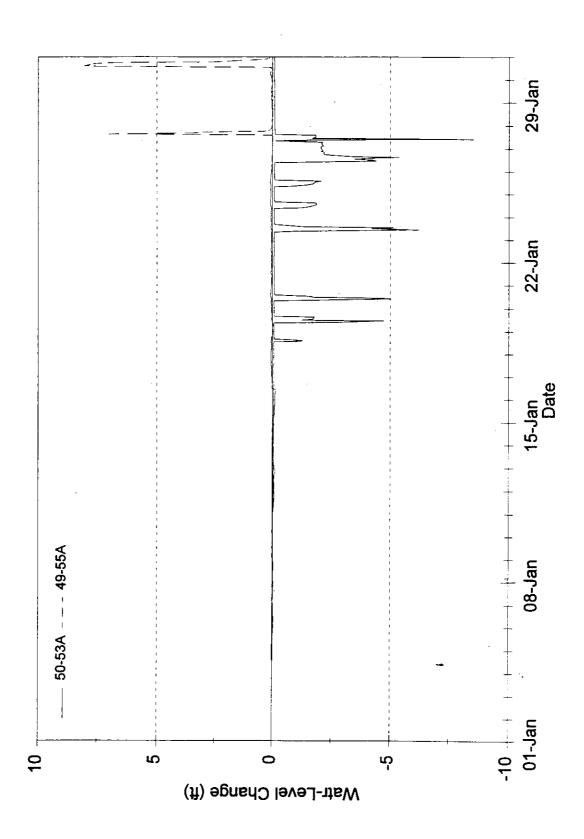


Figure D-8. 216-BY Cribs Site Water-Level Monitoring for February.

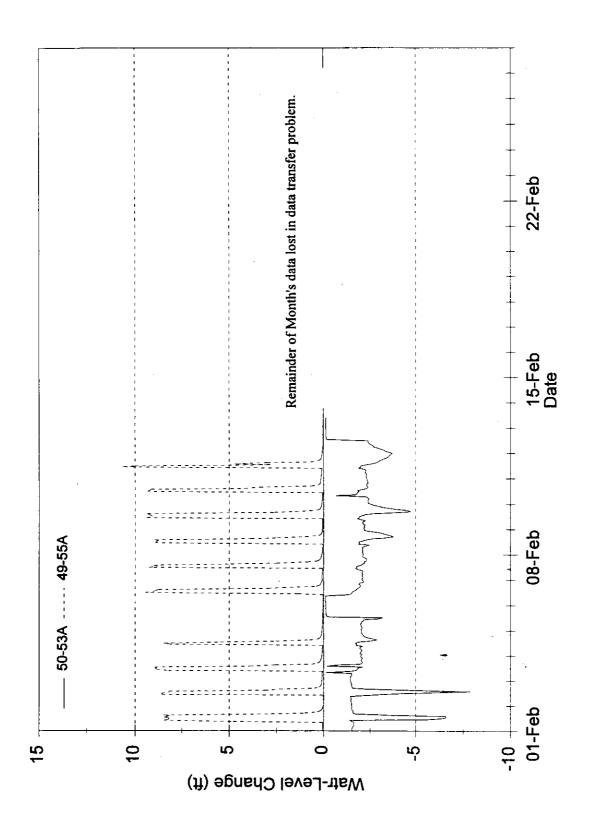


Figure D-9. 216-BY Cribs Site Water-Level Monitoring for March.

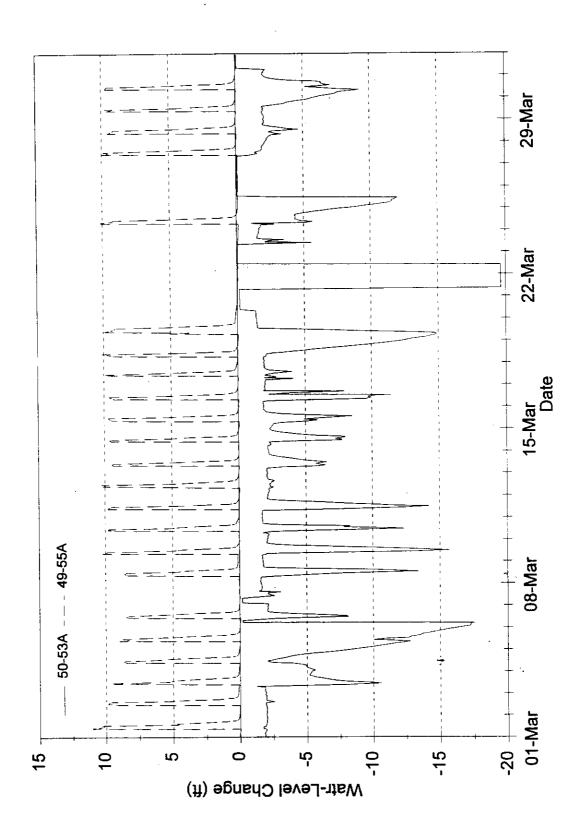
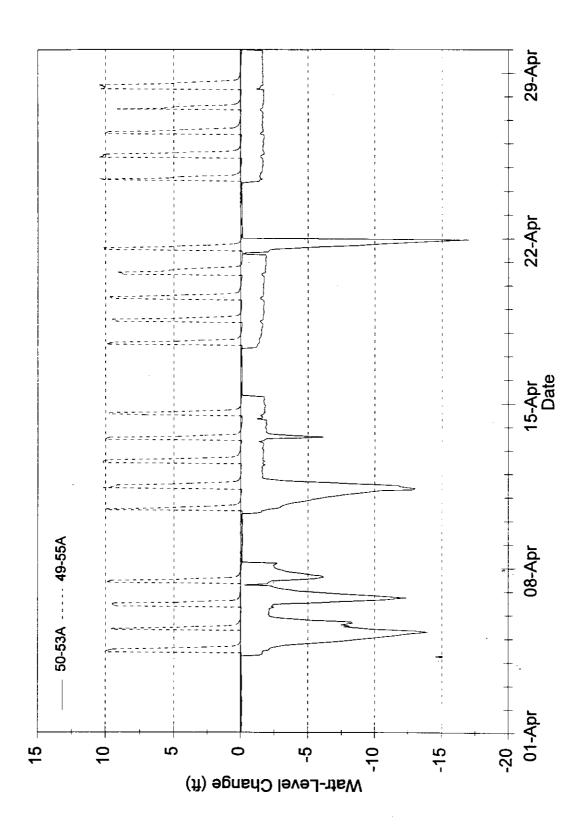
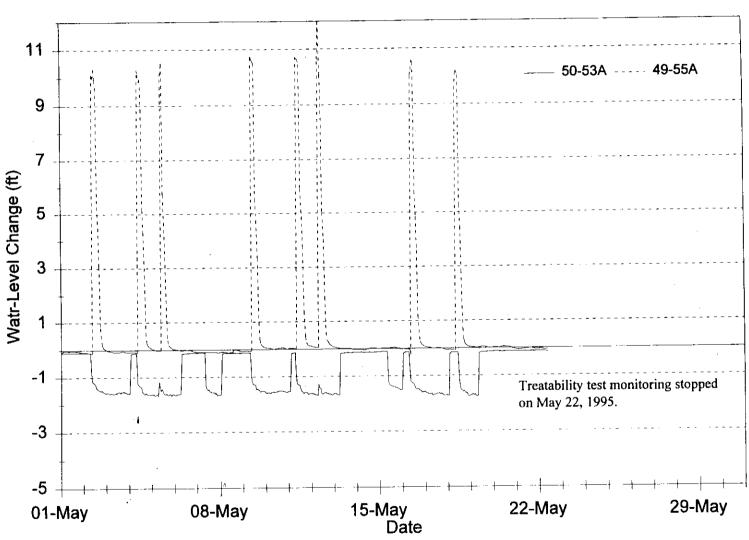


Figure D-10. 216-BY Cribs Site Water-Level Monitoring for April.





216-BY Cribs Site Water-Level Monitoring for May.



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APPENDIX E

REPORT ON IN SITU FLOW VELOCITY MEASUREMENTS AT THE 200-BP-5 OPERABLE UNIT

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APPENDIX E

REPORT ON IN SITU FLOW VELOCITY MEASUREMENTS AT THE 200-BP-5 OPERABLE UNIT

To further support the hydrogeologic assessment of the 216-BY Cribs (BY-Cribs) plume, as directed in the 200 National Priorities List Agreement/Change Control Form (BHI-00203), in situ flow velocities measurements were performed in wells monitoring the plume. The wells were all located between the north end of the 200 East Area and Gable Mountain Gap. Listed in Table E-1 are the wells in which flowmeter measurements were taken. The wells measured are located in Figure E-1. The purpose of the in situ groundwater flow velocity measurements was to determine the groundwater flow rate and direction in the vicinity of the BY-Cribs plume, in and north of the 200 East Area. While measurements were collected at only 13 specific wells, the distribution of these wells allows for an interpretation of the groundwater flow rate and direction, and the factors controlling groundwater flow, across the study area.

Groundwater flow velocities were measured using a KV Associates Model 40 groundwater flowmeter. The instrument utilizes a central heat pulse source surrounded by four matched pairs of thermistors. The heat source provides a thermal pulse which is transported in the direction and the rate of groundwater flow. By collecting measurements at the top, middle, and bottom of the well screened interval, a vertical profile of groundwater flow can be obtained, and by summing the velocity vectors, a representative flow velocity can be determined for the individual screened interval.

Thirteen groundwater monitoring wells were selected for this study. The wells selected were: screened across the water table; screened using a 4- to 6-in.-diameter wire wrapped screen; and located across the study area to provide an areal distribution of measurement points. The flow measurements were obtained between May 24, 1995 and July 27, 1995. The objective was to obtain measurements at the top, middle, and bottom of each saturated screened interval; however, several wells only had sufficient saturated interval to obtain only one or two measurements. The wells measured and measurement points below water level are listed on Table E-1.

The flowmeter measurements were collected by Westinghouse Hanford Company's Groundwater Data Services organization. To obtain a measurement, the flowmeter was lowered into the well and connected to control rods, which were indexed to ensure accurate directional alignment. Once the flowmeter and rods were lowered to the desired depth, the instrument was aligned in a north-south direction which was compensated for 21 degrees of east magnetic declination. Following alignment, the groundwater was allowed to return to hydrostatic conditions for a minimum of 1 h before collecting measurements. Following groundwater stabilization the flowmeter was triggered and data is collected using an automated data logger.

Two data collection tests were conducted at each measurement point. The first is as described above, and a second test following rotating the instrument 180 degrees from the first test. The rotation procedure accounts for any thermistor positional bias and identifies any thermistor malfunctions. Following data collection, the flow velocity vector direction and magnitude were calculated using vector addition and instrument flow rate calibration curves. Velocity is a vector having both a direction and magnitude. The direction is expressed as a compass direction, such as north 15 degrees west, and the magnitude is expressed as a rate in feet per day. In the following discussions when the term velocity is used, both a direction and magnitude are implied.

In situ groundwater flow velocity measurements are influenced by several factors related to well construction, well condition, and aquifer characteristics. The wells tested were selected to ensure that well construction and conditions would minimally impact flow measurements. Conditions that could adversely impact measurements are: screened zones which contain perforations rather than continuous wire wrapped screened; encrustation on the well screens; or a high-hydraulic conductivity contrast differential between the aquifer and filter pack material.

Aquifer heterogeneities or anisotropy can affect both the flow direction and rate. Aquifer heterogeneities are characterized by variable porosity, which can result in preferential flow paths through the aquifer materials. Preferential pathways may exist due to depositional environment such as in coarser main channel deposits bounded by finer grained deposits. In addition, preferential flow paths may be developed during well development or over time through the washing of fine-grained material from the aquifer.

Groundwater flow velocities are controlled by the effective porosity of the saturated sediments and the groundwater flow gradient. The effective porosity of the saturated medium describes the amount of connected pore space which will transmit water. Higher porosity results in lower flow velocity, assuming all other related factors are held constant.

Porosities for the sediments comprising the unconfined aquifer surrounding the BY-Cribs and north to the Gable Gap area are only generally known, and will vary from one location to another within the study area. Effective porosity estimates range from 10% to 30% (Graham et al. 1981). The initial flow velocity results were calculated using a representative effective porosity of 20%, however, results from the 216-B-5 Reverse Well (B-5 Reverse Well) Tracer tests, discussed in Section 4.4.4, determined that a porosity of between 22% and 30% is probably more representative of the unconfined aquifer at the nearby B-5 Reverse Well site. Based on the tracer study data, groundwater flow velocities were recalculated using an effective porosity of 25% based on the tracer study data. Table E-1 presents the calculated flow velocities for both effective porosities.

In the calculation of the flow velocity magnitude resulting from the instrument readings, the effective porosity is the controlling factor external from the calibration curves. Therefore, changing the effective porosity from 20% to 25% results in a 20% reduction in velocity magnitude. Thus, with an expected variable porosity across the study area of as much as 8% (22% to 30%), if only the tracer test study data are used as a range, flow velocity magnitudes may vary by 27% due to uncertainty in the effective porosity of the aquifer.

The water table in the northern portion of 200 East Area and north to Gable Gap is very flat with a very low groundwater gradient of 0.00006. Low groundwater gradients may result in low flow velocities if the aquifer is homogeneous. In the study area, water-table elevations are so similar that contouring is very difficult, and therefore determining flow direction and gradient is imprecise. This is illustrated by the June 1994 water-table elevations shown in Figure E-2. The water levels were measured over a period of days and variability of up to 0.03 m (0.1 ft) is not unexpected. Additional error is added due to uncertainty in surveying of well elevations. Errors of 0.02 ft may be introduced if the same surveying benchmark was used and as much as 0.015 m (0.05 ft) if different benchmarks were used between wells. Therefore, in areas such as the northern portion of the 200 East Area the difference between 122.23 m (401.03 ft) and 1.22.21 m (400.98 ft) is not considered significant.

The underlying basalt surface also controls groundwater flow in a manner analogous to a stream bed controlling water flow in a stream. Irregularities in the stream bed (such as rocks or islands) disrupts

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the smooth laminar flow of the stream resulting in variable flow velocity around and over the irregularities. The impacts of the stream bed, or in this case the underlying basalt surface, become more pronounced near the basalt surface where the irregular surface disrupts the flow the greatest. The basalt surface in the study area has been extensively eroded, and most likely a very irregular surface has developed. In addition, erosional channels or depressions may have developed in the basalt surface which also can direct groundwater flow oblique to the flow gradient direction.

The underlying basalt has also been structurally deformed by tectonic activity. A series of small anticlines and synclines have been interpreted from well logs. This is illustrated in Figure E-3, which is a bedrock geologic map of the 200 East Area and vicinity. Groundwater flow near the anticlines may be controlled by both the structural dip of the basalt surface and surface irregularities associated with erosion.

Because of the potential influences of porosity, well bore effects, and effects of the basalt surface measurements within a well were added using vector addition to determine an average flow velocity for a well. These average values are shown in Figure E-1 and listed in Table E-1.

Based on previous studies (DOE-RL 1993, WHC 1992) and a streamline analysis based on 1993 water-level data (Figure E-4), groundwater flow is generally to the north and northwest within the study area. The flow has been and is strongly influenced by wastewater discharges to the ground. These discharges have resulted in a groundwater divide in the 200 East Area which controls flow direction to the north in the northwestern portion of the 200 East Area.

The groundwater flow direction and magnitude for each well are shown in Figure E-1. In general across the study area the following may be inferred from the flow velocity measurements:

- Groundwater flow directions in the northwest portion of the 200 East Area is to the south, with magnitudes ranging from 0.13 m/day (0.43 ft/day) to 0.49 m/day (1.6 ft/day).
- Groundwater flow directions northeast of the BY-Cribs flows northwest around a basalt high with a magnitude of 0.6 m/day (1.9 ft/day).
- Groundwater north of the 200 East Area and south of West Lake flow northerly with a magnitude ranging from less than 0.01 m/day (0.04 ft/day) to 0.34 m/day (1.1 ft/day).

Groundwater velocity measurements less than 0.01 m/day (0.04 ft/day) are considered less than the practical resolution of the flow meter and necessarily invalidate the flow direction and rate. There was only one location, well 699-52-54, where both flow measurement readings were below the practical resolution of the meter.

Based on the velocity meter results, groundwater flow direction, in the northwest portion of the 200 East Area, is generally to the south. Flow velocity variation was observed and is likely due to the flat gradient and aquifer heterogeneities. Flow velocities varied between 0.13 to 0.34 m/day (0.43 to 1.1 ft/day). The 1993 water-table map (see Figure E-4) shows a hydrologic divide within the central portion of the 200 East Area due to past cooling water disposal activities. A streamline analysis shows that groundwater flow, in 1993, was to the northwest. Groundwater levels have been lowering with time due to decreased discharges to the soil column, and the hydrologic divide may have moved north sufficiently for groundwater flow directions to change to a more southerly

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direction. Groundwater flow in this area may be controlled by a more regional flow gradient rather than the flat local gradient.

The flow direction in the vicinity of well 299-E34-5 is to the northwest with a flow velocity of 0.6 m/day (1.9 ft/day), which was the highest measured. This higher flow velocity is supported by the water level at this well, which also is the highest in the area. The groundwater high may be due to the influence of the groundwater mound beneath B Pond to the southeast. In addition, the groundwater velocity may be influenced by the underlying basalt surface, which is only 0.9 m (3 ft) below the water table.

Also in this vicinity, well 299-E34-11 flows northeasterly and well 699-48-50 flows southerly, which is anomalous to well 299-E34-5. This may be due to the underlying basalt locally controlling flow at wells 299-E34-11 and 699-48-50. Well 299-34-11 has only 1.2 m (4 ft) of saturated interval above the basalt surface and well 699-48-50 has only 3 m (10 ft). Flow velocities may be influenced by the features on basalt surface such as erosional channels or depressions oblique to the general groundwater flow direction shown by well 299-E34-5. In addition, two of the three measurements collected in well 699-48-50 were below the 0.04 practical flowmeter resolution.

Flow directions heading into the Gable Gap area are consistent with a flat water table with a northerly gradient but may be partially controlled by the underlying basalt surface. Well 699-52-57 only has approximately 0.6 m (2 ft) of saturated interval above the basalt, and only one flow measurement was collected from this well. Two measurements were collected at well 699-52-54 and both were below the practical flowmeter resolution. This well has only approximately 0.9 m (3 ft) of saturated thickness. This well may penetrate into a small depression in the basalt surface which has only a very small flow.

The flow direction at well 699-59-58 is southwesterly with a magnitude of 0.20 m/day (0.66 ft/day). This is anomalous to the flow direction predicted from previous studies and from the wells to the south. Structure contour maps of the area show that this well, with 4.6 m (15 ft) of saturated thickness above the basalt bedrock, is on the southerly dipping limb of an anticlinal structure. Erosional features on this southerly dipping limb may be controlling flow in this individual well, with the general regional flow maintaining its northerly flow direction. However, without additional velocity data this cannot be confirmed.

Well 699-60-57 is screened across two zones. The upper zone contains approximately 1.5 m (5 ft) of saturated screened interval, and the lower zone is screened across a section 17 m (57 ft) lower in the well. Results show that groundwater flows in opposite directions between the upper and lower screened intervals. The upper interval flows southwest and the lower interval flows northeast. This well also lies on the axis of an anticlinal structure. This could have an effect on flow directions near the basalt surface. An alternative explanation is that the two screened intervals may represent different aquifers, separated by a confining layer, which flow in opposite directions. Well logs show that a very-fine to fine sand with silt was encountered between the two screen intervals. This zone could cause semiconfined to confined conditions in the lower screened interval. Opposite flow directions between unconfined and confined aquifers have been documented at Fort Ord, California (HLA 1994). Either of the two scenarios are reasonable.

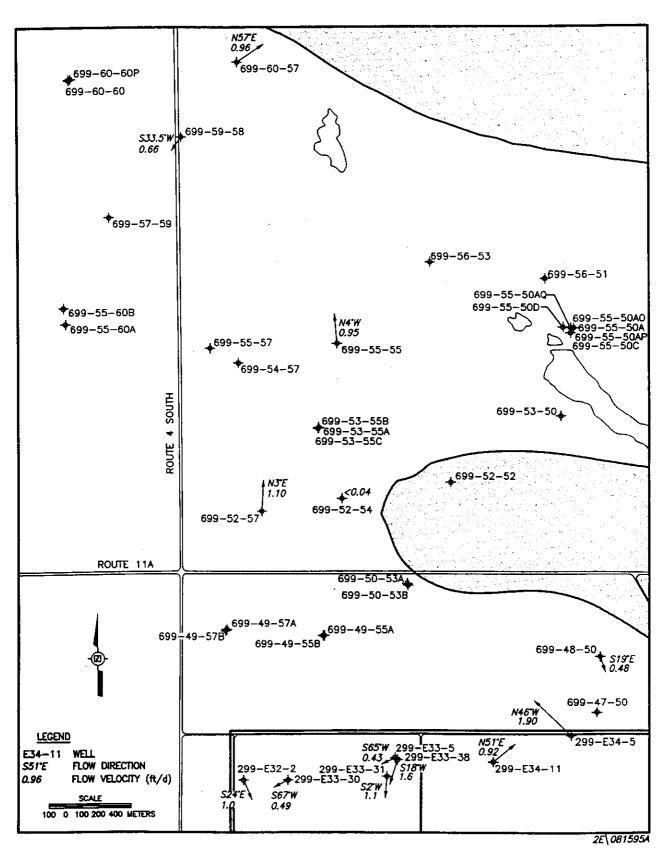
In summary, in situ groundwater flow velocity measurements provide general groundwater flow directions and velocities in areas of very low hydraulic gradient. Groundwater flow velocity measurements can be influenced by aquifer porosity, well bore effects, and the underlying basalt

surface. Groundwater flow in the northeastern portion of the 200 East Area are to the south, at velocities ranging between 0.13 and 0.34 m/day (0.43 and 1.1 ft/day). This is opposite to flow directions interpreted from water-table elevations collected in 1993 which show a northwesterly flow direction. This change in direction may be due to a declining water table and movement of the 200 East Area hydrologic divide to the north. Groundwater east of this area and to the north flows towards Gable Gap and is controlled by the low hydraulic gradient and locally by the underlying basalt surface.

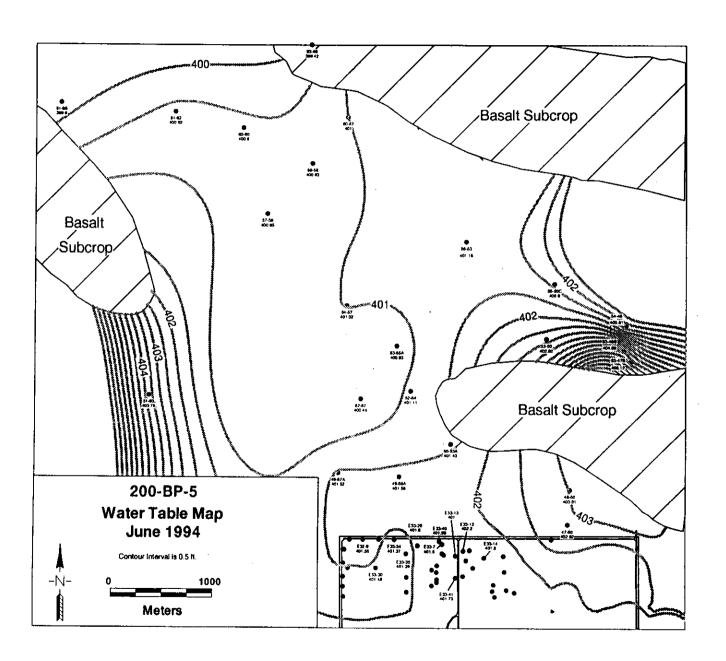
REFERENCES

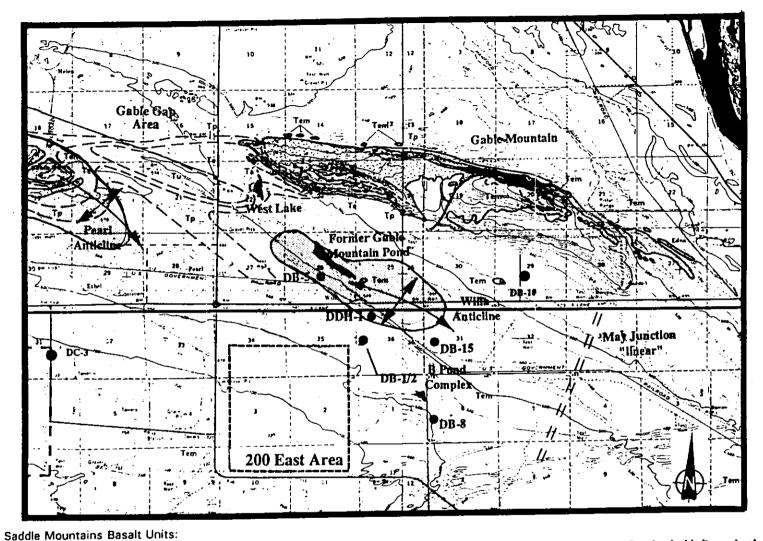
- DOE-RL, 1993, 200 East Groundwater Aggregate Area Management Study Report, DOE/RL-92-19, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
- Graham, M.J, G.V. Last, S.R. Strait, and W.R. Brown, 1981, *Hydrology of the Separations Area*, RHO-ST-42, Rockwell Hanford Operations, Richland, Washington
- HLA, 1994, Basewide Remedial Investigation/Feasibility Study Fort Ord, California, Harding Lawson Associates, Navato, California.
- WHC, 1992, Hydrogeologic Model for the 200 East Groundwater Aggregate Area, WHC-SD-EN-TI-019, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

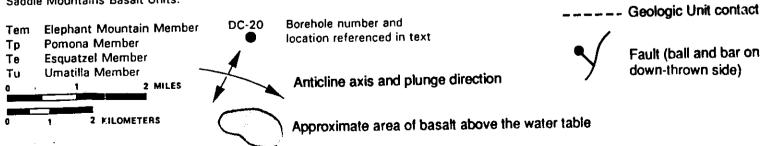
Figure E-1. In Situ Velocity Flowmeter Wells in the 216-BY Cribs Plume.



Water-Table Map at 216-BY Cribs Plume.







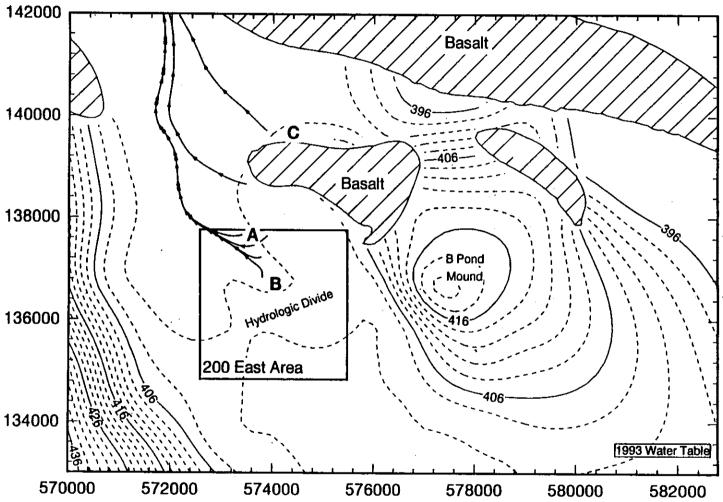


Figure E-4.

Streamline Analysis for 200 East Area, 1993.

Table E-1. In Situ Flow Meter Results.

Well	DBWL (ft)	Velocity (ft/d)		
		$\phi = 0.20\%$	$\phi = 0.25\%$	Direction
299-E32-2	3.7	0.89	0.7	S30°E
	7.7	1.8	1.4	\$34.3°E
Average			1.0	S24°E
299-E33-30	4.3	0.73	0.58	S69.9°W
	8.3	0.66	0.53	S63°W
	12.3	0.45	0.36	S68°W
Average		· · · · · · · · · · · · · · · · · · ·	0.49	S67°W
299-E33-31	2.9	1.7	1.3	S22.6°W
	6.9	2.1	1.7	\$18.7°E
A	8.9	0.59	0.47	\$20.0°W
Average	-		1.1	S2°W
299-E33-35	2.8	0.33	0.27	S42.2°W
	5.8 8.8	1.2 0.23	0.94 0.18	S78.4°W
Aviorage	0.0	0.23	+	\$21.3°W
Average			0.43	S64.5°W
299-E33-38	4.2	1.4	1.1	S23.6°W
	8.2	2.4	2.0	\$15.6°W
Average			1.6	S18.4°W
299-E34-5	1.4	2.37	1.9	S46.6°W
299-E34-11	2.0	1.2	0.96	N51.4°E
	3.5	1.1	0.87	N51.4°E
Average			0.92	N51.4°E
699-48-50	1.2	< 0.04	< 0.04	a
	5.7	0.60	0.48	S18.6°E
	9.7	0.04	< 0.04	(S52.7°E) ^a
Average			0.48	\$18.6°E
699-52-54	0.9	0.04	< 0.04	(S46.5°E) ^a
	1.4	< 0.04	< 0.04	a
Ачегаде		No		
699-52-57	0.8	1.4	1.1	N2.9°E
699-55-55	2.1	1.5	1.2	N1.0°E
	5.1	0.99	0.79	N21.7°W
	8.1	1.1	0.91	N4.3°E
Average			0.95	N4.1°W

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Table E-1. In Situ Flow Meter Results.

Well	DBWL (ft)	Velocity (ft/d)		D:4:
		$\phi = 0.20\%$	$\phi = 0.25\%$	Direction
699-59-58	2.2 6.3 9.3	0.60 0.77 1.2	0.48 0.61 0.93	\$11.2°W \$35.0°W \$43.9°W
Average			0.66	S33.5°W
699-60-57	3.0 65.5 69.5	2.0 0.86 1.6	1.6 0.69 1.2	S79.3°W N55.6°E N58.1°E
Average			0.96 ^b	N57.2°E

NOTES:

bOnly the two lower flow velocities and directions were used in determining the average values.

Average = average value calculated using vector addition methods. NOTE: not an arithmetic average.

DBWL = depth below water level

 ϕ = effective porosity

^{*}Flow velocities below 0.04 ft/day are below practical measurement capabilities of instrument. Values in parentheses are directions for velocities calculated using 0.20% porosity.

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APPENDIX F

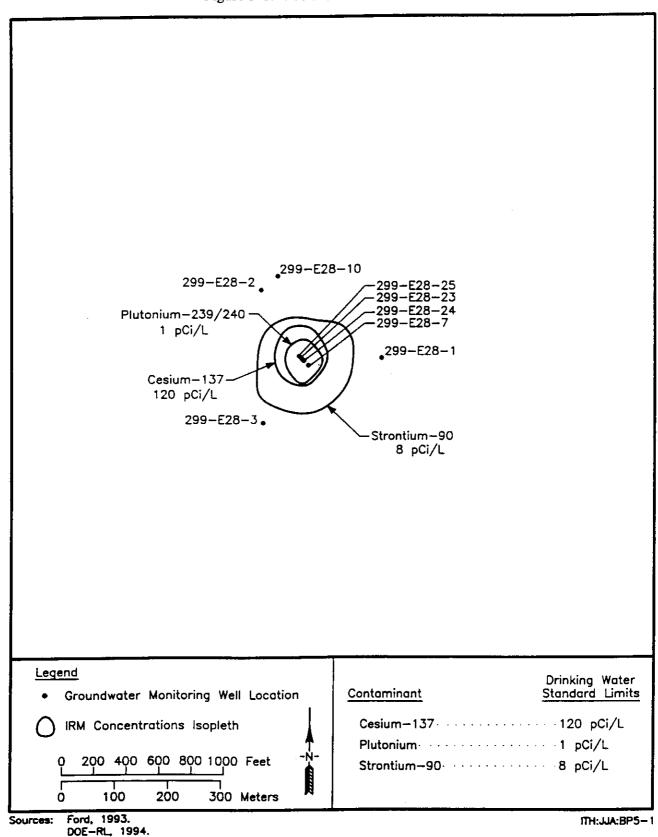
AS-BUILT AND SITE MAPS FOR 200-BP-5 PUMP-AND-TREAT SITES

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Table F-1. Treatability Test Wells Summary Table.

WELL #	DRILLED	TOTAL DEPTH	OPEN INTERVAL	DIAMETER	TEST USE
216-B-5	1945	299′	240'-299'	8"	N/A
299-E28-7	1948/79	338′	270′-335′	6"	1st Injection Well
299-E28-23	1979	328′	278'-328'	4"	Exteraction Well
299-E28-24	1980	327′	277′-327′	4"	Obeservatio n
299-E28-25	1980	329′	279′-329′	4"	2nd Injection Well
699-50-53A	1955	159′	142'-156'	8"	Extraction Well
699-50-53B	1990	225′	215'-225'	4"	Observation
699-49-55A	1961	141′	124′-139	6"	Injection Well
699-49-55B	1982	227'	175′-226′	6"	Observation
699-49-57A	1956	164'	144'-161'	8#	Observation
699-49-57B	1990	230′	219'-230'	4"	Observation

Figure F-1. 216-B-5 Reverse Well Plume.



F-4

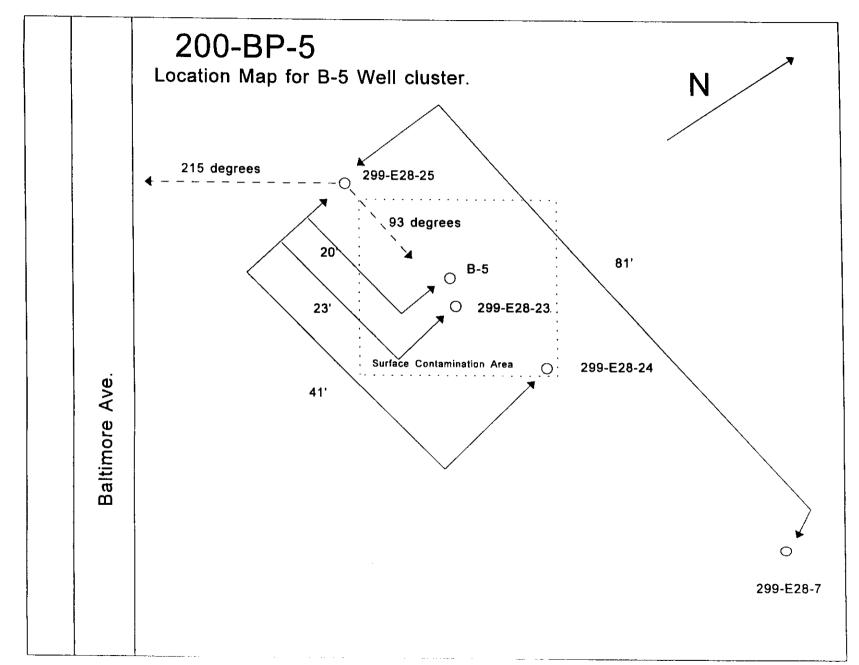


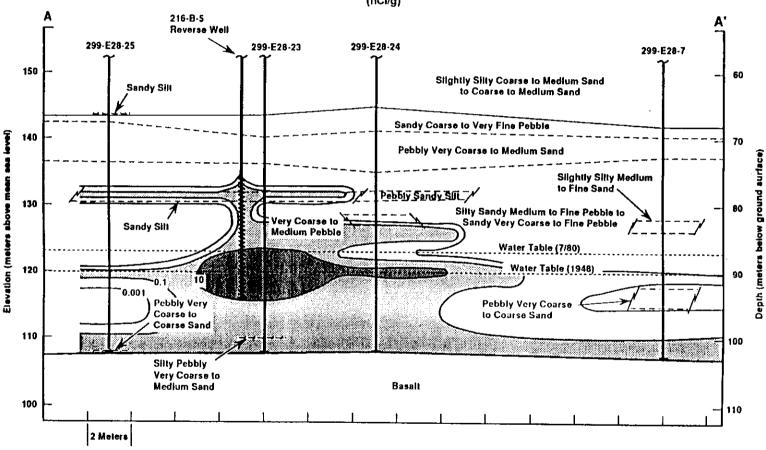
Figure F-2. 200-BP-5 Location Map for the 216-B-5 Reverse Well Cluster

Figure F-3.

igure F-3. Cross-Section of Cesium-137 Distribution (pCi/L) in the Vicinity of the 216-B-5 Reverse Well. (Smith 1980)

Geologic Cross Section A-A'



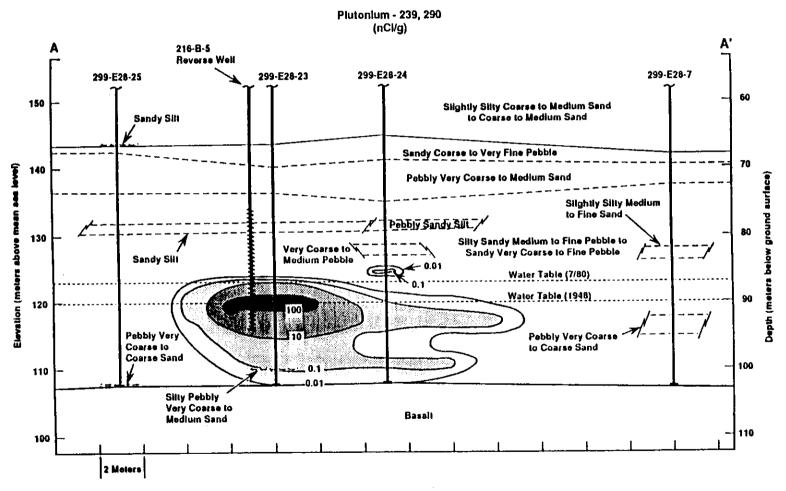


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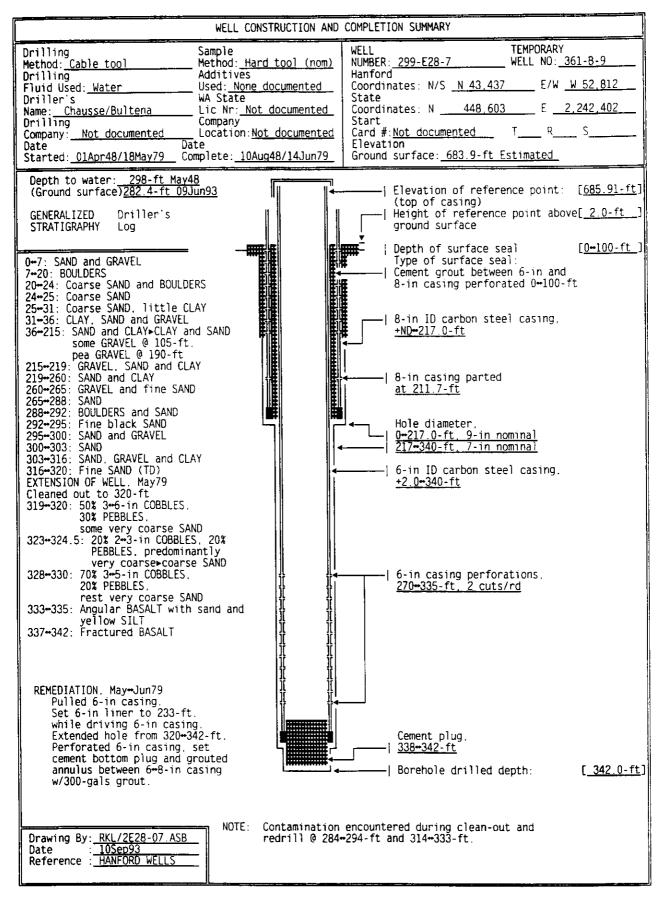
Figure F-4.

re F-4. Cross-Section of Plutonium-239/240 Distribution (pCi/L) in the Vicinity of the 216-B-5 Reverse Well. (Smith 1980)

Geologic Cross Section A-A'



DOE/RL-95-59 Rev. 0



9613494 17202

DOE/RL-95-59 Rev. 0

SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 299-E28-7

299-E28-7 WELL DESIGNATION

Not applicable RCRA FACILITY

CERCLA UNIT 200 Aggregate Area Management Study

HANFORD COORDINATES

N 43,437 W 52.812 N 448.603 E 2.242.402 [HANCONV] LAMBERT COORDINATES :

DATE DRILLED Aug48-Deepened Jun79

342.0-ft DEPTH DRILLED MEASURED DEPTH (GS) : Not documented DEPTH TO WATER (GS) : 298-ft. May48: 282.4-ft, 09Jun93

8-in, carbon steel, ND-217.0-ft: CASING DIAMETER

6-in carbon steel +2.1-340.0-ft

ELEV TOP CASING

685.91-ft. 683.9-ft. Estimated ELEV GROUND SURFACE :

PERFORATED INTERVAL : 270+335-ft Not applicable SCREENED INTERVAL

COMMENTS FIELD INSPECTION, 10Aug93.

6 and 8-in carbon steel casing. Capped and locked

2-ft pad, 4 posts, identification stamped on brass marker.

Not in radiation zone.

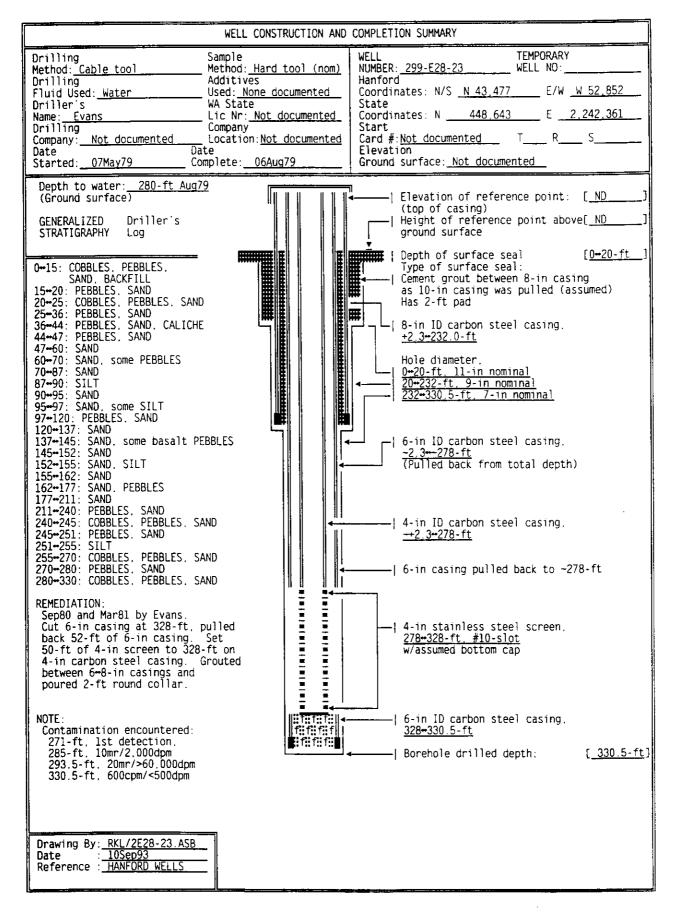
Driller AVAILABLE LOGS TV SCAN COMMENTS Not applicable DATE EVALUATED Not applicable **EVAL RECOMMENDATION** Not applicable

LISTED USE Seperations area semiannual water levels measurement, 20Apr73+09Jun93; WHC ES&M w/l monitoring.

CURRENT USER PNL sitewide sampling 93

PUMP TYPE Hydrostan

MAINTENANCE



4613494 (11)

DOE/RL-95-59 Rev. 0

SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 299-E28-23

WELL DESIGNATION 299-E28-23 RCRA FACILITY Not applicable

200 Aggregate Area Management Study CERCLA UNIT

HANFORD COORDINATES

N 43,477 W 52.852 N 448.643 E 2,242.361 [HANCONV] LAMBERT COORDINATES Aug79-Completed Sep80 and Mar81 DATE DRILLED

DEPTH DRILLED (GS)
MEASURED DEPTH (GS)
DEPTH TO WATER (GS) 330.5-ft Not documented 280-ft, Aug79

8-in, carbon steel, +2.3-232.0-ft; 6-in carbon steel +2.3-278.0-ft CASING DIAMETER 4-in carbon steel, +2.3+278-ft

ELEV TOP CASING Not documented ELEV GROUND SURFACE : Not documented

PERFORATED INTERVAL : None SCREENED INTERVAL 278+328-ft

FIELD INSPECTION, 240ct89. COMMENTS

3 carbon steel casings. 2-ft pad. no posts. capped. and locked.

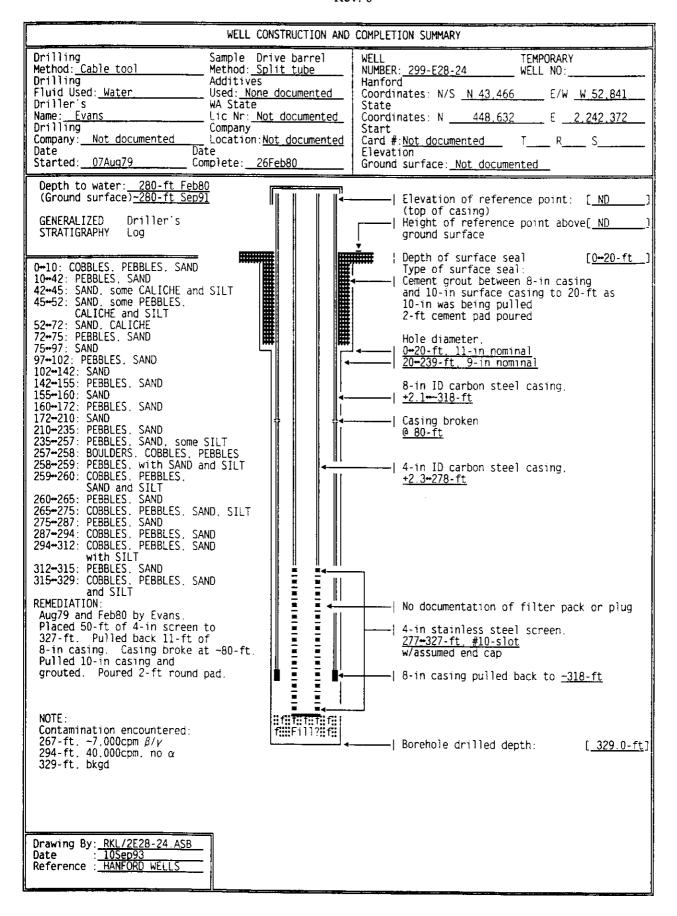
No permanent identification. In radiation zone.

Driller, Geologist Not applicable AVAILABLE LOGS TV SCAN COMMENTS DATE EVALUATED Not applicable EVAL RECOMMENDATION : Not applicable

LISTED USE No water level measurement data, CURRENT USER WHC ES&M operational sampling. PNL sitewide sampling 93

PUMP TYPE Electric submersible

MAINTENANCE



SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 299-E28-24

299-E28-24 WELL DESIGNATION Not applicable RCRA FACILITY

200 Aggregate Area Management Study CERCLA UNIT

HANFORD COORDINATES :

N 43,466 W 52,841 N 448,632 E 2,242,372 [HANCONV] LAMBERT COORDINATES Feb80-Completed Aug79 and Feb80 DATE DRILLED

DEPTH DRILLED (GS) 329.0-ft MEASURED DEPTH (GS) Not documented 280-ft. Aug79. 282.9-ft. Sep91 DEPTH TO WATER (GS) (TOC):

8-in, carbon steel, +2.1-318.0-ft; 4-in carbon steel, +2.1-278-ft CASING DIAMETER

Not documented ELEV TOP CASING ELEV GROUND SURFACE Not documented None

PERFORATED INTERVAL SCREENED INTERVAL

277+327-ft FIELD INSPECTION, 240ct89 COMMENTS

2-ft round cement pad. Capped and locked. No posts, no permanent identification.

8-in and 4-in carbon steel casing. In radiation zone.

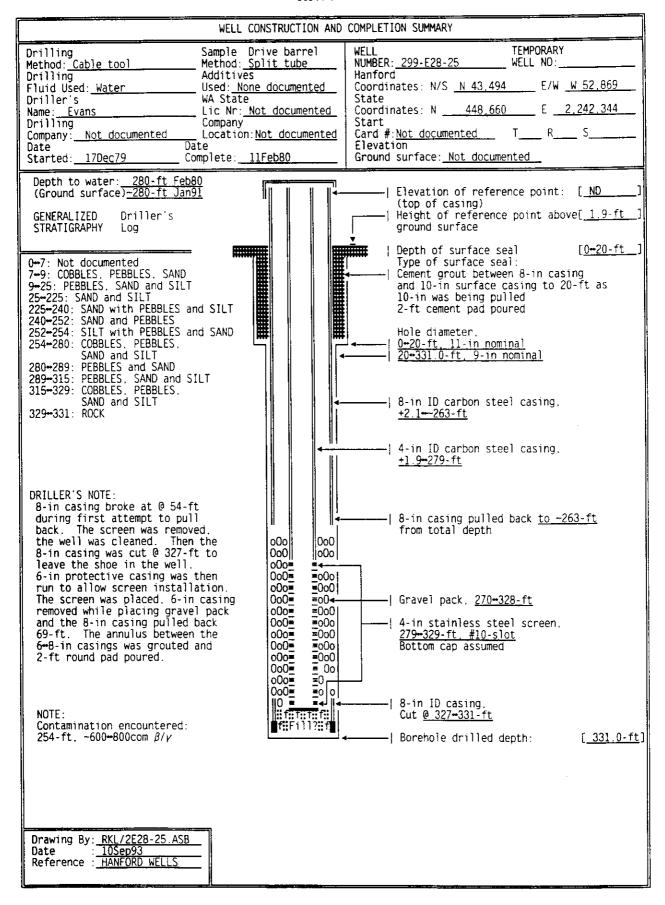
AVAILABLE LOGS TV SCAN COMMENTS Driller, Geologist Not applicable DATE EVALUATED Not applicable EVAL RECOMMENDATION Not applicable

LISTED USE Water levels measured (depth to water only) 06Apr90→23Sep91

CURRENT USER PNL sitewide sampling 93

PUMP TYPE None documented

MAINTENANCE



SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 299-E28-25

299-E28-25 WELL DESIGNATION RCRA FACILITY

Not applicable 200 Aggregate Area Management Study CERCLA UNIT

N 43,494 W 52,869

HANFORD COORDINATES : N 448,660 E 2,242,344 [HANCONV]

DATE DRILLED Feb80 331.0-ft DEPTH DRILLED (GS) : MEASURED DEPTH (GS) Not documented 280-ft, Feb80; 280-ft, 31Jan91 DEPTH TO WATER (GS) :

8-in. carbon steel. +1.9--263.0-ft; CASING DIAMETER 4-in carbon stee!, +1.9-279-ft

Not documented ELEV TOP CASING ELEV GROUND SURFACE : Not documented

PERFORATED INTERVAL : None 279+329-ft SCREENED INTERVAL

COMMENTS FIELD INSPECTION, 240ct89

2-ft round cement pad. Capped and locked. No posts, no permanent identification.

8-in and 4-in carbon steel casing. Not in radiation zone.

AVAILABLE LOGS Driller, Geologist Not applicable TV SCAN COMMENTS Not applicable DATE EVALUATED **EVAL RECOMMENDATION:** Not applicable

Water levels measured (depth to water only) 23Mar90+31Jan91 LISTED USE

PNL sitewide sampling 93 CURRENT USER

PUMP TYPE MAINTENANCE None documented

Figure F-5. Contaminant Plumes and Location Map.

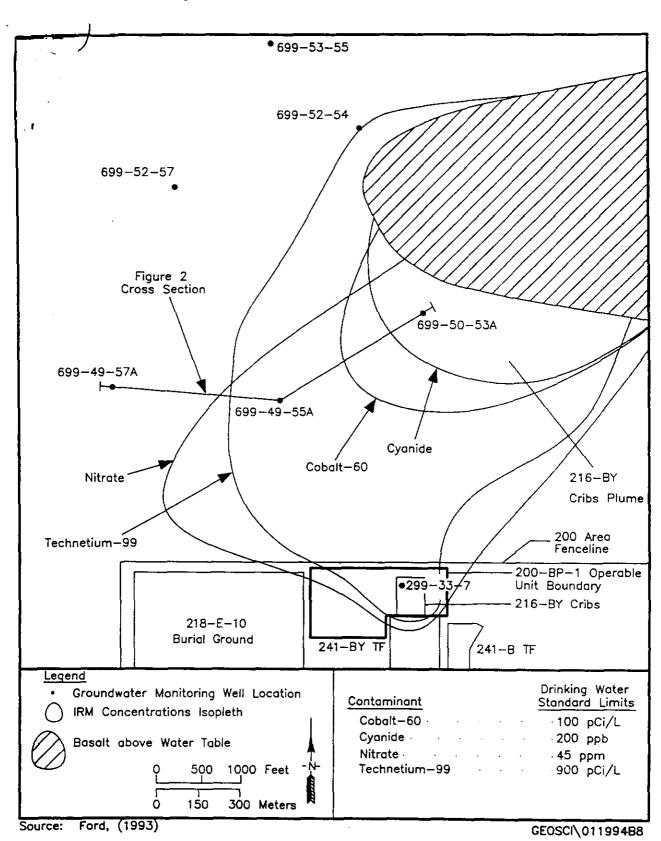


Figure F-6. Location Map for the 216-BY Cribs.

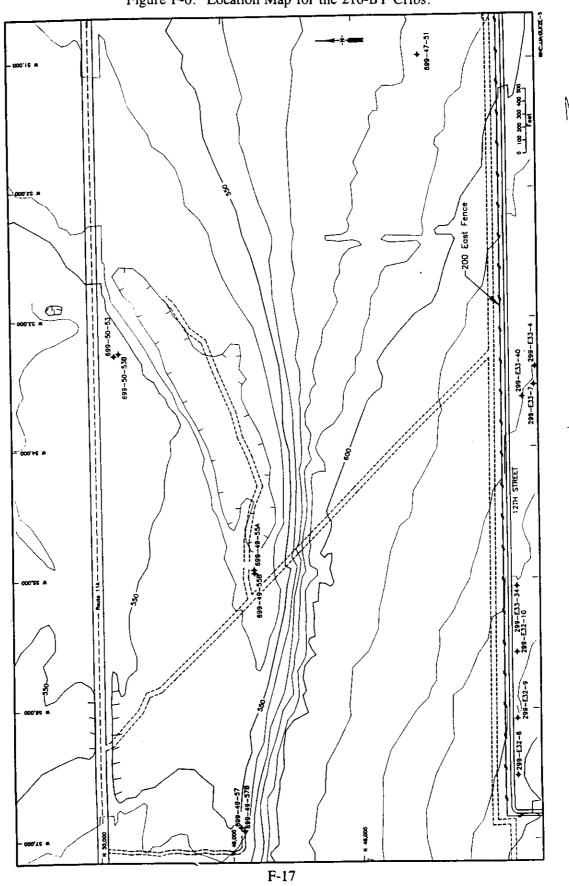
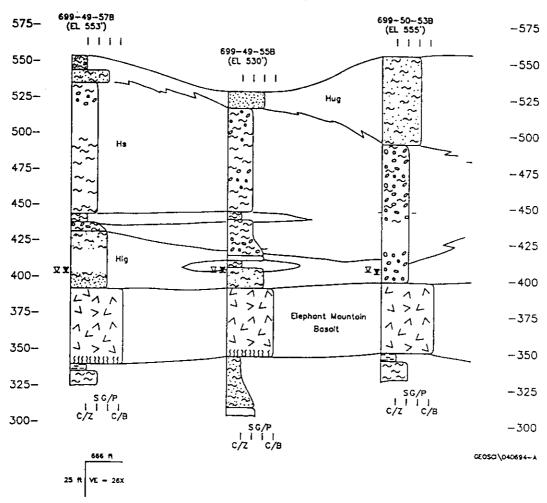
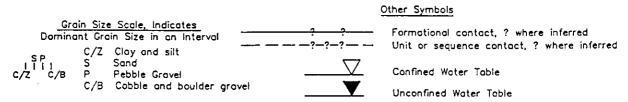


Figure F-7. Geologic Cross Section.





Additional Lithologic Symbols, Includes Subordinate Lithologies

_	M	-!-1-
	Clay	LICU

~_~ Silt rich

www. Sondy

Pebble Gravel

000 Cobble and Boulder

515 Bosoit

Stratigraphic Abbreviations

Hun — Undifferentiated Hanford formation

Hug — Upper gravel sequence, Hanford formation

Hs - Sandy sequence, Hanford formation

Hig - Lower gravel sequence, Honford formation

EM - Elephont Mountain Member

RRI - Rattlesnake Ridge interbed

P - Pomono Member

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As-Built Diagrams for Wells:

699-50-53A

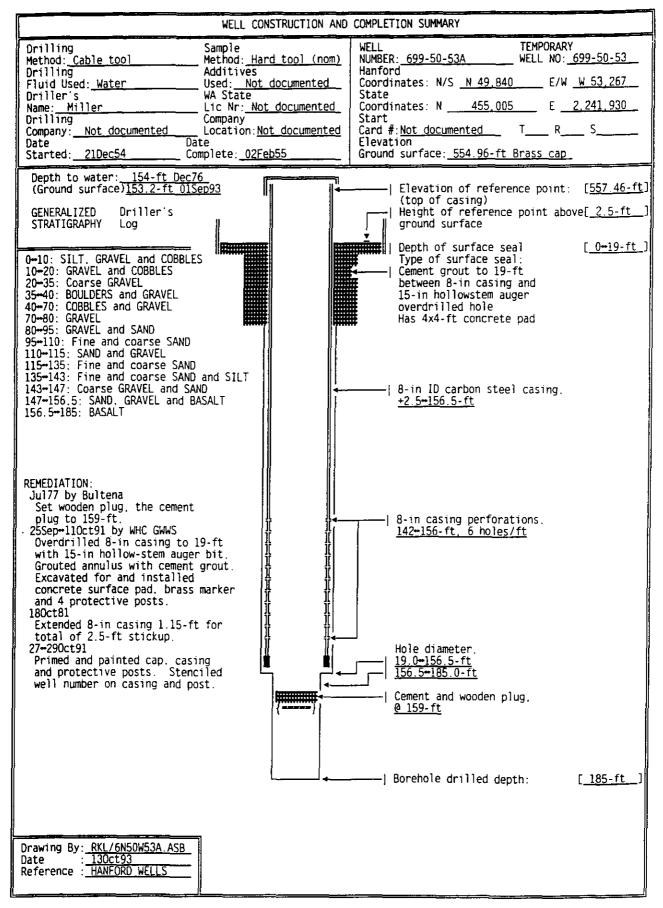
699-50-53B

699-49-55A 699-49-55B

699-49-57A

699-49-57B

F-19



SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 699-50-53A

699-50-53A WELL DESIGNATION Not applicable RCRA FACILITY

200-BP-1 (200 Aggregate Area Management Study) CERCLA UNIT

HANFORD COORDINATES LAMBERT COORDINATES N 49,840 53,267 W

N 455.005 E 2,241,930 [HANCONV]

DATE DRILLED Feb55 DEPTH DRILLED (GS) 185-ft

162-ft, Aug90 TV 154-ft, Dec76: 153.2-ft, 01Sep93 MEASURED DEPTH (GS) DEPTH TO WATER (GS)

8-in, carbon steel. +2.5-156.5-ft 557.46-ft. [25Feb92-NGV] CASING DIAMETER [25Feb92-NGVD'29) ELEV TOP OF CASING ELEV GROUND SURFACE 554.96-ft, Brass cap [25Feb92-NGVD'29]

142-156-ft PERFORATED INTERVAL Not applicable SCREENED INTERVAL

FIELD INSPECTION, 110ct89. COMMENTS Carbon steel casing.

No pad, capped and locked no posts. Identification stamped on brass marker.

FIELD INSPECTION, 060ct92,

8-in carbon steel casing. Capped and locked

4-ft by 4-ft concrete pad, 4 posts. Identification stamped on brass marker in pad.

Not in radiation zone. OTHER:

Driller AVAILABLE LOGS

Depths referenced to ground surface; TV SCAN COMMENTS

Jan90:

Depth to bottom: 160-ft Bottom of casing: 156-ft Depth to water: 150-ft

Perforated 138-156-ft (?). 6 cuts/rd/ft. Material flaking off caused by camera.

Casing has average scale build-up.

10Aug90:

Depth to bottom: 162-ft

Depth to water: 151-ft, floating debris

Casing clean. Perforated 139+(?)-ft, 6 cuts rd/ft. The visible were open. Water murky with suspended debris. Vision obscured by possibly algae.

13Nov90 DATE EVALUATED

EVAL RECOMMENDATION : Install a 2-in void grout surface seal outside

8-in casing to approximately 18-ft. OR: Perforate 3-18-ft and install a 4-in liner with cement basket to approximately

20-ft. Pressure grout to 40-psi.

Install protective posts and concrete pad per WAC 173-160-510 and field

conditions.

4. Survey to water level measurement standards. 200 BP-1 quarterly water level measurement, 07Feb55+01Sep93 LISTED USE

WHC ER characterization and ES&M w/l monitoring, CURRENT USER PNL sitewide sampling and w/l monitoring 93

Hydrostar, intake at 156.9-ft, (top-of-casing) PUMP TYPE

21May90: Pulled pump, scrubbed casing and bailed debris. Considerable MAINTENANCE

contamination found.

06&11Jun90: Developed well with pump to <5 NTU.

13Aug90: Installed pump and new cap. 25Sep91: Cleared site.

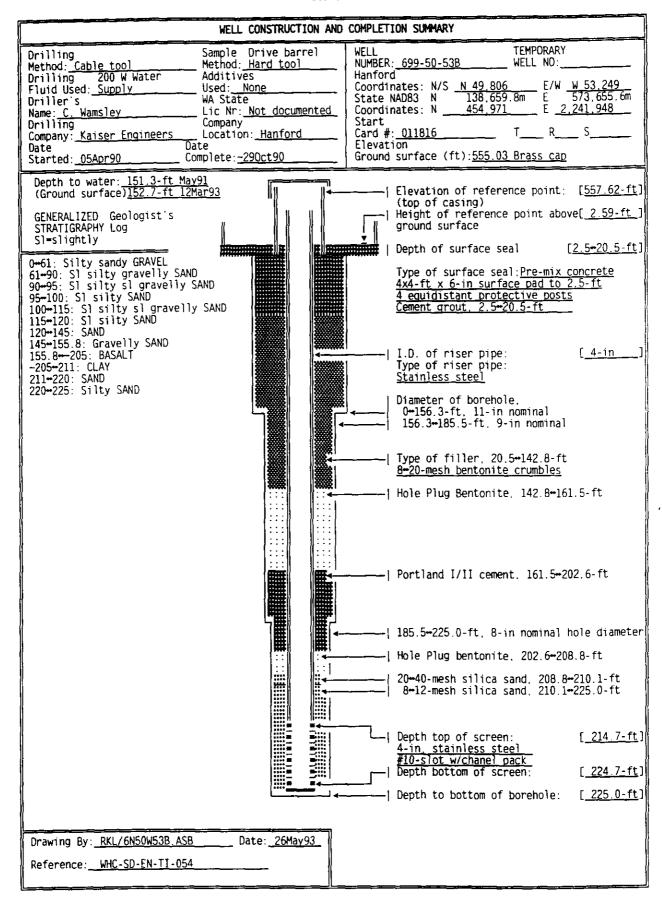
REMEDIATION

01-020ct91; Overdrilled 8-in casing to 19-ft with 15-in hollow-stem auger.

Grouted annulus with cement grout (Al powder added). 07-090ct91; Excavated for surface pad and protective posts.

110ct91: Installed 4-ft by 4-ft surface pad, brass marker and 4 protective posts. 180ct91: Extended casing 1.15-ft for 2.50-ft stickup.

27-290ct91: Primed and painted cap, casing and protective posts. Stenciled well number on casing and post.



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SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 699-50-53B

699-50-53B WELL DESIGNATION

200-BP-1 (200 Aggregate Area Management Study) CERCLA UNIT

Not applicable RCRA FACILITY

53,249 [25Jan91-200E] HANFORD COORDINATES N 49.806 E 2,241.948 [HANCONV] E 573,655.6m [25Jan91-NAD83] N 454,971 LAMBERT COORDINATES :

N 138,659.8m E

DATE DRILLED -0ct90 DEPTH DRILLED (GS): 225.0-ft MEASURED DEPTH (GS) : Not documented 151.3-ft, 07May90: DEPTH TO WATER (GS) :

152.7-ft, 12Mar93

4-in stainless steel. +0.3+214.7-ft; 6-in stainless steel. +2.59-0.5-ft CASING DIAMETER

[25Jan91-NGVD:29] ELEV TOP CASING ELEV GROUND SURFACE 557.62-ft. 555.03-ft, Brass cap [25Jan91-NGVD 29]

Not applicable PERFORATED INTERVAL

214.7+224.7-ft, 4-in #10-slot stainless steel w/channel pack SCREENED INTERVAL

FIELD INSPECTION, 07Mar91; COMMENTS

4 and 6-in stainless steel casing. 4-ft by 4-ft concrete pad. 4 posts, 1 removable. Capped and locked, brass cap in pad with well ID.

Not in radiation zone.

OTHER:

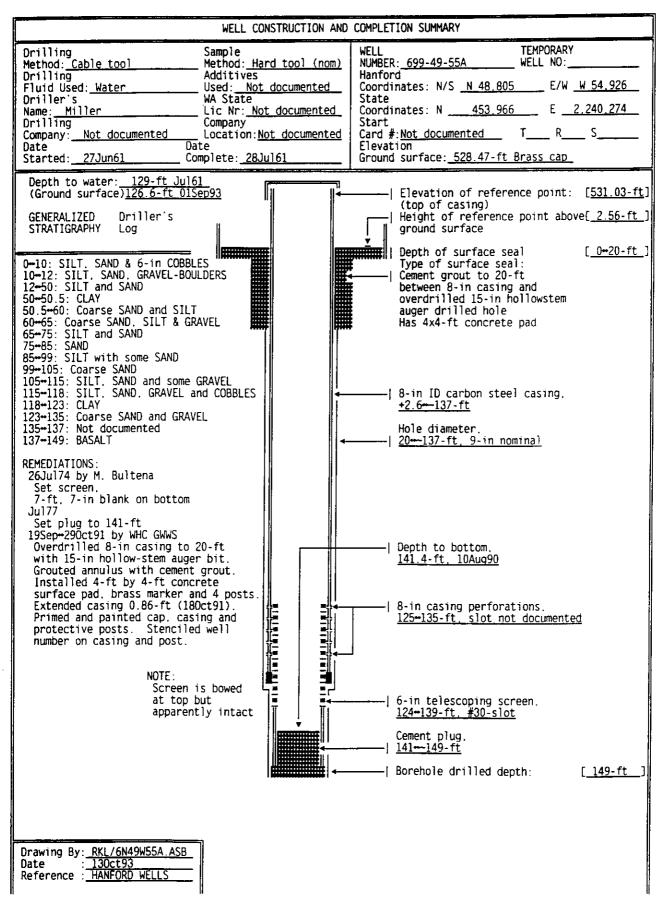
Geologist, driller AVAILABLE LOGS TV SCAN COMMENTS Not applicable Not applicable DATE EVALUATED EVAL RECOMMENDATION Not applicable

200-BP-1 Quarterly water level measurement, 01Mar91+12Mar93; LISTED USE

Not on PNL or WHC operational water sample schedule

PUMP TYPE Hydrostar MAINTENANCE

F-23 RL95-59.W51/A2



SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 699-49-55A

WELL DESIGNATION 699-49-55A RCRA FACILITY Not applicable

200-BP-1 (200 Aggregate Area Management Study) CERCLA UNIT

N 48,805 54.926 W

HANFORD COORDINATES LAMBERT COORDINATES E 2.240,274 [HANCONV] N 453,966

DATE DRILLED Ju161 149-ft DEPTH DRILLED (GS)

141.4-ft, Aug90 TV MEASURED DEPTH (GS) 129-ft. Jul61: DEPTH TO WATER (GS) : 126.6-ft, 01Sep93

8-in. carbon steel. +2.56-137-ft CASING DIAMETER ELEV TOP OF CASING ELEV GROUND SURFACE 531.03-ft. [25Feb92-NGVD'29] 528.47-ft. Brass cap [25Feb92-NGVD'29]

PERFORATED INTERVAL 125+135-ft

SCREENED INTERVAL 124-139-ft, Nominal 8-in telescoping.

#30-slot

FIELD INSPECTION, 110ct89. COMMENTS

Carbon steel casing.

No pad, capped and locked, no posts.

No permanent identification.
FIELD INSPECTION, 160ct92.
8-in carbon steel casing. Capped and locked

4-ft by 4-ft concrete pad, 4 posts. Identification stamped on brass marker in pad.

Not in radiation zone.

OTHER: Driller

Depths referenced to ground surface; TV SCAN COMMENTS

Dec89:

Depth to bottom: 139-ft Bottom of casing: 139-ft Depth to water: 122.5-ft

Casing weathered. Screen 121-137-ft. Screen is bowed in on one side

but intact. Crud collecting on screen. Blank 137-139-ft.

10Aug90:

Depth to bottom: 141.4-ft, soft floating stuff
Depth to water: 126-ft, floating debris
Casing clean. Screen 124-139-ft. Slots are open, screen looks good.

Water real good. 13Nov90

DATE EVALUATED

AVAILABLE LOGS

EVAL RECOMMENDATION :

Install a 2-in void grout surface seal outside 1.

8-in casing to approximately 18-ft. OR: Perforate 3-18-ft and install a 4-in liner with cement basket to approximately

20-ft. Pressure grout to 40-psi.

Install protective posts and concrete pad per WAC 173-160-510 and field

conditions.

Survey to water level measurement standards.

200 BP-1 quarterly water level measurement, 14Aug61+01Sep93 WHC ES&M ER characterization and w/l monitoring, LISTED USE CURRENT USER

PNL sitewide sampling 93 Hydrostar, intake at 137.9-ft. (top-of-casing) PUMP TYPE

17&19Apr90: Pulled pump, scrubbed casing and screen, bailed debris. MAINTENANCE

19&26Jun90: Developed well by pumping to <5 NTU.

REMEDIATION

13Aug90: Installed pump.
19Sep91: Cleared site.
24Sep91: Overdrilled 8-in casing with 15-in auger bit to 20-ft. Grouted annulus with cement grout (Al powder added).

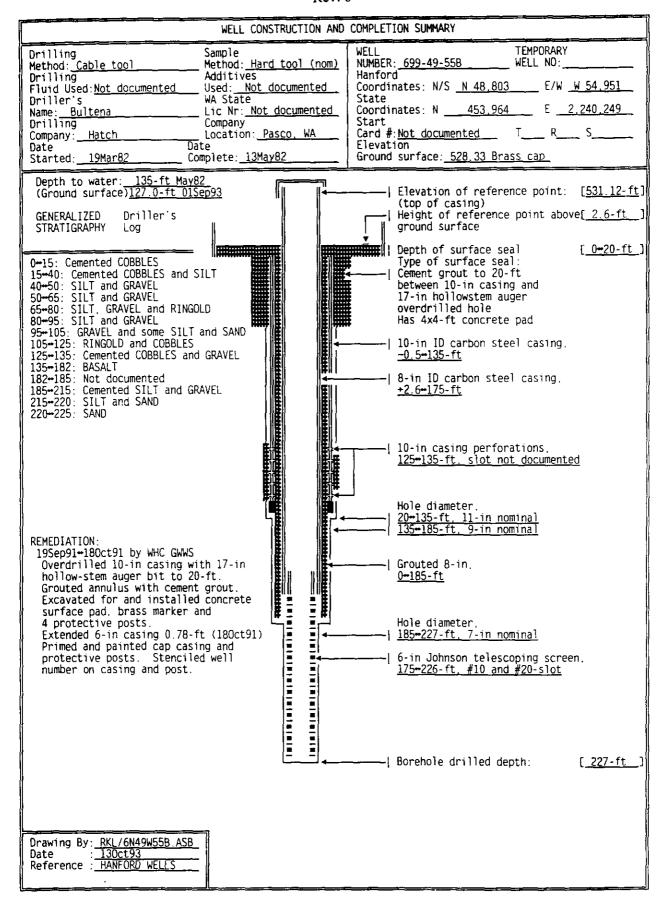
080ct91: Excavated for surface pad and protective posts.

110ct91: Installed 4-ft by 4-ft surface pad, brass marker and 4 protective posts. 180ct91: Extended casing 0.86-ft for total stickup of 2.52-ft.

27+290ct91; Primed and painted cap, casing and protective posts.

Stenciled well number on casing and post.

F-25



96.151.04.1211

DOE/RL-95-59 Rev. 0

SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 699-49-55B

699-49-55B WELL DESIGNATION Not applicable 200-BP-1 (200 Aggregate Area Management Study) RCRA FACILITY CERCLA UNIT HANFORD COORDINATES LAMBERT COORDINATES N 48,803 54,951 W E 2,240,249 [HANCONV] N 453,964 May82 227-ft DATE DRILLED DEPTH DRILLED (GS) 225-ft, Jun90 TV 135-ft, May82; MEASURED DEPTH (GS) : DEPTH TO WATER (GS) : 127.0-ft, 01Sep93 10-in, carbon steel ~0.5+135-ft; CASING DIAMETER 8-in, carbon steel, +2.6+175-ft 531.12-ft, ELEV TOP OF CASING [25Feb92-NGVD:29] ELEV GROUND SURFACE : 528.53-ft, Brass cap [25Feb92-NGVD'29] PERFORATED INTERVAL : None 175+226-ft, 6-in telescoping, SCREENED INTERVAL #10 and 20-slot, Johnson. Blank 170+175-ft. FIELD INSPECTION, 07Mar91. COMMENTS Carbon steel casing. (2). No pad, capped and locked, no posts. No permanent identification. FIELD INSPECTION, 160ct92, 8-in carbon steel casing. Capped and locked 4-ft by 4-ft concrete pad, 4 posts. Identification stamped on brass marker in pad. Not in radiation zone. OTHER: Driller AVAILABLE LOGS TV SCAN COMMENTS Depths referenced to ground surface: Jan90: Depth to bottom: 223-ft Depth to water: 123-ft Top of screen 175-ft, needs scrubbing. 18Jun90: Depth to bottom: 225-ft Depth to water: 124.3-ft, some floating debris but fairly clean. Casing clean. Screen 175.9-225-ft, clean. Water clear. Camera runs were not successful in obtaining pictures until 2 weeks after development. DATE EVALUATED 13Nov90 EVAL RECOMMENDATION : Install a 2-in void grout surface seal outside 8-in casing to approximately 18-ft, OR; Perforate 3-18-ft and install a 4-in liner with cement basket to approximately 20-ft. Pressure grout to 40-psi. Install protective posts and concrete pad per WAC 173-160-510 and field conditions. Shorten monitored interval to approximately 15-ft by plugging back to approximately 205-ft below ground surface. Survey to water level measurement standards. 200 BP-1 quarterly water level measurement, 25May82+01Sep93 LISTED USE CURRENT USER WHC ERcharacterization and ES&M w/l monitoring. PNL sitewide sampling and w/l monitoring 93 PUMP TYPE Electric submersible 12&16Apr90; Brushed casing and screen, bailed debris. MAINTENANCE 31May90; Developed well by pumping to minimum of 9.4 NTU. 01Jun90: Attempted camera survey. 13Aug90: Installed pump. 19Sep91: Cleared site.

REMEDIATION

20-26Sep91: Overdrilled 10-in casing with 17-in auger bit to 19-ft.

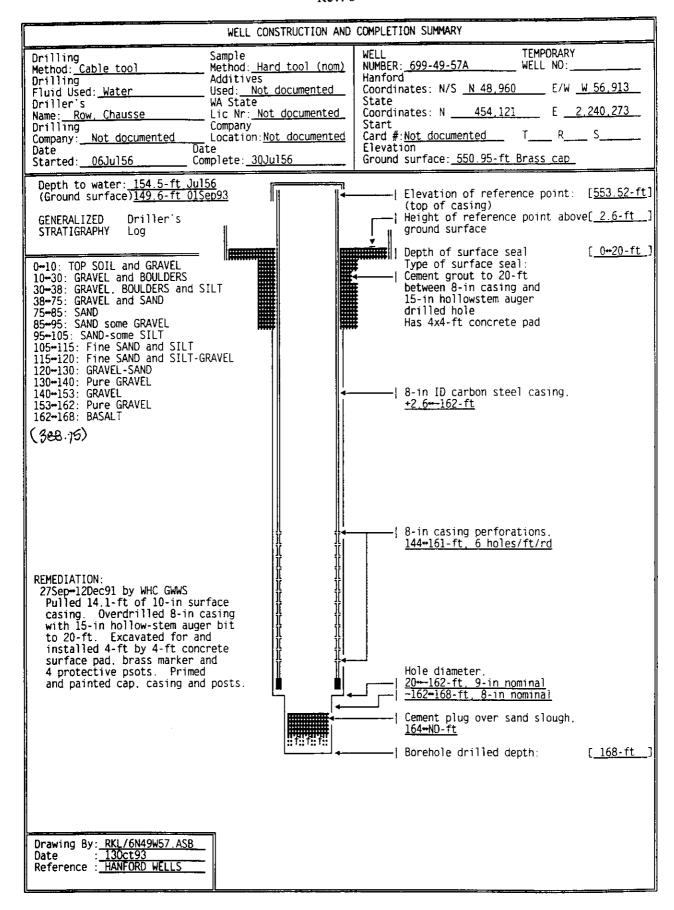
Grouted annulus with cement grout (Al powder added).

080ct91; Excavated for surface pad and protective posts.
110ct91; Installed 4-ft by 4-ft surface pad, brass marker and 4 protective posts.
180ct91; Extended 6-in casing 0.78-ft for 2.52-ft total stickup.

27-290ct91: Primed and painted cap, casing and protective posts.

Stenciled number on casing and post.

F-27



46 BASA 72 P

DOE/RL95-59 Rev. 0

SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 699-49-57A

699-49-57A WELL DESIGNATION RCRA FACILITY CERCLA UNIT Not applicable

200-BP-1 (200 Aggregate Area Management Study)

56,913 N 48.960 HANFORD COORDINATES W

E 2,240,273 [HANCONV] LAMBERT COORDINATES N 454,121

Ju156 DATE DRILLED DEPTH DRILLED (GS) MEASURED DEPTH (GS) 168-ft

164.6-ft, Oct89 TV 154.5-ft, Ju156; DEPTH TO WATER (GS) : 149.6-ft, 01Sep93

8-in, carbon steel, +2.57--162-ft CASING DIAMETER 553.52-ft. [25Feb92-NGVD 29] 550.95-ft. Brass cap [25Feb92-NGVD 29] ELEV TOP OF CASING ELEV GROUND SURFACE

144-161-ft PERFORATED INTERVAL SCREENED INTERVAL Not applicable

FIELD INSPECTION, 110ct89. COMMENTS

Carbon steel casing. No pad, capped and locked, no posts.

No permanent identification.
FIELD INSPECTION. 160ct92.
8-in carbon steel casing. Capped and locked

4-ft by 4-ft concrete pad, 4 posts. Identification stamped on brass marker in pad.

Not in radiation zone.

OTHER: Driller

AVAILABLE LOGS TV SCAN COMMENTS

Nov89, depths referenced to ground surface:

Depth to bottom: 164.6-ft Depth to water: ~148.5-ft

Perforations start at ~142-ft and ended at ~162-ft. Perforations could be

seen to bottom of casing. Well casing fairly clean of scale.

DATE EVALUATED 13Nov90

EVAL RECOMMENDATION:

Pull 10-in casing and install a 2-in void grout surface seal outside 8-in casing to at least 18-ft. OR:

Perforate 3-18-ft and install a 4-in liner with cement basket to approximately

20-ft. Pressure grout to 40-psi.

Install protective posts and concrete pad per WAC 173-160-510 and field

conditions.

Survey to water level measurement standards.

200 BP-1 quarterly water level measurement, 22Jun64+01Sep93 LISTED USE

CURRENT USER WHC ER characterizatoin and ES&M w/l monitoring,

PNL sitewide sampling 93 Electric submersible PUMP TYPE 130ct89; Pulled pump. MAINTENANCE

22Nov89: Installed pump. REMEDIATION 27Sep91; Cleared site.

11Nov91; Pulled 14.1-ft of 10-in casing.

Overdrilled 8-in casing to 20-ft with 15-in hollow-stem auger bit.

12Nov91; Grouted annulus with cement grout (Al powder added).

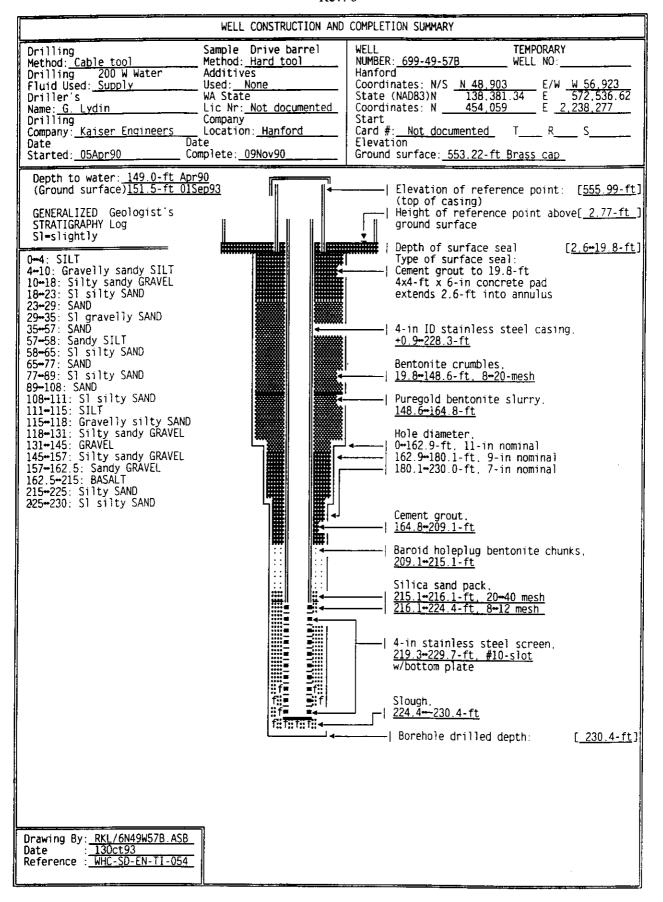
14-15Nov91: Excavated for surface pad. annular extension and protective posts. 20Nov91: Installed 4-ft by 4-ft concrete surface pad. annular extension.

brass marker and 4 protective posts.

04+05Dec91: Extended 8-in casing 0.72-ft for total of 2.5-ft stickup. Stamped casing and brass marker with well number. Removed form.

11-12Dec91; Primed and painted cap, casing and posts.

F-29 RL95-59.W51/A2



9613494.0213

DOE/RL-95-59 Rev. 0

SUMMARY OF CONSTRUCTION DATA AND FIELD OBSERVATIONS RESOURCE PROTECTION WELL - 699-49-57B

WELL DESIGNATION : 699-49-57B

CERCLA UNIT 200-BP-1 (200 Aggregate Area Management Study)

RCRA FACILITY : Not applicable

HANFORD COORDINATES: N 48.903 W 46.923 [25Jan91-200E]
LAMBERT COORDINATES: N 454.059 E 2.238.277 [HANCONV]
N 138.381.3m E 572.536.6m [25Jan91-NAD83]

DATE DRILLED : Nov90 DEPTH DRILLED (GS) : 230.4-ft

MEASURED DEPTH (GS): Not documented DEPTH TO WATER (GS): 149.0-ft. 19Apr90: 151.5-ft. 01Sep93

CASING DIAMETER : 4-in stainless steel. +0.9+228.3-ft: 6-in stainless steel. +2.77-0.5-ft

ELEV TOP CASING : 555.99-ft, [25Jan91-NGVD'29] ELEV GROUND SURFACE : 553.22-ft, Brass cap [25Jan91-NGVD'29]

PERFORATED INTERVAL: Not applicable

SCREENED INTERVAL : 219.3 229.7-ft, 4-in #10-slot stainless steel COMMENTS : FIELD INSPECTION, 07Mar91:

COMMENTS: FIELD INSPECTION, 07Mar91; 6-in stainless steel casing. 4-ft by 4-ft concrete pad, 4 posts, 1 removable

capped and locked, brass cap in pad with well ID.

Not in radiation zone.

OTHER:

AVAILABLE LOGS : Geologist, driller TV SCAN COMMENTS : Not applicable DATE EVALUATED : Not applicable EVAL RECOMMENDATION : Not applicable

LISTED USE : 200-BP-1 quarterly water level measurement. 01Mar91+01Sep93:

CURRENT USER : WHC ER characterizatin and ES&M w/l monitoring,

PNL sitewide sampling 93

PUMP TYPE : Hydrostar

MAINTENANCE :

RL95-59, W51/A2 F-32

9613494 0214 DOE/RL-95-59 Rev. 0

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